February 25, 1991

W. G. Hairston, III

ELV-02566 0850

Docket No. 50-424

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D. C. 20555

Gentlemen:

VOGTLE ELECTRIC GENERATING PLANT LICENSEE EVENT REPORT PERSONNEL ERROR RESULTS IN AUXILIARY FEEDWATER SYSTEM ACTUATION

In accordance with 10 CFR 50.73, Georgia Power Company hereby submits the enclosed report related to an event which occurred on February 4, 1991.

Sincerely, U.K.M Con For

WGH, III/NJS/gm

Enclosure: LER 50-424/1991-002

xc: Georgia Power Company Mr. C. K. McCoy Mr. W. E. Shipman Mr. P. D. Rushton Mr. R. M. Odom

NORMS

U. S. Nuclear Regulatory Commission Mr. S. D. Ebneter, Regional Administrator Mr. D. S. Hood, Licensing Project Manager, NRR Mr. B. R. Bonser, Senior Resident Inspector, Vogtle

80 Eoc 6-89)	Ecom 366 89)			U.S. NUCLEAR REGULATORY COMMISSION						APPROVED ONB NO. 3150-0104 EXPIRES: 4/30/92								
		1	LICE	NSE	EE	V ZN	T RE	PORT	(L	ER)		- sectors						
ACTU	TY NA	HE (1)		xx700.1	2 17 12	(115) 1	0.013/1		C DI A	T - UNI	T 1	1	SCRET NUMBE	R (2) _ PAGE (3)				
TYLE	(4)			VUG114	C ELLE	GIRI .	C GEAR	2041114	6 FLA	NI - UNI	1 1	10	50004	1 2 4 1 0F 3				
			RESU							TEM ACT	ATION			a de la compositiva d				
	T DATE				NUMBE		-		RT DAT					INVOLVED (8)				
IONTH	DAY	YEAR	YEAR	58	IQ NUN		REV	MONTH	DAY	YEAR	P.	ACILITY NAM	5	0 5 0 0 0				
0 2	04	91	91	0	0 2		0 0	02	25	91				05000				
OPE	RATING		THIS	REPOR	1 1 5 1	SUBMI	TIED P	URSUAN	T TO T	HE REQUI	REMENT	S OF 10 CFR	(11)					
MODE (9) 1			20.402(b)			20.405(c)			X	50.73(8)(2)(iv)	73.71(b)						
POWER 20.			presented in the second s				50.36	prises.			50.73(a)(2		73.71(e)					
			designed and	0.405(-		(c)(2)			50.73(a)(2		OTHER (Specify in				
			accessed.	0.405(M	(a)(2)		-	50.73(a)(2		Abstract below)				
				0.405(0.405(×.000		(a)(2) (a)(2)			50.73(a)(2 50.73(a)(2						
				i ristriaini d			LICENSE	E CON	ACT F	OR THIS	ER (1)	2)	denise adverse source and	And the second second second section and in an exact				
NAME	a realization											on and incoments on a proper limit of		ELEPHONE NUMBER				
													AREA CODE					
R.	M. 01	DOM, N	UCLEA						N FALL	UPE DECC	DIDED	IN THIS REP	404	826-3201				
	1	1			UFAC-		EPORT	T	n rais				MANUFAC-	REPORT				
CAUSE	SYSTE	M COMP	ONENT	TUR			D NPRDS			CAUSE	SYSTEM	COMPONENT	TURER	TO NPRDS				
							AND ADDRESS	-										
					and the second se							and the second	and a second sec					
	1			SUP	PLEME	NTAL	REPORT	EXPEC	TED (14.)		L		MONTHI DAY IYEA				
									T	anna an			EXPECTED SUBMISSION					
Y [ESCIE	yes, c	omple	te EXP	ECTED	SUB	AISSION	DATE) - 13	ON D			DATE (15)					

On 2-4-91, plant personnel were performing surveillance testing in the Train B Safeguards Test Cabinet (STC). The Balance of Plant (BOY) Operator and a trainee were in the process of testing a slave relay. The trainee placed his hand on the test button, S935, then removed it to re-verify the step in the procedure. He then erroneously put his hand on the button directly below the proper button, placed it in the test position, and manually depressed it at 0036 CST, before either the trainee or the BOP operator recognized the error. This button (S928) energized a different slave relay which caused the steam inlet valve to the Turbine Driven Auxiliary Feedwater Pump (TDAFWP) to open, starting the pump. The Reactor Operator (RO) in the control room observed the alarm, checked that steam generator (SG) water levels were normal, and took action to limit the TDAFWP discharge to the SGs. After determining the reason for the TDAFWP actuation, the RO secured the TDAFWP and restored the Auxiliary Feedwater System to standby readiness at 0119 CST.

The trainee committed a personnel error by inadvertently failing to follow procedure and in not employing self-checking to verify the button number prior to depressing it. Also, the BOP operator did not exercise sufficient supervision of the trainee. The trainee and the BOP operator have been counseled.

(6-89) LICENSEE EVENT REF TEXT CONTINUAT	S. NUCLEAR REGULATORY COMMISSION ORT (LER) ION			APPROVED EXP IRI	OHE:	NO 3150- 4/30/92	0104			
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5) P							AGE (3	
		YEAR		SEQ NUM		REV		Π		
VOGTLE ELECTRIC GENERATING PLANT - UNIT 1	05000424	91		002		0.0	2	0F	3	

A. REQUIREMENT FOR REPORT

This report is required per 10 CFR 50.73 (a)(2)(iv) because an unplanned Engineered Safety Feature actuation occurred.

D. UNIT STATUS AT TIME OF EVENT

At the time of this event, Unit 1 was operating in Mode 1 (Power Operations) at 100% of rated thermal power. There was no inoperable equipment which contributed to the occurrence of this event.

C. DESCRIPTION OF EVENT

On 2-4-91, plant personnel were performing Solid State Protection System (SSPS) surveillance testing in the Train B Safeguards Test Cabinet (STC), per procedure 14649-1, "SSPS Slave Relay K746 Train B Test Containment Ventilation Isolation." The Balance of Plant (BOP) Operator and a trainee were in the process of testing a slave relay which is used to actuate containment ventilation isolation. The trainee placed his hand on the correct test button, S935, then removed it to re-verify the step in the procedure. He then erroneously put his hand on the button directly below the proper button, placed it in the test position, and manually depressed it at 0036 CST, before either the trainee or the BOP operator recognized the error. This button (S928) energized slave relay K641 and sequentially, the AX2 relay. This created an opening permissive for steam inlet valve 1HV-5106, which started the Turbine Driven Auxiliary Feedwater Pump (TDAFWP). The Reactor Operator (RO) in the control room acknowledged an alarm which indicated that the inlet valve was opening, checked to determined that steam generator (SG) water levels were normal, and took action to limit the TDAFWP discharge to the SGs. This action included decreasing the TDAFWP speed and manually closing the TDAFWP discharge valves As a result, there was no noticeable change in steam generator water levels or reactor power as a consequence of the pump start. After determining the reason for the TDAFWP actuation, the RO secured the TDAFWP and restored the Auxiliary Feedwater System (AFW) to standby readiness at 0119 CST.

D. CAUSE OF EVENT

The causes of the event are:

 The Georgia Power Company trainee committed a cognitive personnel error by inadvertently failing to follow procedure 14649-1 and in not employing self-checking to verify the button number prior to depressing it. The BOP operator committed a cognitive personnel error by failing to exercise sufficient supervision over the trainee. There were no unusual characteristics of the work location which contributed to the occurrence of these errors.

U.S. NUCLEAR REGULATORY COMMISSION (6-89) LICENSEE EVENT REPORT (LER) TEXT CONTINUATION						APPROVED OMB NO 3150-0104 EXPIRES: 4/30/92									
ACILTITY	NAME (1		DOCKET NUMBER (2)		T	PAGE (3									
		김 씨와 집에 많은 것이 같아요. 같이 많이		YEAR		SEQ	NUM		REV			1	-		
VOGTLE	ELEC A	IC GENERATING PLANT - UNIT 1	05000424	9 1		0	0 2		0	0	3	OF	3		
	2.	Although a lesson plan in developed and training was group of personnel who had class.	in progress, the t	raine	e w	as	one	of	a	sma					

E. ANALYSIS OF EVENT

The AFW system started as designed and the control room personnel responded properly to throttle flow to the SGs and prevent a plant transient. Based on these considerations, there was no adverse effect on plant safety or the health and safety of the public as a result of this event.

F. CORRECTIVE ACTIONS

- The trainee has been counseled regarding the importance of self-checking. The BOP operator has been counseled on the importance of exercising proper supervision of trainees and his responsibility for all trainee actions. By 4-1-91, a summary of this event will become required reading, or will be discussed in group meetings, for Operations, Maintenance, HP/Chemistry and Engineering Support departments' personnel, stressing the importance of self-checking and attention to detail.
- By 5-1-91, self-checking/verification training will be sequenced into the initial licensed operator training program for on the job training in the control room.
- Each SSPS actuation switch will be border marked to visually block in its nameplate, switch, and switch number (both units) by 5-1-91.

G. ADDITIONAL INFORMATION

1. Failed Components:

None

2. Previous Similar Events:

LER 50-424/1987-015, dated 5-13-87. Corrective actions included counseling.

3. Energy Industry Identification System Code:

Solid State Protection System - JG

Auxiliary Feedwater System - BA

Containment Isolation Control System - JM