



Carolina Power & Light Company

Brunswick Nuclear Project
P. O. Box 10429
Southport, N.C. 28461-0429
September 28, 1990

FILE: B09-13510C
SERIAL: BSEP/90-0665

10CFR50.73

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 2
DOCKET NO. 50-325
LICENSE NO. DPR-62
LICENSEE EVENT REPORT 2-90-012

Gentlemen:

In accordance with Title 10 of the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is submitted in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

J. L. Harness, General Manager
Brunswick Nuclear Project

TH/th

Enclosure

cc: Mr. S. D. Ebnetter
Mr. N. B. Le
BSEP NRC Resident Office

00201

9010050280 900928
PDR ADOCK 05000324
S PDC

IE22
11

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 800 LINES FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-430), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)

Brunswick Steam Electric Plant, Unit 2

DOCKET NUMBER (2)

0 5 0 0 0 3 2 4 1 OF 0 1

PAGE (3)

TITLE (4)

Unit 2 Reactor Scram During Reactor Start Up

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER (5)																														
08	30	90	90	012	000	09	28	90			0 5 0 0 0																														
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following): (11)																																						
2			<table border="0"><tr><td>20.402(a)</td><td>20.405(x)</td><td>X</td><td>80.73(a)(2)(iv)</td><td>73.71(b)</td></tr><tr><td>20.405(a)(1)(i)</td><td>80.36(a)(1)</td><td></td><td>80.73(a)(2)(v)</td><td>73.71(c)</td></tr><tr><td>20.405(a)(1)(ii)</td><td>80.36(a)(2)</td><td></td><td>80.73(a)(2)(vi)</td><td>OTHER (Specify in Abstract below and in Text, NRC Form 306A)</td></tr><tr><td>20.405(a)(1)(iii)</td><td>80.73(a)(2)(i)</td><td></td><td>80.73(a)(2)(vii)(A)</td><td></td></tr><tr><td>20.405(a)(1)(iv)</td><td>80.73(a)(2)(ii)</td><td></td><td>80.73(a)(2)(vii)(B)</td><td></td></tr><tr><td>20.405(a)(1)(v)</td><td>80.73(a)(2)(iii)</td><td></td><td>80.73(a)(2)(ix)</td><td></td></tr></table>									20.402(a)	20.405(x)	X	80.73(a)(2)(iv)	73.71(b)	20.405(a)(1)(i)	80.36(a)(1)		80.73(a)(2)(v)	73.71(c)	20.405(a)(1)(ii)	80.36(a)(2)		80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)	20.405(a)(1)(iii)	80.73(a)(2)(i)		80.73(a)(2)(vii)(A)		20.405(a)(1)(iv)	80.73(a)(2)(ii)		80.73(a)(2)(vii)(B)		20.405(a)(1)(v)	80.73(a)(2)(iii)		80.73(a)(2)(ix)	
20.402(a)	20.405(x)	X	80.73(a)(2)(iv)	73.71(b)																																					
20.405(a)(1)(i)	80.36(a)(1)		80.73(a)(2)(v)	73.71(c)																																					
20.405(a)(1)(ii)	80.36(a)(2)		80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)																																					
20.405(a)(1)(iii)	80.73(a)(2)(i)		80.73(a)(2)(vii)(A)																																						
20.405(a)(1)(iv)	80.73(a)(2)(ii)		80.73(a)(2)(vii)(B)																																						
20.405(a)(1)(v)	80.73(a)(2)(iii)		80.73(a)(2)(ix)																																						
POWER LEVEL (10)																																									
0 0 8																																									

LICENSEE CONTACT FOR THIS LER (12)

NAME

Terri Jones, Regulatory Compliance Specialist

TELEPHONE NUMBER

AREA CODE 9 1 9 4 5 7 - 2 0 3 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	J K	L C V	F 1 3 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			1	1	0 1 9 0

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 30, 1990, Unit 2 reactor start-up was in progress. The reactor was at approximately 8% power and 300 psig. The Emergency Core Cooling Systems were operable in standby line up except for the High Pressure Coolant Injection System which was inoperable due to not performing surveillance test PT-09.2. At 1656 the start-up level control valve (SULCV) failed closed resulting in a level transient. At 1657 the Reactor Protection System (RPS) low level #1 setpoint (165") was reached causing a reactor scram. Primary Containment Isolation System groups 2, 6 and 8 also received an isolation signal and actuated per design. Scram recovery was in accordance with the emergency flowcharts and procedures. Approximately 20 minutes after the scram level was stabilized. An investigation into the failure of the SULCV is continuing. The safety significance of this event is minimal. Level was recovered without the need for safety system injection and the unit is designed for a level transient from full power. A supplement to this report will be issued by November 1, 1990.