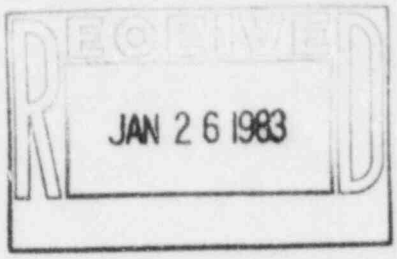
Public Service Company of Colorado

16805 WCR 19 1/2, Platteville, Colorado 80651-9298

January 10, 1983
Fort St. Vrain
Unit No. 1
P-83014



Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Fort St. Vrain Nuclear
Generating Station
Public Service Company of Colorado
Docket No. 50-267
Operating License DPR-34

Dear Sir:

Attached please find the December, 1982, Monthly Operating Information for the Fort St. Vrain Nuclear Generating Station.

Very truly yours,

Don Warembourg by Milt McBride

Don Warembourg
Manager, Nuclear Production

DW/arh

Attachment

cc: Mr. John T. Collins
Office of MIPC

A005

OPERATING DATA REPORT

DOCKET NO. 50-267
DATE January 10, 1983
COMPLETED BY L. M. McBride
TELEPHONE (303) 785-2224

OPERATING STATUS

NOTES

1. Unit Name: Fort St. Vrain
2. Reporting Period: 821201 through 821231
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

   None

9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: Restriction to 70% pending resolution of contractual matters

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	8760	30721
12. Number of Hours Reactor Was Critical	423.3	5168.3	19746.7
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	0.0	3266.2	13174.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	6,303.1	1,855,480.6	6,789,425.4
17. Gross Electrical Energy Generated (MWH)	0	635,548	2,326,904
18. Net Electrical Energy Generated (MWH)	-4,234	568,851	2,123,110
19. Unit Service Factor	0.0	37.3	42.9
20. Unit Availability Factor	0.0	37.3	42.9
21. Unit Capacity Factor (Using MDC Net)	0.0	19.7	20.9
22. Unit Capacity Factor (Using DER Net)	0.0	19.7	20.9
23. Unit Forced Outage Rate	100.0	44.3	36.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Plant recovery 830101 - 11.4 hours. Surveillance testing 830323 through 830412 - 504.0 hours.
25. If Shut Down at End of Report Period, Estimated Date of Startup: Reactor is critical at low power.
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

Dupl of 8301280080

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-267

Unit Fort St. Vrain

Date January 10, 1983

Completed By L. M. McBride

Telephone (303) 785-2224

Month December, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.0	17	0.0
2	0.0	18	0.0
3	0.0	19	0.0
4	0.0	20	0.0
5	0.0	21	0.0
6	0.0	22	0.0
7	0.0	23	0.0
8	0.0	24	0.0
9	0.0	25	0.0
10	0.0	26	0.0
11	0.0	27	0.0
12	0.0	28	0.0
13	0.0	29	0.0
14	0.0	30	0.0
15	0.0	31	0.0
16	0.0		

*Generator on line but no net generation.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-267
UNIT NAME Fort St. Vrain
DATE January 10, 1983
COMPLETED BY L. M. McBride
TELEPHONE (303) 785-2224

REPORT MONTH December, 1982

NO.	DATE	TYPE	DURATION	REASON	METHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM CODE	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
82-01	821201	F	744.0	H	3	N/A	IBH	INSTRU	Loop 1 shutdown followed by reactor scram and turbine-generator trip on 820930. Outage continued due to primary coolant chemistry impurities and repair of steam generator.

REFUELING INFORMATION

1. Name of Facility.	Fort St. Vrain Unit No. 1
2. Scheduled date for next refueling shutdown.	September 1, 1983
3. Scheduled date for restart following refueling.	November 1, 1983
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?	Yes
If answer is yes, what, in general, will these be?	Use of type H-451 graphite.
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10CFR Section 50.59)?	----------
If no such review has taken place, when is it scheduled?	----------
5. Scheduled date(s) for submitting proposed licensing action and supporting information.	Not scheduled at this time; to be determined.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	----------
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	1482 HTGR fuel elements. 11 spent HTGR fuel elements
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.

REFUELING INFORMATION (CONTINUED)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1992 under Agreements AT(04-3)-633 and DE-SC07-79ID01370 between Public Service Company of Colorado, General Atomic Company, and DOE.*

* The 1992 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accomodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1992.