

APPENDIX

U.S. NUCLEAR REGULATORY COMMISSION
REGION IV

Report: 50-298/82-37

Docket: 50-298

Licensee: Nebraska Public Power District
P. O. Box 499
Columbus, Nebraska 68601

Facility Name: Cooper Nuclear Station

Inspection At: Cooper Nuclear Station, Nemaha County, Nebraska

Inspection Conducted: December 1-31, 1982

Inspector: D. L. DuBois
D. L. DuBois, Senior Resident Reactor
Inspector

1/6/83
Date

Approved: T. F. Westerman
T. F. Westerman, Chief
Reactor Project Section A

1/11/83
Date

Inspection Summary

Inspection Conducted December 1-31, 1982 (Report 50-298/82-37)

Areas Inspected: Routine, announced inspection of operational safety verifications, monthly surveillance and maintenance observations, licensee event followup, followup of previously identified items, and TMI action plan requirements. This inspection involved 31 inspector-hours onsite by one NRC inspector.

Results: Within the areas inspected no violations or deviations were identified.

DETAILS1. Persons Contacted

*L. Lessor, Plant Superintendent
P. Thomason, Ass't. Plant Superintendent
K. Wire, Operations Supervisor
V. Wolstenholm, QA Supervisor
J. Sayer, C & HP Supervisor

*Indicates presence at exit meetings.

2. Operational Safety Verification

The NRC inspector observed control room operations, instrumentation, controls, reviewed applicable logs, and conducted discussions with control room operators. The NRC inspector verified operability of:

Automatic Depressurization System
"A" & "B" Core Spray Systems
Standby Liquid Control System
Number 1 Diesel Generator

The NRC inspector reviewed safety clearance records, including verification that affected components were removed from and returned to service in a correct and approved manner, that redundant equipment was verified operable, and that limiting conditions for operation were adequately identified and maintained. The NRC inspector also verified that maintenance requests had been initiated for equipment discovered to require repair or routine preventive upkeep, appropriate priority was assigned, and maintenance commenced in a timely manner commensurate with assigned priorities.

Tours of accessible areas of the facility were conducted to observe normal security practices, plant and equipment conditions, including cleanliness, radiological controls, fire suppression systems, emergency equipment, potential fire hazards, fluid leaks, excessive vibration and instrumentation adequacy.

These reviews and observations were conducted to verify that facility operations were in conformance with the requirements established in the Technical Specification, 10 CFR, and Administrative Procedures.

No violations or deviations were identified in this area.

3. Monthly Surveillance Observation

The NRC inspector observed portions of Technical Specification required surveillance tests to verify that testing was performed in accordance with adequate procedures, test instrumentation was in calibration, limiting conditions for operations were met, removal and subsequent restoration of affected components was accomplished, test results conformed with Technical Specification and procedure requirements, tests were reviewed by

personnel other than the person directing the test, and deficiencies identified during testing were properly reviewed and resolved by appropriate management personnel.

The following Surveillance Tests were selected and observed:

- 6.2.1.4.2 PCIS MSL High Flow Calibration and Functional Test
- 6.2.2.2.7 Safety Valve and Relief Valve Position Indicating Operability Check
- 6.3.12.1 Number 1 Diesel Generator Operability Test

These reviews and observations were conducted to verify that facility operations were in conformance with the requirements established in the Technical Specification, 10 CFR, and administrative procedures.

No violations or deviations were identified in this area.

4. Monthly Maintenance Observations

The following clearance orders were independently verified for proper placement/restoration of affected components:

- 82-791, "C" Sparger Pump
- 82-793, Reactor Core Isolation Cooling Pump
- 82-799, Fire Protection System
- 82-801, Post Accident Sampling System

Included with the above were checks for availability of redundant equipment, adequate safety isolation and clearance, accomplishment of work in accordance with approved procedures and Technical Specification requirements, verification that QC checks were performed as required, cleanliness controls and health physics coverages were adequate.

No violations or deviations were identified in this area.

5. Licensee Event Followup

The following LER's are closed on the basis of the NRC inspector's inoffice review, review of licensee documentation, and discussions with licensee personnel:

- LER 82-09 Failure of Suppression Chamber Level Instrument PC-LI-13
- LER 82-10 Trip of Diesel Generator #1
- LER 82-11 Failure of Recirculation System Sample Valve KR-740AV to Close
- LER 82-14 Abnormal Degradation of Reactor Water Cleanup System Piping

6. Followup of Previously Identified Items

Open Item 8112-01 (Closed) Installation of Standby Gas Treatment System (SGTS) By-Pass Valves

In NRC Inspection Report 50-298/81-12, paragraph 10, the NRC inspector documented inspection results applicable to TMI Action Plan Item II.E.4.2, titled

"Interim Position for Containment Purge and Vent Valve Operation." Included was information concerning the installation of SGTS by-pass valves.

During the 1981 review of Minor Design Change (MDC) 80-003, titled "Standby Gas Treatment System Cross Connect to Spare Pipe to Elevated Release Point," several items remained incomplete pending the licensee's final review and approval of the MDC 80-003 completion report. The NRC inspector has subsequently verified that operating procedures and as-built drawings were revised and that operations personnel received training applicable to MDC 80-003.

7. NUREG - 0737, TMI Action Plan Requirements

Item II.B.3 Section 2.B (Closed) Post Accident Sampling - Modify

NUREG - 0737, Item II.B.3, Post Accident Sampling Capability, requires the licensee to have the capability to promptly obtain reactor coolant and containment atmosphere samples under accident conditions.

In NRC Inspection Report 50-298/82-05, paragraph 7, the NRC inspector verified that the reactor coolant and containment atmosphere post accident sampling system (PAS) was installed and operational. Subsequent concerns were identified by the NRC Region IV staff concerning the ability of the licensee's staff to obtain samples in the event of a complete loss of onsite and offsite power. In a letter from Pilant to Vassallo, dated September 1, 1982, the licensee provided the NRC with additional answers to the criteria applicable to this item. Also, the NRC inspector reviewed CNS Chemistry Procedure 8.4.1.1, Revision 1, dated July 7, 1982, "Post Accident Sampling System Reactor Coolant and Containment Atmosphere Sampling," and verified that provisions for sampling during a complete loss of site power appear adequate.

The NRC inspector's review of the installed equipment and related documentation verified that the licensee is in conformance with the requirements of NUREG - 0737, Item II.B.3 (2.B), and Task Item II.B.3 of TI2515/59.

8. Exit Meetings

Exit meetings were conducted at the conclusion of each portion of the inspection. The plant superintendent was informed of the above findings.

