



**GPU Nuclear**  
P.O. Box 388  
Forked River, New Jersey 08731  
609-693-6000  
Writer's Direct Dial Number:

December 29, 1982

Mr. Ronald C. Haynes, Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
131 Park Avenue  
King of Prussia, PA 19406

Dear Mr. Haynes:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Licensee Event Report  
Reportable Occurrence No. 50-219/82-58/03L

This letter forwards three copies of a Licensee Event Report (LER) to report Reportable Occurrence No. 50-219/82-58/03L in compliance with paragraph 6.9.2.b.2 of the Technical Specifications.

Very truly yours,

*Peter B. Fiedler*  
Peter B. Fiedler  
Vice President and Director  
Oyster Creek

PBF:lse  
Enclosures

cc: Director (40 copies)  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Director (3)  
Office of Management Information and  
Program Control  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

NRC Resident Inspector  
Oyster Creek Nuclear Generating Station  
Forked River, NJ 08731

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OYSTER CREEK NUCLEAR GENERATING STATION  
Forked River, New Jersey 08731

Licensee Event Report  
Reportable Occurrence No. 50-219/82-58/03L

Report Date

December 29, 1982

Occurrence Date

November 28, 1982

Identification of Occurrence

During surveillance testing, main steam drain valves V-1-106 and V-1-110 failed to fully close when given the closed signal with its control switch. The valves were both secured and deactivated in the isolation position, which is a limiting condition for operation permitted by the Technical Specifications, paragraph 3.5.A.3.a.1.b.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.b.2.

Conditions Prior to Occurrence

The event occurred during routine startup operations.

Major Parameters

Reactor Power - 753 MWT  
Generator Load - 218 MWe  
Reactor Coolant Temperature - 525°F

Description of Occurrence

During performance of the "MSIV Closure and IST Test", main steam drain valves V-1-106 and V-1-110 failed the acceptance criteria in the closing direction. Both valves would not fully close when given the appropriate signals. The two remaining main steam drain valves, V-1-107 and V-1-111, were both operable. Valves V-1-106 and V-1-110 were closed by bypassing the control circuitry, deactivated, and secured in their isolation position as required by the Technical Specifications.

Apparent Cause of Occurrence

The apparent cause of the occurrence is unknown at this time. The valves were stroked several times and current checks were performed in an attempt to discover the problem. Both valve operators did not exhibit their full load amperage values upon closing, and the charts taken during these readings were evaluated with no abnormalities noted. These checks were made with the reactor depressurized and reactor coolant temperature less than 212°F.

Analysis of Occurrence

The Main Steam drain valves are primary containment isolation valves, and, as such, are required to close when reactor isolation signals are present. In this case, valves V-1-106 and V-1-110 would not have fully closed. It was demonstrated, however, that the valves were capable of being closed by bypassing the control circuitry. Additionally, these valves are maintained in the closed position during power operation. Based on this, the safety significance of the event is considered minimal.

Corrective Action

Valves V-1-106 and V-1-110 were closed and tagged out of service, as required by the Technical Specifications. At the present time, all four (4) main steam drain valves are tagged in the closed (normal) position. An investigation as to the cause of this event will be continued during the upcoming refueling outage.