

GPU Nuclear
P.O. Box 388
Forked River, New Jersey 08731
609-693-6000
Writer's Direct Dial Number:

December 27, 1982

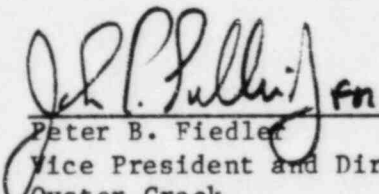
Mr. Ronald C. Haynes, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Mr. Haynes:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report
Reportable Occurrence No. 50-219/82-59/03L

This letter forwards three copies of a Licensee Event Report (LER) to report Reportable Occurrence No. 50-219/82-59/03L in compliance with paragraph 6.9.2.b.2 of the Technical Specifications.

Very truly yours,


Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:lse
Enclosures

cc: Director (40 copies)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director (3)
Office of Management Information and
Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731

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OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731

Licensee Event Report
Reportable Occurrence No. 50-219/82-59/03L

Report Date

December 27, 1982

Occurrence Date

November 24, 1982

Identification of Occurrence

"A" reactor recirculation pump was taken out of service. "A" loop was left idle but not isolated which is a limiting condition for operation permitted by Technical Specifications, paragraph 3.3.F.2.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.b.2.

Conditions Prior to Occurrence

The plant was operating at steady state power

Reactor Power - 1000 MWt
Generator Output - 306 MWe
Mode Switch Position - Run

Description of Occurrence

"A" reactor recirculation pump was taken out of service. The suction and bypass valves remained open.

Apparent Cause of Occurrence

"A" reactor recirculation pump was manually removed from service due to mechanical seal leakage, as indicated by seal cavity pressure readings and flow alarms. This action was taken due to an increasing drywell unidentified leak rate and the suspect indication exhibited in "A" recirculation pump seal.

Analysis of Occurrence

Reactor operation is permitted with four recirculation loops in service provided the idle recirculation loop is not isolated from the reactor vessel. As specified in Technical Specification 3.10.A, 4-loop operation imposes more restrictive MAPLHGR limits. The more restrictive limits were satisfied at the time of this occurrence, thus there is no safety significance.

Corrective Action

During a subsequent shutdown, "A" reactor recirculation pump seal was confirmed to be leaking and the seal cartridge assembly was replaced.

"A" reactor recirculation pump has been returned to service.

Failure Data

None