LICENSEE EVENT REPORT

CONTROL BLOCK:
V A S P S 1 2 0 0 - 0 0 0 - 0 0 3 4 1 1 1 1 4
CONT ARPORT L 6 0 5 0 0 0 2 8 1 7 1 2 1 9 8 2 E 1 2 3 0 78 2 E SOURCE SOURCE 60 DOCKET NUMBER 56 69 EVENT DATE 75 REPORT DATE 80
With unit 2 at HSD, while conducting the RCS Integrity Test (PT-11), a weld leak
on the 'A' steam generator channel head drain piping at 2-RC-159 was identified.
Immediate action was taken to return the unit to CSD. This event is reportable
pursuant to T.S.6.6.2.a.(3). The leakage was within the capability of the normal
old make up system, therefore, the health and safety of the public were not affected.
017
T S 9 SYSTEM CAUSE CAUSE COMP. VALVE
CODE SUBCODE S
17 REPORT S 2
ACTION FUTURE COMPONENT METHOD HOURS 22 ATTACHMENT NPRO-4 PRIME COMP. COMPONENT MANUFACTURER B 18 Z 19 C 20 Z 21 0 0 0 0 Y 23 N 24 N 25 W 1 2 0 26
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) THE cause of this event is believed to be poor fusion between successive passes
- in a small area of the weld. The unit was returned to CSD and the defective weld !
The second district Evenination of the final weld revealed as defeated
was repaired. Liquid renetrant Examination of the linar weld revealed no defects.
1 3
7 8 9 FACILITY STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32
1 5 G 28 O O O O O N/A B Periodic Test 11
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 N/A N/A N/A N/A 80
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39 N/A 11 12 13 15 15 15 15 15 15 15 15 15 15 15 15 15
PERSONNEL INJURIES NUMBER DESCRIPTION 41 N/A N/A
LOSS OF OR DAMAGE TO FACILITY (3) TYPE DESCRIPTION N/A
PUBLICITY SSUIED DESCRIPTION (5) B301070288 B21230 NRC USE ONLY PDR ADOCK 05000281 PDR
7 8 9 10 68 69 80 S

ATTACHMENT 1

SURRY POWER STATION, UNIT NO. 2

DOCKET NO: 50-281

REPORT NO: 82-075/01T-0 EVENT DATE: 12-19-82

TITLE OF THE EVENT: 2-RC-159 Weld Leak

1. DESCRIPTION OF THE EVENT:

With the unit at hot shutdown, while conducting the Reactor Coolant System (RCS) integrity test (Periodic Test 11) a socket weld leak on the 'A' steam generator channel head drain piping at 2-RC-159 was identified. This event is reportable in accordance with Technical Specification 6.6.2.a.(3).

2. PROBABLE CONSEQUENCES AND STATUS OF REDUNDANT EQUIPMENT:

2-RC-159 is connected to the steam generator through a 3/8" pipe approximately 5" long. A Westinghouse analysis has determined the normal charging system is capable of maintaining RCS inventory in the event of a 3/8" pipe break. Therefore the health and safety of the public were not affected.

3. CAUSE:

The cause of the event is believed to be inadequate fusion between successive weld passes. This may have been due to water being present at the time the weld was made. Since the defect did not extend to the weld surface it was not detected by the liquid penetrant examination done at the time.

4. IMMEDIATE CORRECTIVE ACTION:

Actions were taken to return the unit to cold shutdown.

5. SUBSEQUENT CORRECTIVE ACTION:

The weld was ground out in the area of the defect and repaired. A liquid Penetrant Examination of the final weld revealed no defects. Additionally, a hydrostatic test was successfully completed.

6. ACTIONS TAKEN TO PREVENT RECURRENCE:

None.

7. GENERIC IMPLICATIONS:

None.