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November 22, 1982
JPN-82-86

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Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Mr. Domenic B. Vassallo, Chief
Operating Reactors Branch No. 2
Division of Licensing

Subject: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
Safety/Relief Valve Leakage

Reference: PASNY letter, J.P. Bayne to D.B. Vassallo
dated August 12, 1982 (JPN-82-69)

Dear Sir:

As a result of recent FitzPatrick operating experience with safety/relief valve (SRV) leakage, there have been several discussions between members of your staff and Mr. J.A. Gray, Jr. of my staff. During these discussions, verbal agreements were reached which constitute an interim resolution of the issue. These agreements are restated in this letter to eliminate any potential for misunderstanding.

The Power Authority has committed to abide by the requirements of the attached draft changes to the FitzPatrick Technical Specifications, beginning October 22, 1982. These draft changes require prompt notification of elevated SRV tailpipe temperatures. It was understood that this reporting requirement was satisfied for the "D" SRV by virtue of the fact that the NRC was already aware of the condition.

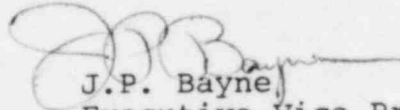
The attached draft changes also require the submission of an engineering evaluation for the corresponding temperature increase. This evaluation was to have been submitted in conjunction with a proposed license amendment which would have incorporated the attached reporting requirements into the Technical Specifications; and, permitted continued operation with any one SRV inoperable. Submittal of this proposed amendment is no longer being requested by the NRC staff. However, the engineering evaluation to fulfill the requirements of the attached changes is essentially the same as the safety evaluation submitted with the referenced letter.

A001

The Power Authority's commitments with regard to as-found testing of SRVs were stated in the referenced letter. The commitments remain unchanged.

If you have any questions, please contact Mr. J.A. Gray, Jr. of my staff.

Very truly yours,



J.P. Bayne
Executive Vice President
Nuclear Generation

cc: Mr. J. Linville
Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 136
Lycoming, N.Y. 13093

Mr. Ron Barton
United Engineers & Constructors, Inc.
30 S. 17th Street
Philadelphia, PA 19101

3.6 (cont'd)

5. If, for a period of longer than 24 hours, the temperature of any safety/relief discharge pipe is more than 40°F above its steady state value, or the acoustical monitor reading of any safety/relief valve discharge pipe is more than 3 times greater than its steady state value, the following actions shall be taken:
- a. a report shall be issued in accordance with 6.9.A.4.1 which addresses the actions that have been taken or a schedule of actions to be taken.
 - b. an engineering evaluation shall be performed justifying continued operation for the corresponding increase in temperature or acoustical monitor reading.
 - c. the affected safety/relief valve shall be removed at the next cold shutdown of 72 hours or more, tested in the as-found condition, and recalibrated as necessary prior to reinstallation.

The steady state values of temperature and acoustical monitor readings shall be as measured after 5 days of steady state power operation. If the steady state temperature is greater than 275°F a., b. and c. above apply without regard to temperature increase.

This page is effective until the Reload 5/Cycle 6 refueling outage.