NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE: N C B E P 2 ②	LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 ③	LICENSE TYPE: 4 1 1 1 1 ④	CAT ⑤	
0 1 CON'T	REPORT SOURCE: L ⑥	DOCKET NUMBER: 0 5 0 - 0 3 2 4 ⑦	EVENT DATE: 0 9 2 4 8 2 ⑧	REPORT DATE: 1 0 2 2 8 2 ⑨	

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 While performing a review of plant surveillance procedures to assure technical
0 3 adequacy, it was determined that the required once per 18 months operability test of
0 4 the Unit No. 2 reactor instrumentation isolation excess flow check valves, 2-B21-F047
0 5 C & D and 2-B21-F049 C & D, was not addressed in plant procedures. This event did not
0 6 affect the health and safety of the public.
0 7 Technical Specifications 4.6.3.4, 6.9.1.9c
0 8

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	S A ⑪	D ⑫	Z ⑬	Z Z Z Z Z Z ⑭	Z ⑮	Z ⑯

⑰ LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 2	—	1 0 3	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS ㉒	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
G ⑱	X ⑲	Z ⑳	Z ㉑	0 0 0 0	Y ㉓	N ㉔	Z ㉕	Z 9 9 9 ㉖

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 This procedural deficiency resulted from a failure to recognize the technical specifi-
1 1 cation surveillance applicability of these valves during the technical review of the
1 2 plant modification which installed them. An 18-month frequency operability test
1 3 procedure for these valves, PT-02.1.25, was developed and implemented and the subject
1 4 surveillance was satisfactorily completed.

	FACILITY STATUS	% POWER	OTHER STATUS ㉚	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION ㉜
1 5	H ㉘	0 0 0 ㉙	NA	C ㉛	Procedure Technical Review

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY ㉟	LOCATION OF RELEASE ㊱
1 6	Z ㉝	Z ㉞	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION ㊴
1 7	0 0 0 ㊲	Z ㊳	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION ㊶
1 8	0 0 0 ㊵	

8211040164 821022
PDR ADOCK 05000324
S PDR

LOSS OF OR DAMAGE TO FACILITY ㊸

	TYPE	DESCRIPTION
1 9	Z ㊷	NA

PUBLICITY

	ISSUED	DESCRIPTION ㊺	NRC USE ONLY
2 0	N ㊹	NA	

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GPO 917-926

LER ATTACHMENT - RO #2-82-103

Facility: BSEP Unit No. 2 Event Date: 9-24-82

An On-site Nuclear Safety group technical review of plant surveillance procedures revealed the required 18-month frequency operability test of the Unit No. 2 reactor instrumentation isolation excess flow check valves was not reflected in plant procedures. This procedural deficiency resulted from a failure to recognize these valves as applicable to the required surveillance during the technical review of the plant modification which installed these valves in 1981. Because these valves are not remotely testable as were the air-operated plug type valves they replaced, and because they were a different type valve, it was not obvious to the personnel performing the technical review of the subject modification that these valves required the development of a specific surveillance procedure to permit surveillance compliance. Following this discovery PT-02.1.25 was developed, approved, and implemented to reflect the subject surveillance and it was satisfactorily completed. As a result of this discovery, all plant service groups' key supervisory personnel were apprised of this event.

The same corresponding reactor instrumentation isolation excess flow check valves installed on Unit No. 1, as part of a similar plant modification, will also be tested as required by PT-02.1.25.