NRC FORM 366 **U.S. NUCLEAR REGULATORY COMMISSION** (7.77) LICENSEE EVENT REPORT CONTROL BLOCK: (1)(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 0 0 0 0 0 0 - 0 0 3 LICENSE NUMBER 25 2 2 0 GAEIIH 0 LICENSEE CODE CON'T REPORT 5 0 0 0 3 6 6 7 1 0 0 9 8 0 1 (8) 1 0 2(6)SOURCE DOCKET NUMBER EVENT DATE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) Starting on October 9, 1982, with Hatch Unit 2 at 75% power, and Hatch 0 2 Unit 1 at 50% power, plant personnel (as a result of an NRC audit on 0 3 October 8, 1982) discovered that plant procedures did not adequately 0 4 test the automatic initiation logic of several plant systems. The Tech 0 5 Specs. testing requirements for the systems were not adequately met. 0 6 Health and safety of the public were not affected by this non-repetitive event. CAUSE SYSTEM CAUSE COMP VALVE SUBCODE COMPONENT CODE SUBCODE Z 13 Z (16) SICI ZI Z (14) Z (15) D (12) ZI ZI ZI ZI (11 18 OCCURRENCE REVISION SEQUENTIAL REPORT EVENT YEAR REPORT NO CODE LER/RO 81 Τ 0 21 11 11 0 1 NUMBER PRIME COMP. SUPPLIER NPRD-4 FORM SUB. COMPONENT ACTION FUTURE TAKEN ACTION SHUTDOWN METHOD ATTACHMENT SUBMITTED EFFECT ON PLANT (22) HOURS 0 0 0 G(18) Z (19 Z (25) Z 9 91 7 Z N (24) (26)CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Plant procedures did not adequately test their respective systems. New procedures were written and performed to test the logic excluded in the existing procedures. The systems involved are now in compliance with the requirements. Further investigation is underway and a subsequent report will be written. 4 RG METHOD OF FACILITY (30) DISCOVERY DESCRIPTION (32) % POWER OTHER STATUS 5 NRC Audit on SBGT System 7 D (31) 0 NA 80 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) OF RELEASE (33) Z (34) NA NA 45 44 80 PERSONNEL EXPOSURES. DESCRIPTION (39) 0 0 (37) Z (38) NA 80 PERSONNEL INJURIES DESCRIPTION (41) 0 | 0 0 (40) NA 8211040083 821021 PDR ADDCK 05000366 80 LOSS OF OR DAMAGE TO FACILITY (43) PDR DESCRIPTION NA Z (42) 80 PUBLICITY NRC USE ONLY DESCRIPTION (45 N (44) NA 80 15 H. L. Sumner - Supt. Plt. Eng. Serv. 912-367-7851 NAME OF PREPARER. PHONE .

LER #: 50-366/1982-112 Licensee: Georgia Power Company Facility Name: Edwin I. Hatch Docket #: 50-366

## Narrative Report for LER 50-366/1982-112

During an NRC audit exit interview held on October 8, 1982, the site was notified of potential problems concerning the completeness of logic system testing. Starting on October 9, 1982, with Hatch Unit 2 at 75% power, and Hatch Unit 1 at 50% power, plant personnel discovered that plant procedures did not adequately test the automatic initiation logic of the following: Standby Gas Treatment System, Unit 2 (Deviation Report Number 2-82-253, Discovery date: 10-9-82, test required per Tech. Specs. 4.6.6.1.d.2), Reactor Core Isolation Cooling System (Deviation Report Number 2-82-255, Discovery date: 10-14-82, testing required per Tech. Specs. 4.3.4.1 and Table 3.3.4.1), High Pressure Coolant Injection System (Deviation Report Number 2-82-257, Discovery date: 10-14-82, testing required per Tech. Specs. 4.5.1.c.1), Automatic Depressurization System (Deviation Report Number 2-82-258, Discovery date: 10-14-82, testing required per Tech. Specs. 4.5.2.a), and Standby Gas Treatment System, Unit 1 (Deviation Report Number 1-82-185, Discovery date: 10-14-82, testing required per Unit 1 Tech. Specs. 4.7.B.1.d and Unit 2 Tech. Specs. 3/4.6.6.1). Health and safety of the public were not affected by this non-repetitive event.

The event resulted from the failure of procedures to adequately test several plant systems. The inadequacies included failure to test a relay and/or the continuity of one or more sets of contacts in each of the logic systems involved. New procedures were written and performed to test the logic excluded in the existing procedures. Further investigation is underway and a subsequent update report will be submitted.