(7-77)

LICENSEE EVENT REPORT

CONTROL BLOCK: [| | | | |] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	I L D R S 3 (2)	0 0 - 0 0 0 0 0 - 0 0 (3)	4 1 1 1 1 (4)	(5)
7 8	9 LICENSEE CODE 14	15 LICENSE NUMBER 25	26 LICENSE TYPE 30	57 CAT 58

CON'T

0 1	REPORT SOURCE L (6)	0 5 0 0 0 2 4 9 (7)	1 2 0 1 8 1 (8)	1 2 0 7 8 1 (9)
7 8	60	61 DOCKET NUMBER 68	69 EVENT DATE 74	75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During unit startup Rx pressure was being maintained at about 50 psig in preparation for

0 3 HPCI system operability test. When Rx pressure decreased to 32 psig, rod N-9 was notched

0 4 from position 02 to 04. Unexpectedly, the rod went to position 06 and less than a 5

0 5 second period resulted (T.S. 6.6.B.1.d). The reactor scrammed due to IRM Hi Hi. There

0 6 was no effect on public health or safety because all Rx protective systems functioned

0 7 as designed. Previous occurrence: R.O. 50-237-76-74.

0 8

0 9	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
	R B (11)	E (12)	X (13)	C R D R V E (14)	Z (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
8 1	—	0 4 0	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	X (19)	A (20)	C (21)	0 0 4 8	X (23)	Y (24)	N (25)	G 0 [illegible] 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of the unexpected rod movement from position 02 to 06 is unknown. The single

1 1 notch control switch operation and proper latching of the rod (at various positions)

1 2 were subsequently verified with no difficulties identified. Work request has been

1 3 issued to check single notch control switch during the next refueling outage. On-site

1 4 review conducted to ensure no safety limits were exceeded prior to unit startup.

1 5	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
	C (28)	0 0 0 (29)	N/A	A (31)	Operator Observation

1 6	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
	Z (33)	Z (34)	N/A	N/A

PERSONNEL EXPOSURES

1 7	NUMBER	TYPE	DESCRIPTION (39)
	0 0 0 (37)	Z (38)	N/A

PERSONNEL INJURIES

1 8	NUMBER	DESCRIPTION (41)
	0 0 0 (40)	N/A

LOSS OF OR DAMAGE TO FACILITY (43)

1 9	TYPE	DESCRIPTION
	Z (42)	N/A

PUBLICITY

2 0	ISSUED	DESCRIPTION (45)	NRC USE ONLY
	N (44)	N/A	

NAME OF PREPARER B. A. Schroeder PHONE: 815-942-2920, X-[illegible]

8210270254 811215
PDR ADOCK 05000249
S PDR

ATTACHMENT TO LICENSEE EVENT REPORT 81-40/01T-0
COMMONWEALTH EDISON COMPANY (CWE)
DRESDEN UNIT 3 (ILDRS-3)
DOCKET # 050-249

During unit startup, Rx pressure was being maintained at about 50 psig in preparation for HPCI system operability test. When Rx pressure decreased to about 32 psig, rod N-9 was notched from position 02 to 04. Unexpectedly, the rod went from position 02 to 06. The rod notch override switch was not being used at the time and rod withdrawal was in accordance with approved station procedures and control rod withdrawal sequences. This caused a Rx scram (IRM Hi-Hi trips). A review of SRM and IRM recorders indicated that a less than a five second period (T.S.6.6.B.1.d) occurred prior to the reactor scram.

The cause of the unexpected rod movement from 02 to 06 is unknown. The single notch control switch operation and proper latching of the rod (at various positions) were subsequently verified with no difficulties identified. The control rod timing was checked and found to be slightly fast, but within Tech. Spec. limits. The timing was adjusted and rechecked. The CRD drive pressure was normal. A work request has been issued to check the timing and contact condition of the single notch control switch during the upcoming refueling outage.

The results of the on-site investigation and review by the Nuclear Fuel Services Department found that no safety limits were exceeded. There was no effect on public health or safety because all reactor protective systems functioned as designed. If the control switch investigation proves inconclusive, the CRD will be replaced prior to startup at BOC-8. No further action deemed necessary.

Commonwealth Edison

DEVIATION REPORT

DVR NO. - - - -
STA 12 UNIT 3 YEAR 81 NO 80

PART 1 TITLE OF DEVIATION: Rx Scram - IRM Hi Hi

OCCURRED: 12-1-81 (DATE) 1050 (TIME)

SYSTEM AFFECTED: 500 RPS

PLANT STATUS AT TIME OF EVENT: MODE Startup, PWR(MWT) .1, LOAD(MWE) 0, TESTING ☐ YES ☒ NO

DESCRIPTION OF EVENT:
While attempting to maintain Rx critical and Rx pressure less than 50 psig, due to HPCI being inop., inserted rod N-9 from position 04 to 02. Rx went subcritical and pressure eventually decreased to 32 psig. Attempted to withdraw rod N-9 with single notch control switch from position 02 to 04 but rod moved out to position 06 and Rx scrammed due to IRM Hi Hi. Preliminary investigation by nuclear engineers determined Rx period to be less than 5 seconds.

10 CFR50.72 NRC RED PHONE NOTIFICATION MADE 1100 hrs. ☒ YES ☐ NO

EQUIPMENT FAILURE ☐ YES ☒ NO WORK REQUEST NO.

RESPONSIBLE SUPERVISOR: James R. Doxsey DATE 12-1-8[illegible]

PART 2 OPERATING ENGINEER'S COMMENTS:
An immediate review of the SRM and IRM recorder traces indicated that a period of less than 5 seconds occurred prior to the Rx scram. NFS and the nuclear engineers are investigating. An onsite review will be performed prior to Rx startup.

☐ EVENT OF PUBLIC INTEREST
☐ TECH. SPEC. VIOLATION
☐ NON REPORTABLE OCCURRENCE
☒ 14 DAY REPORTABLE/T.S. 6.6.B.1.d
☐ 30 DAY REPORTABLE/T.S. ____
☐ ANNUAL/SPECL REPORT REQ'D

A.I.R. # ____
L.E.R. # 81-40/01T-0

☒ 24-HOUR NRC NOTIFICATION REQ'D
TELEPH Tom Tongue (REGION III) 12-1-81 (DATE) 1115 (TIME)
TELEGM/TELECOPY Keppler (REGION III) 12-1-81 (DATE) 1455 (TIME)

☐ CECo CORPORATE NOTIFICATION MADE IF ABOVE NOTIFICATION IS PER 10CFR21

☐ 5-DAY WRITTEN REPORT REQ'D PER 10CFR21

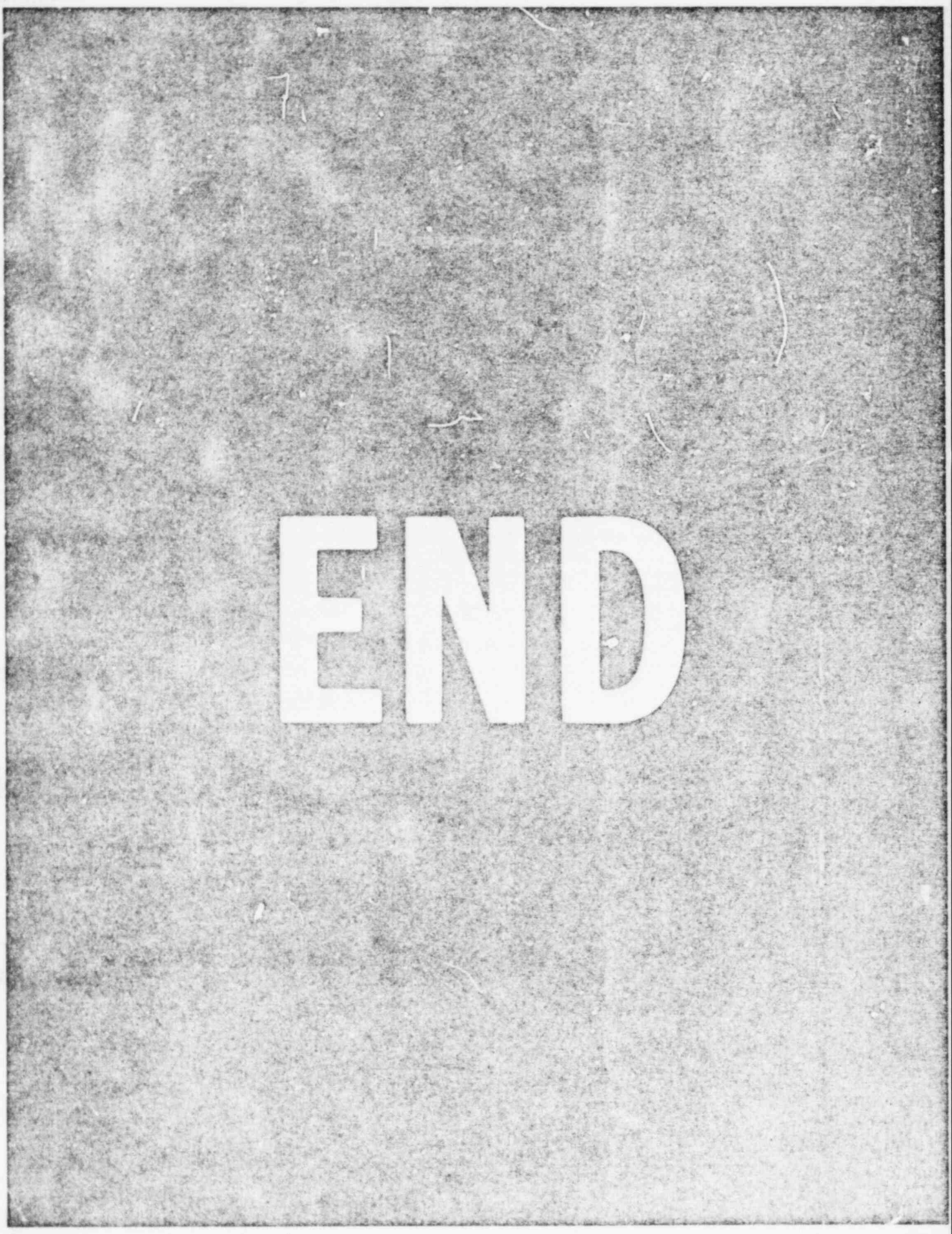
TELEPH E. Palmer (CECo CORPORATE OFFICER) 12-1-81 (DATE) 1210 (TIME)

PRELIMINARY REPORT COMPLETED AND REVIEWED: Michael Wright (OPERATING ENGINEER) 12-1-81 (DATE)

INVESTIGATED REPORT & RESOLUTION ACCEPTED BY STATION REVIEW: Jack Brunner 12/15/81; John M. Glenn 12/15/81

RESOLUTION APPROVED AND AUTHORIZED FOR DISTRIBUTION: for STATION SUPERINTENDENT 12-15-81 (DATE)

88-5176 (FORM 19-52-1) 10-81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