OPERATING DATA REPORT

DOCKET NO. 50-416
DATE 10/15/82
COMPLETED BY J. D. Richardson
TELEPHONE 601-969-2630

OPERATING STATUS

1. Unit Name: Grand Gulf Nuclear Station
2. Reporting Period: September 1982
3. Licensed Thermal Power (MWt): 191
4. Nameplate Rating (Gross MWe): 1372.5
5. Design Electrical Rating (Net MWe): 1250
6. Maximum Dependable Capacity (Gross MWe): N/A
7. Maximum Dependable Capacity (Net MWe): N/A

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: as restricted by O. L. NPF-13 as listed in 3.0 above.

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	0	0	0
12. Number Of Hours Reactor Was Critical	0	0	0
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	0	0	0
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	0	0
17. Gross Electrical Energy Generated (MWH)	0	0	0
18. Net Electrical Energy Generated (MWH)	0	0	0
19. Unit Service Factor	N/A	N/A	N/A
20. Unit Availability Factor	N/A	N/A	N/A
21. Unit Capacity Factor (Using MDC Net)	N/A	N/A	N/A
22. Unit Capacity Factor (Using DER Net)	N/A	N/A	N/A
23. Unit Forced Outage Rate	N/A	N/A	N/A

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
N/A

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	8/14/82	8/18/82
INITIAL ELECTRICITY	2/83	
COMMERCIAL OPERATION	8/83	

19. **UNIT SERVICE FACTOR**. Compute by dividing hours the generator was on line (item 14) by the gross hours in the reporting period (item 11). Express as percent to the nearest tenth of a percent. Do not include reserve shutdown hours in the calculation.

20. **UNIT AVAILABILITY FACTOR**. Compute by dividing the unit available hours (item 14 plus item 15) by the gross hours in the reporting period (item 11). Express as percent to the nearest tenth of a percent.

21. **UNIT CAPACITY FACTOR (USING MDC NET)**. Compute by dividing net electrical energy generated (item 18) by the product of maximum dependable capacity (item 7) times the gross hours in the reporting period (item 11). Express as percent to the nearest tenth of a percent.

22. **UNIT CAPACITY FACTOR (USING DER NET)**. Compute as in item 21, substituting design electrical rating (item 5) for maximum dependable capacity.

23. **UNIT FORCED OUTAGE RATE**. Compute by dividing the total forced outage hours (from the table in Unit Shutdowns and Power Reductions) by the sum of hours generator on line (item 14) plus total forced outage hours (from the table in Unit Shutdowns and Power Reductions). Express as percent to the nearest tenth of a percent.

24. **SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)**. Include type (refueling, maintenance, other), proposed date of start of shutdown, and proposed length of shutdown. It is recognized that shutdowns may be scheduled between reports and that this item may not be all inclusive. Be as accurate as possible as of the date the report is prepared. This item is to be prepared each month and updated if appropriate until the actual shutdown occurs.

25. Self-explanatory.

26. Self-explanatory. Note, however, that this information is requested for all units in startup and power ascension test status and is not required for units already in commercial operation.

    TEST STATUS is defined as that period following initial criticality during which the unit is tested at successively higher outputs, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the unit is generally considered by the utility to be available for commercial operation.

    Date of **COMMERCIAL OPERATION** is defined as the date that the unit was declared by the utility owner to be available for the regular production of electricity, usually related to the satisfactory completion of qualification tests as specified in the purchase contract and to the accounting policies and practices of the utility.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-416
UNIT 1
DATE 10/15/82
COMPLETED BY J. D. Richardson
TELEPHONE 601-969-2630

MONTH September

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	No Power Generated	17	
2		18	
3		19	
4		20	
5		21	
6		22	
7		23	
8		24	
9		25	
10		26	
11		27	
12		28	
13		29	
14		30	
15		31	
16			

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-416
UNIT NAME Grand Gulf Nuclear Station
DATE 10/15/82
COMPLETED BY J. D. Richardson
TELEPHONE 601-969-2630

REPORT MONTH September

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely[1]. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT #. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161) using the following criteria:

A. If a component failed, use the component directly involved.

B. If not a component failure, use the related component: e.g., wrong valve operated through error; list valve as component.

C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

[1]Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

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OPERATING DATA REPORT
SUPPLEMENTAL INFORMATION

A temporary alteration was made on September 27, 1982, to the Liquid Radwaste System to allow discharge in excess of 35 gpm, after having been reviewed and approved by the Plant Safety Review Committee (PSRC). The following information is submitted pursuant to Technical Specification 6.15.1:

6.15.1.

a. An evaluation was conducted and a determination was made in conformance with 10 CFR 50.59 that the temporary jumpering and resetting of interlocks to allow discharging non-radioactive water through the liquid radwaste line in excess of 35 gpm will not increase the probability of an occurrence or the consequences of an accident previously evaluated in the FSAR as discharge flow. This alteration is encompassed by the FSAR Accident Analysis. The discharge is in conformance with the National Pollution Discharge Elimination System requirements.

b. The reason for the temporary change is to allow the discharge of quantities of non-radioactive water, in excess of the flow rates presently allowed by the interlocks. Non-radioactive water, slightly contaminated with glycol, held in the radwaste inventory, should be reduced in volume by discharging up to 3 million gallons having a maximum concentration of 20 ppm biodegradable organic matter. Dilution would occur by discharging to the river at the rate of approximately 90 gpm.

c. The temporary change removes, by means of jumper cables, the low-flow interlock for cooling tower dilution flow of 11,300 gpm and raises the maximum radwaste discharge flow from 35 to 90 gpm. The radwaste discharge flow and dilution flow was calculated using the Offsite Dose Calculation Manual (ODCM) as required by Technical Specifications (Tech. Spec. 3.3.7.11, LCO).

d. It is anticipated that no water with significantly above background detectable amounts of radiation, as determined by sampling, will be discharged. The predicted releases are negligible and within the limits addressed in the FSAR.

e. Since non-radioactive water will be discharged, there are no significant changes in expected maximum exposures to individuals in unrestricted areas, nor significant differences from exposures previously calculated in the FSAR.

f. Since no prior release data of significant quantities is available, this non-radioactive discharge cannot be meaningfully compared.

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g. The change in exposure to plant operating personnel will be negligible since it is anticipated that no water with significantly above background radiation will be discharged.

h. The change was reviewed and found acceptable by the PSRC as shown by approval signature on the 10 CFR 50.59 evaluation form retained in PSRC files, and as addressed in the notes of the PSRC meeting considering this temporary alteration.