FORM 366

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

1 | O | H | D | B | S | 1 | (2) | Ø | Ø | - | Ø | Ø | N | P | F | - | Ø | 3 | (3) | 4 | 1 | 1 | 1 | 1 | (4) | | | (5)

8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

NT

1 | REPORT SOURCE | L | (6) | Ø | 5 | Ø | - | Ø | 3 | 4 | 6 | (7) | 1 | Ø | Ø | 6 | 7 | 8 | (8) | 1 | 1 | Ø | 1 | 7 | 8 | (9)

8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

2 | At 2315 hours on 10/6/78, Channel 1 of the Steam and Feedwater Rupture Control System

3 | (SFRCS) tripped, rendering the channel inoperable. The unit was placed in Action

4 | Statement 13 of Technical Specification 3.3.2.2. There was no danger to the health

5 | and safety of the public or unit personnel. The other SFRCS channels were operable

6 | during the period that SFRCS Channel 1 was inoperable. SFRCS Channel 1 failed in

7 | the safe position (in the tripped position) by the failure. (NP-33-78-121)

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9 | SYSTEM CODE: C D (11) | CAUSE CODE: E (12) | CAUSE SUBCODE: E (13) | COMPONENT CODE: I N S T R U (14) | COMP. SUBCODE: P (15) | VALVE SUBCODE: Z (16)

8 9 10 11 12 13 18 19 20

(17) LER RO REPORT NUMBER: EVENT YEAR 7 8 | — | SEQUENTIAL REPORT NO. 1 Ø 2 | / | OCCURRENCE CODE Ø 3 | REPORT TYPE L | — | REVISION NO. Ø

21 22 23 24 26 27 28 29 30 31 32

ACTION TAKEN: A (18) | FUTURE ACTION: Z (19) | EFFECT ON PLANT: Z (20) | SHUTDOWN METHOD: Z (21) | HOURS (22): Ø Ø Ø Ø | ATTACHMENT SUBMITTED: Y (23) | NPRD-4 FORM SUB.: Y (24) | PRIME COMP. SUPPLIER: A (25) | COMPONENT MANUFACTURER: L Ø 4 5 (26)

33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

0 | The power supply for the channel was found to be defective. The unit was removed from

1 | Action Statement 13 of Technical Specification 3.3.2.2 at 2301 hours on 10/9/78 when

2 | the unit entered Mode 4. The defective power supply was replaced with another bench-

3 | tested unit from stock on 10/10/78.

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5 | FACILITY STATUS: E (28) | % POWER: Ø 9 2 (29) | OTHER STATUS (30): NA | METHOD OF DISCOVERY: A (31) | NA | DISCOVERY DESCRIPTION (32)

8 9 10 12 13 44 45 46 80

6 | ACTIVITY RELEASED: Z (33) | CONTENT OF RELEASE: Z (34) | AMOUNT OF ACTIVITY (35): NA | LOCATION OF RELEASE (36): NA

8 9 10 11 44 45 80

PERSONNEL EXPOSURES

7 | NUMBER: Ø Ø Ø (37) | TYPE: Z (38) | DESCRIPTION (39): NA

8 9 11 12 13 80

PERSONNEL INJURIES

8 | NUMBER: Ø Ø Ø (40) | DESCRIPTION (41): NA

8 9 11 12 80

LOSS OF OR DAMAGE TO FACILITY (43)

9 | TYPE: Z (42) | DESCRIPTION: NA

8 9 10 80

PUBLICITY

0 | ISSUED: N (44) | DESCRIPTION (45): NA

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NRC USE ONLY

TOLEDO EDISON COMPANY
DAVIS-BESSE UNIT ONE NUCLEAR POWER STATION
SUPPLEMENTAL INFORMATION FOR LER NP-33-78-121

DATE OF EVENT: October 6, 1978

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Steam and Feedwater Rupture Control System (SFRCS) Channel 1 inoperable due to inoperable power supply.

Conditions Prior to Occurrence: The unit was in Mode 1, with Power (MWT) = 2550, and Load (MWE) = 870.

Description of Occurrence: At 2315 hours on October 6, 1978, Channel 1 of the SFRCS tripped, rendering the channel inoperable. The unit was placed in Action Statement 13 of Technical Specification 3.3.2.2, which requires the operability of each of two SFRCS channels in Modes 1, 2, and 3. Action Statement 13 states that the inoperable channel must be placed in the tripped condition within one hour.

Designation of Apparent Cause of Occurrence: The cause of the occurrence is attributed to the failure of PS-1, the 48 VDC Lambda power supply (Part No. LCS-D-48) for the channel.

Analysis of Occurrence: There was no danger to the health and safety of the public or to unit personnel. The other SFRCS channel was operable during the period that SFRCS Channel 1 was inoperable. SFRCS Channel 1 failed in the safe position (in the tripped position) by the failure.

Corrective Action: The unit was removed from Action Statement 13 of Technical Specification 3.3.2.2 at 2301 hours on October 9, when the unit entered Mode 4.

Under Maintenance Work Order I&C559-78 on October 10, 1978, an Instrument and Control technician replaced the defective power supply with another bench-tested unit from stock. SFRCS Channel 1 was returned to operable status.

Failure Data: There have been no previously reported failures of 48 VDC power supplies on SFRCS channels. Failure of a 24 VDC power supply for Steam Generator Level 1, Channel 1, was reported in Licensee Event Report NP-33-77-24.