

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | 0 | H | D | B | S | 1 | 2 | 0 | 0 | - | 0 | 0 | N | P | F | - | 0 | 3 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

CON'T REPORT SOURCE L | 6 | 0 | 5 | 0 | - | 0 | 3 | 4 | 6 | 7 | 0 | 5 | 1 | 1 | 7 | 8 | 8 | 0 | 6 | 0 | 5 | 7 | 8 | 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

0 2 | At 1700 hours on May 11, 1978, Containment Vacuum Relief Valve CV 5076 was found to
0 3 | have excessive leakage during the performance of the Local Leak Rate Test. There
0 4 | was no danger to the health and safety of the public or to unit personnel. The unit
0 5 | was in a shutdown condition. Overall allowable leakage for containment was not ex-
0 6 | ceeded. No core alteration or movement of irradiated fuel within containment took
0 7 | place while the valve was inoperable. (NP-33-78-61)

0 9 | SYSTEM CODE S A | CAUSE CODE E | CAUSE SUBCODE B | COMPONENT CODE V A L V E X | COMP. SUBCODE B | VALVE SUBCODE C | LER/RO REPORT NUMBER 7 8 | SEQUENTIAL REPORT NO. 0 5 1 | OCCURRENCE CODE 0 3 | REPORT TYPE L | REVISION NO. 1 | ACTION TAKEN B | FUTURE ACTION Z | EFFECT ON PLANT Z | SHUTDOWN METHOD Z | HOURS 0 0 0 0 | ATTACHMENT SUBMITTED Y | NRPD-4 FORM SUB. Y | PRIME CO. P. SUPPLIER A | COMPONENT MANUFACTURER F 1 3 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

1 0 | The valve wafer was not properly seating against the valve liner. The valve was
1 1 | removed from service and the line blanked off to maintain containment integrity by
1 2 | 1830 hours on May 12, 1978. The valve was repaired by the vendor, Fisher Valve, and
1 3 | returned to service on June 16, 1978, after successful completion of a stroke test
1 4 | and leak rate test.

1 5 | FACILITY STATUS G | % POWER 0 0 | OTHER STATUS NA | METHOD OF DISCOVERY B | DISCOVERY DESCRIPTION Surveillance Test ST 5061.02

1 6 | ACTIVITY RELEASED 2 | CONTENT 2 | AMOUNT OF ACTIVITY NA | LOCATION OF RELEASE NA

1 7 | PERSONNEL EXPOSURES NUMBER 0 0 0 | TYPE 2 | DESCRIPTION NA

1 8 | PERSONNEL INJURIES NUMBER 0 0 0 | DESCRIPTION NA

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE Z | DESCRIPTION NA

2 0 | ISSUED DESCRIPTION N | PUBLICITY NA | PUBLICATION DESCRIPTION NA | NRC USE ONLY | 7810310330 | PHONE: 419-259-5000, Ext. 250

TOLEDO EDISON COMPANY
DAVIS-BESSE UNIT ONE NUCLEAR POWER STATION
SUPPLEMENTAL INFORMATION FOR LER NP-33-78-61

DATE OF EVENT: May 11, 1978

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Containment Vacuum Relief Valve CV 5076 was declared inoperable.

Conditions Prior to Occurrence: The unit was in Mode 6, with Power (MWT) = 0, and Load (MWE) = 0.

Description of Occurrence: At 1700 hours on May 11, 1978, Containment Vacuum Relief Valve CV 5076 was found to have excessive leakage during the performance of Surveillance Test ST 5061.02, "Local Leak Rate Test" (LLRT). The valve was declared inoperable.

The unit was in Mode 6 and, therefore, was not placed into the Action Statement of Technical Specification 3.6.1.1, which requires primary containment integrity to be maintained in Modes 1, 2, 3 and 4.

This occurrence is being reported as a component failure.

Designation of Apparent Cause of Occurrence: The valve wafer was not properly seating against the valve liner possibly due to improper initial installation or foreign material on the seating surface.

Analysis of Occurrence: There was no danger to the health and safety of the public or to unit personnel. The unit was in a shutdown condition. Overall allowable leakage for containment was not exceeded. No core alteration or movement of irradiated fuel within containment took place while the valve was inoperable.

Corrective Action: The valve was removed from service and the line blanked off to maintain containment integrity by 1830 hours on May 12, 1978. The valve was returned to the valve vendor, Fisher Valve, for installation of a new liner. The valve was reinstalled per Maintenance Work Order 78-1193. After successful completion of a stroke test and a leak rate test, the valve was returned to service on June 16, 1978.

Failure Data: One other Containment Relief Valve, CV 5070, leakage was also excessive when tested on May 9, 1978. See Licensee Event Report NP-33-78-60.

LER #78-051