



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS  
WASHINGTON, D. C. 20555

April 30, 1991

MEMORANDUM FOR: James M. Taylor  
Executive Director for Operations

ATTN: J. Blaha

FROM: *R. F. Fraley*  
R. F. Fraley, Executive Director, ACRS

SUBJECT: 372ND ACRS MEETING FOLLOW-UP ITEMS

Based on discussions regarding methods for improved implementation and follow-up of ACRS recommendations, the Committee agreed that a summary of Actions, Agreements, Assignments, Requests, and Commitments made during each full Committee meeting will be sent to your office following each meeting. Attached per this agreement is such a summary related to the 372nd ACRS Meeting, April 11-13, 1991.

Attachment:  
As stated

cc w/attachment:  
E. L. Jordan, AEOD  
R. M. Bernero, NMSS  
T. E. Murley, NRR  
E. S. Beckjord, RES  
S. Chilk, SECY  
J. Mitchell, OCM/KC  
G. Marcus, OCM/KR  
D. Trimble, OCM/JC  
J. Guttmann, OCM/FR

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SUMMARY OF  
ACTIONS, AGREEMENTS, ASSIGNMENTS, REQUESTS  
AND FOLLOW-UP MATTERS  
372ND ACRS MEETING  
APRIL 11-13, 1991

During its 372nd meeting, April 11-13, 1991, the Advisory Committee on Reactor Safeguards discussed several matters and completed the reports and letter noted below. In addition, the Committee authorized me to transmit the memorandum noted below. Copies of the reports to the Commission and the memorandum are attached.

REPORTS TO THE COMMISSION

- Staff Evaluation and Recommendations on Maintenance Rulemaking (Report to Chairman Carr, dated April 17, 1991.)
- Draft Final Rule on Nuclear Power Plant License Renewal (Report to Chairman Carr, dated April 17, 1991.)
- Proposed Policy Issues Identified in SECY-91-078, "Chapter 11 of the Electric Power Research Institute's (EPRI's) Requirements Document and Additional Evolutionary Light-Water Reactor (LWR) Certification Issues" (Report to Chairman Carr, dated April 23, 1991.)

LETTER TO THE EDO

- Proposed Resolution of Generic Safety Issue 130, "Essential Service Water System Failures at Multi-Unit Sites" (Letter to J. M. Taylor, EDO, dated April 18, 1991.)

MEMORANDUM

- Proposed Amendments to 10 CFR Part 50 to Update and Clarify Emergency Preparedness Regulations for Nuclear Power Plants (Memorandum for E. S. Beckjord, RES, from R. F. Fraley, ACRS, dated April 17, 1991.)

Mr. Beckjord was informed that the Committee deferred its review of the subject amendments until after the public comments have been received and resolved by the NRC staff and concluded that it had no objection to the staff's proposal to issue these amendments for public comment.

LIST OF FOLLOW-UP MATTERS

- The Committee agreed to schedule the 373rd ACRS meeting between May 8 and 11, 1991. The meeting will convene at 2:00 p.m. on Wednesday, May 8, 1991. (Mr. Fraley has the follow-up action on this matter.)

- In its April 17, 1991 report to Chairman Carr related to the Draft Final Rule on Nuclear Power Plant License Renewal, the Committee requested that the staff provide the ACFS with an opportunity to review the regulatory guide and standard review plan associated with license renewal prior to their issuance. (Mr. Igne has the follow-up action on this matter.)
- In its April 18, 1991 letter to the EDO related to the resolution of Generic Issue 130, the Committee stated that it would consider the advisability of requiring a separate and independent cooling system for reactor coolant pump seals during its review of the proposed resolution of Generic Issue 23, "Reactor Coolant Pump Seal Failures." (Mr. Boehnert has the follow-up action on this matter.)
- The Committee suggested that the Improved Light Water Reactors Subcommittee consider a potential additional ELWR certification issue, raised by Mr. Michelson, concerning the sequencing of safety loads onto the safety buses following a LOCA and loss of offsite power, and recommend a course of action to the full Committee for consideration. (Dr. El-Zeftawy has the follow-up action on this matter.)
- The Committee agreed with the proposal by Mr. Carroll that the AEOD staff brief the Committee on the results and findings of the AEOD study related to Performance Indicators of Safety System Failures after completion of this study. (Mr. Rotella has the follow-up action on this matter.)
- The Committee agreed with the proposal by Dr. Catton, Chairman of the Thermal Hydraulic Phenomena Subcommittee, that the RES staff be invited to brief the Committee regarding documentation of NRC-sponsored computer codes during the May 8-11, 1991 ACRS meeting. (Mr. Boehnert has the follow-up action on this matter.)
- The Committee agreed to discuss further, during the May 1991 ACRS meeting, the preparation of a comprehensive ACRS report to the Commission on the NRC Safety Research Program and budget. Dr. Catton, Chairman of the Safety Research Program Subcommittee, agreed to provide background information for use by the Committee in discussing this matter. (Mr. Boehnert has the follow-up action on this matter.)
- The Committee agreed with the proposal by Mr. Carroll, Chairman of the Plant Operations Subcommittee, that the Committee hear a briefing by the staff regarding the proposed final version of the rule on Emergency Response Data System during the May 1991 ACRS meeting. (Mr. Boehnert has the follow-up action on this matter.)

- The Committee decided to defer its consideration of research needs for Westinghouse AP-600 and General Electric SBWR passive plants until after the SECY paper on this matter has been made available to the Committee. (Dr. El-Zeftawy and Mr. Rotella have the follow-up action on this matter.)
- The Committee agreed that, during one of the future meetings, Westinghouse and General Electric should be invited to brief the Committee on the design details of Westinghouse AP-600 and GE SBWR passive plants and associated research needs. (Dr. El-Zeftawy and Mr. Rotella have the follow-up action on this matter.)
- The Committee decided that the Systematic Assessment of Experience Subcommittee should discuss the validity of the concerns expressed by the Nuclear Utility Group on Equipment Qualification in its March 6, 1991 letter to Mr. Ward on the AEOD findings and conclusions related to solenoid-operated valve problems and recommend a course of action to the full Committee for consideration. (Mr. Rotella has the follow-up action on this matter.)

#### FUTURE ACTIVITIES

The Committee agreed to consider the following items during the 373rd, April 11-13, 1991 ACRS meeting:

- Containment Design Criteria for Future Nuclear Power Plants - Continue discussion of the proposed ACRS report to the NRC regarding containment design criteria for future light-water reactor plants to deal with severe accidents.
- Reactor Operating Experience - Briefing and discussion of recent events and incidents that have occurred at nuclear power plants, including the steam generator tube failure at the Mihama Power Station in Japan, and problems potentially affecting steam generator tube integrity in U.S. nuclear power plants. Representatives of the NRC staff and the licensees will participate, as appropriate.
- Meeting with the Director of the Office of Nuclear Reactor Regulation - Discuss matters of mutual interest with the Director of the Office of Nuclear Reactor Regulation.
- Fitness for Duty - Discuss and comment on the NRC staff's proposed final version of 10 CFR Part 55, "Fitness for Duty Requirements for Licensed Operators." Representatives of the NRC staff will participate, as appropriate.

- 10 CFR Part 50, Appendix J - Discuss and comment on the NRC staff's proposed final revisions to Appendix J to 10 CFR Part 50, "Leakage Rate Testing of Containments of Light-Water-Cooled Nuclear Power Plants," and a related Regulatory Guide, "Containment System Leakage Testing." Representatives of the NRC staff and industry will participate, as appropriate.
- Emergency Response Data System - Discuss and comment on the NRC staff proposed Final Rule on the Emergency Response Data System. Representatives of the NRC staff will participate, as appropriate.
- USSR Reactor Pressure Vessel Annealing Experience - Briefing by and discussion with the NRC staff regarding the USSR reactor pressure vessel annealing experience.
- NRC Code Documentation Requirements - Briefing and comment on the NRC requirements pertaining to documentation of NRC-sponsored computer codes.
- NRC Safety Research Program - Discuss the format and content of the ACRS report to the Commission on the NRC Safety Research Program and Budget.
- Mortality Among Radiation Workers at Oak Ridge National Laboratory - Briefing by and discussion with the NRC staff regarding the recent studies of the mortality among workers at the Oak Ridge National Laboratory.
- NRC Use of Probabilistic Risk Assessment - Discuss a proposed ACRS report to the NRC regarding the use of probabilistic risk assessment.
- Appointment of New Members - Discuss qualifications of candidates proposed for appointment to the ACRS.
- Future Activities - Discuss anticipated Subcommittee activities and items proposed for consideration by the full Committee. Discuss administrative matters related to the conduct of Committee business, as appropriate.
- ACRS Subcommittee Activities - Reports on and discussion of the status of assigned Subcommittee activities.
- Miscellaneous - Continue discussion of issues that were not completed at previous ACRS meetings as time and availability of information permit.



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WASHINGTON, D. C. 20555

May 17, 1991

NOTE TO: Donald H. Lanham, Acting Chief  
Docketing and Document Control Desk Section  
Document Control Branch

FROM: Ethel M. Barnard  
Advisory Committee on Reactor Safeguards

SUBJECT: PLACEMENT OF "FULL TEXT" ACRS DOCUMENTS ON NUDOCS

In accordance with direction from Jim Blanton, I have attached the following documents for "full text" processing through the NUDOCS system:

ACRS-2735 - Thermal Hydraulic Phenomena SC Meeting, 12/12/90 re  
NRC Program on Interfacing Systems LOCA (ISLOCA)

Also enclosed is a 3 1/2" diskette containing the "electronic text" of the documents. The documents are in "WORDPERFECT" format. Your signature in the space below, will serve as confirmation of receipt of the "Hard Copy" and "Electronic Copy". Please return a copy of this note to me at Mail Stop P-315. If you have any questions concerning this matter, please feel free to contact me on x27691.

Ethel M. Barnard, ACRS

Received by Donald H. Lanham:

Signature: \_\_\_\_\_ Date: \_\_\_\_\_





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS  
WASHINGTON, D. C. 20555

April 17, 1991

The Honorable Kenneth M. Carr  
Chairman  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Chairman Carr:

SUBJECT: DRAFT FINAL RULE ON NUCLEAR POWER PLANT LICENSE RENEWAL

During the 372nd meeting of the Advisory Committee on Reactor Safeguards, April 11-13, 1991, we reviewed the draft of the final rule on nuclear power plant license renewal (10 CFR Part 54). Our Subcommittee on Plant License Renewal discussed this matter during its April 8, 1991 meeting. During our consideration of this matter, we had the benefit of discussions with representatives of the NRC staff, NUMARC, and Northern States Power Company. The latter is the licensee for the Monticello Nuclear Generating Plant, which is a lead plant in the license renewal program. We also had the benefit of the document referenced.

The ACRS reported to you on the proposed license renewal rule in its report of April 11, 1990. Since that time, the proposed rule was published for public comment. The staff received 197 comments. It has assimilated information from these comments and information received in a number of interactions with industry and has prepared a draft final rule. The schedule calls for the final rule to be published by June 28, 1991, and for other parts of the rulemaking package, a regulatory guide and a standard review plan, to be published about one year later.

As stated in our April 11, 1990 report, we concur with the approach being taken by the staff in this rulemaking. However, there are two areas of disagreement between the staff and NUMARC that we would like to bring to your attention. The first might require a modification in the draft final rule. The second is related to implementation of the rule.

The first matter is an issue on which we do not have a recommendation except that it should receive your consideration. The draft final rule requires that each applicant for license renewal develop a "compilation" of its Current Licensing Basis. Although it is not precisely clear what this means, it was agreed that it would, at a minimum, include a list of all licensing commitments agreed to by the applicant over the history of its plant. Industry representatives believe this is unnecessary.

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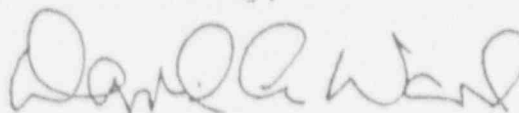
April 17, 1991

The second issue is how implementation of the rule will be limited in scope to concentrate resources for aging management where needed. The rule would require that each applicant develop a list of Systems, Structures, and Components Important to License Renewal (SSCITLR) and then implement an aging management program appropriate for items on that list. The staff's position is that the original SSCITLR list should include all those items in the plant that play a role in meeting any docketed commitment the licensee has made. This would include the original license; commitments related to new rules as they came into being; and commitments made in response to Safety Evaluation Reports, Information Notices, Bulletins, Generic Letters, and Orders.

The industry representatives told us that such a definition of SSCITLR would result in a list that includes 85 to 90 percent of all equipment in the plant. They believe that application of a special aging program to all of these items would be unnecessary and onerous. The process of reducing the initial SSCITLR list to just those items to be covered by a special aging program is critical. Items important to implement other commitments would not thereby be ignored. They would be maintained through the new license period just as they are now.

We believe that selection of those items to be subjected to a special aging program should be based on technical rather than legal argument. Our understanding is that a program of this nature can be developed with the rule as presently drafted. However, implementation will require careful crafting of the regulatory guide and the standard review plan. We would like the opportunity to review these documents before they are issued.

Sincerely,



David A. Ward  
Chairman

Reference:

Memorandum dated March 6, 1991 from Warren Minners, Office of Nuclear Regulatory Research, to Raymond F. Fraley, ACRS, Subject: Final Rule on Nuclear Power Plant License Renewal, with enclosures (Predecisional)