U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No. 50-263/77-06

Docket No. 50-263 License No. DPR-22

Licensee: Northern States Power Company
414 Nicollet Mall
Minneapolis, MN 55401

Facility Name: Monticello Nuclear Generating Plant

Inspection at: Monticello Site, Monticello, MN

Inspection Conducted: May 9-13, 1977

Inspector: N. C. Choules — May 26, 1977 (date signed)

Approved By: R. F. Warnick, Chief, Reactor Projects Section 2 — 5/26/77 (date signed)

Inspection Summary

Inspection on May 9-13, 1977, (Report No. 50-263/77-06)
Areas Inspected: Routine, unannounced inspection of surveillance testing, procedures, annual report, nonroutine event reports, and outstanding items. The inspection involved 32 inspector-hours on site by one NRC inspector.
Results: Of the five areas inspected, no items of noncompliance or deviations were identified in four areas; one apparent item of noncompliance (infraction - failure to approve temporary changes to two procedures, Paragraph 2) was identified in one area.

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DETAILS

1. Person Contacted

*L. R. Eliason, Plant Manager
*M. H. Clarity, Superintendent, Plant Engineering and Radiation Protection
*W. E. Anderson, Superintendent, Operations and Maintenance
W. A. Sparrow, Operations Supervisor
*D. D. Antony, Plant Engineer, Operations
W. A. Shamla, Plant Engineer, Technical

The inspector also talked with and interviewed several other licensee employees, including members of the Operations, Engineering, and Instrument and Control section.

*denotes those present at exit interview.

2. Procedures

The following procedures were reviewed by the inspector:

a. Operating Procedures

B.3.3	Automatic Pressure Relief
B.3.5	Standby Liquid Control
B.4.1	Primary Containment (Inerting and Deinerting Sections)
B.5.2	Rod Worth Minimizer
B.5.7	Reactor Level Control
B.6.6	Condensate Demineralizer
B.8.2	Plant Makeup
B.8.7	Heating and Ventilation
B.9.9	250 Volt DC System
B.9.10	125 Volt AC System
C.2	Power Operations
C.3	Shutdown Procedures

b. Emergency Procedures

C.4.III.G	Reactor Building Closed Cooling Water System
C.4.III.H	Feedwater System Failure
C.4.IV	Acts of Nature
C.4.V	Procedures for Plant Shutdown from Outside the Control Room

c. Preventive Maintenance Procedures

OCD* and PM**	4280	Auto Pressure Relief Maintenance
OCD and PM	4840	SBLC System Electrical Maintenance
OCD and PM	4400	Condensate Demineralizer Element Replacement
OCD and PM	4851	Safeguards Bus Source Breaker Maintenance
OCD and PM	4104	Emergency Diesel Generator Inspection
OCD and PM	4842	Emergency Service Water System Electrical Maintenance
OCD and PM	4855	Essential MCC Tie Breaker Electrical Maintenance
PM 4120		RCIC System Inspection

*Operations Control Document
**Preventive Maintenance

d. Administrative Procedures

4ACD 3.1	Monticello Plant Organization
4ACD 4.7	Plant Operato. and Control Room Activities

The procedures listed above were reviewed to verify that:

a. Procedures and changes to procedures are reviewed and approved in accordance with Technical Specifications.

b. Procedure changes were made to reflect Technical Specifications revisions.

c. Changes made to these procedures were in conformance with 10 CFR 50.59 requirements.

d. The overall content of procedures listed in a, b, and c, above, are in conformance with the Technical Specifications.

e. The technical content of selected procedures listed in a, b, c, and d, above, are adequate to control safety-related operations within applicable regulatory requirements.

In the review of OCD 4840 completed on October 10, 1975, and OCD 4855 completed on March 29, and April 1, 1974, it was noted that changes were made to these procedures without the apparent concurrence of two individuals holding senior operator licenses as required by Technical Specification 6.5.D. For OCD 4840, a change was made to correct the SBLC pump motor breaker identification number on page 4 of the OCD. For OCD 4855, changes were made on the Isolation Table 1. There was no documentation that the changes had been concurred in by the two senior licensed operators.

From review of these changes, it appeared that they should have been made permanent changes to the procedures. Permanent changes had not been made at the time of the inspection even though each procedure had received a biannual review by the Operations Committee since the changes were made.

While the inspector was still at the site the licensee initiated action to have OCDs 4840 and 4855 changed. At the exit interview, the licensee stated that in addition to changing the procedures they would instruct their personnel on the requirements for making changes to safety-related maintenance procedures. The inspector stated that this action should be adequate and no reply to the noncompliance item would be required.

As a result of the inspector's review and discussion with the licensee's representatives, the licensee will make the revision to procedures as follows. This was discussed in the exit interview.

a. B.9.9 and B.9.10 - Add an abnormal procedure for switching from the normally lined up battery charger to the backup battery charger in case of failure to the normal battery charger.

b. C.4.V - Add instructions for the use of an alternate relief valve in the depressurization of the reactor.

c. PM 4280 - Add a requirement to record "as found" safety valve/ relief valve settings.

3. Surveillance

a. The inspector selected a sampling of Technical Specifications testing requirements and verified that the licensee has surveillance test procedures which accomplished the required surveillance testing. The review of the following surveillance test procedures showed that prerequisites and preparation for test are specified, acceptance critiera is specified, and operational checks prior to returning equipment to service are specified when required.

b. The following surveillance tests performed during the past year were reviewed.

Procedure Number	Title or Requirement
0013 and 0043	IRM Rod Block Calibration and Scram Test
0042	IRM Functional Test
0052 and 0155	HPCI High Steam Flow Sensor Test and Calibration
0058	HPCI Steam Line Area Temperature Test

0085	Standby Liquid Control Pump Flow Rate Check
0112	Safety Valve/Relief Valve Testing
0126	Reactor Coolant and Drywell Leak Check
0144	Primary Containment O_2 Concentration
0185 and 0186	Substation Weekly and Quarterly Battery Checks
0199	24 Volt Battery Monthly Check
0200	24 Volt Battery Quarterly Check
0212	Rod Worth Minimizer Operability Verification

The inspector reviewed selected completed surveillance tests from the above and determined from the sample reviewed that tests are being performed at their required frequency. The inspector observed the performance of Surveillance Test 0085.

No items of noncompliance or deviations were identified in the review of the above surveillance testing.

4. 1976 Annual Operating Report

The subject report was reviewed and indicated that the information required by the Technical Specification had been reported. Review of the shift supervisor's logbook indicated that the forced outages during 1976 were as reported.

No items of noncompliance or deviations were identified.

5. Reportable Occurrences

The following reportable occurrences were reviewed by examination of logs, records, internal reports, and through discussions with plant personnel. Occurrences were reviewed for completion of reporting requirements, investigation and determination of cause, proposed corrective measures, and completion of corrective actions.

a. RO 77-02,[1/] Failure to Review Standby Gas Treatment Surveillance Test

This occurrence was reviewed in a previous inspection.[2/] For this inspection, the inspector reviewed the licensee's corrective action in regard to assuring that Technical Specifications changes are implemented into affected procedures. The licensee has implemented procedure No. 3071, Technical Specification Requirement Control Procedures, to assure the above. This procedure should assure Technical Specification changes are implemented into procedures and will provide a good record of action taken for each change.

1/ RO 50-263/77-02, NSP to RIII, dtd 2/247/77.
2/ IE Inspection Rpt No. 50-263/77-05.

b. RO 77-05,[3] Torus Water Volume Less than Technical Specification Limit

This occurrence was the result of a failure in the differential pressure transducer bellows giving an erroneous level indication. This was discovered when a redundant level indicator was installed showing a difference between the two level indicators. Routine comparison of the readings of the two installed level indicators should prevent recurrence.

c. RO 77-07,[4] HPCI Isolation Temperature Switch Setpoint Drift

As a result of this and other switch setpoint drifts, the licensee is looking at other type switches which have a smaller temperature range and less potential for setpoint drift.

No items of noncompliance or deviations were identified.

6. Outstanding Item

In a previous inspection,[5] the licensee stated they would prepare an LPRM calibration procedure. The licensee has revised Operations Manual Section C.2 to include an LPRM calibration procedure.

No items of noncompliance or deviations were identified.

7. Exit Interview

The inspector met with licensee representatives (denoted in Paragraph 1) at the conclusion of the inspection on May 13, 1977. The inspector summarized the scope and findings of the inspection. The licensee acknowledged statements made by the inspector with respect to the item of noncompliance, and presented his corrective action (Paragraph 2).

3/ RO 50-263/77-05, NSP to RIII, dtd 3/15/77.
4/ RO 50-263/77-07, NSP to RIII, dtd 4/22/77.
5/ IE Inspection Rpt No. 50-263/76-18.