. . NRC FORM 366 (12-81) PPROVED BY OMB U.S. NUCLEAR REGULATORY COMMISSION 3150-0011 EXPIRES 4-30-82 LICENSEE EVENT REPORT PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION CONTROL BLOCK: 0 1 CON'T 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES On August 2, 1983, with the Plant in Mode 1, the borated water volume of 0 2 Accumulator "A" decreased to 7354 gallons because of valve leakage. Technical 0 3 Specification 3.5.1 requires that a minimum volume of 7368 gallons be maintained 0 4 in Modes 1, 2 and 3. There were no adverse consequences from this event. 0 5 Normal level and pressure were re-established within 18 minutes of the 0 6 occurrence. 0 7 0 8 8.0 COMP. VALVE CODE CAUSE CAUSE COMPONENT CODE VIAIL EI (12) VEX 0 9 1. SEQUENTIAL REPORT NO. CODE REVISION REPORT NO. YPE LER/RO REPORT NUMBER 813 01911 0 3 1L (17) 32 28 30 31 MANUFACTURER 26 COMPONENT SHUTDOWN CHMENT NPRD-4 IME COMP ACTION FUTURE Y 2 -UPPLIER ACTION ON PLANT METHOD N 24 X 18 B 19 1Z 121 Z 20 10 1010 A 25 R13 H 10 23 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The cause of this event was leakage through valves in the ECCS Check Valve Leak 1 0 Detection System. The valves were verified to be in the locked closed position. 1 1 A blank flange was then installed to stop the leakage until the values can be 1 2 repaired during the next Plant maintenance outage. 1 3 1 4 80 . METHOD OF FACILIT (30) (32) DISCOVERY DESCRIPTION OTHER STATUS DISCOVER POWER STATUS E 28 1 0 0 29 N/A A 31 Operator Observation 5 10 12 13 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) Z 33 Z 34 N/A N/A 1 6 80 10 11 TYPE DESCRIPTION 39 01010 7 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 0 0 0 0 N/A 1 8 1.1 12 LOSS OF OR DAMAGE TO FACILITY TE22 Z (2) N/A 1 9 8309090444 830831 PUBLICITY ISSUED DESCRIPTION (45) PDR ADOCK 05000395 NRC USE ONLY N 44 PDR 2 0 PHONE: (803)* 345-5209 NAME OF PREPARER (Mckinne J.

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764 ATLAND LORGIA

O. W. DIXON, JR. VICE PRESIDENT NUCLEAR OPERATIONS August 31, 1983EP 6 A9:34

Mr. James P. O'Reilly Regional Administrator U.S. Nuclear Regulatory Commission Region II, Suite 2900 101 Marietta Street, N.W. Atlanta, Georgia 30303

> SUBJECT: Virgil C. Summer Nuclear Station Docket No. 50/395 Operating License No. NPF-12 Thirty Day Written Report LER 83-091

Dear Mr. O'Reilly:

Please find attached Licensee Event Report #83-091 for Virgil C. Summer Nuclear Station. This Thirty Day Report is required by Technical Specification 6.9.1.13.(b) as a result of entry into Action Statement (a) of Technical Specification 3.5.1, "Emergency Core Cooling Systems," on August 2, 1983.

Should there be any questions, please call us at your convenience.

Very truly yours, O. W. Dixon Jr.

CJM:OWD/mac/fjc Attachment

cc: V. C. Summer E. H. Crews, Jr. T. C. Nichols, Jr.,/O. W. Dixon, Jr. E. C. Roberts H. N. Cyrus Group/General Managers O. S. Bradham R. B. Clary C. A. Price A. R. Koon D. A. Lavigne J. F. Heilman C. L. Ligon (NSRC)
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J. C. Miller
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Mr. James P. O'Reilly LER No. 83-091 Page Two August 31, 1983

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

On August 2, 1983, with the Plant in Mode 1, Operations personnel were attempting to identify a leakage source between the Emergency Core Cooling System (ECCS) and Train "B" of the Residual Heat Removal (RHR) System. The source of this leakage was suspected to be in the ECCS Check Valve Leak Detection System.

XVT-37, "ECCS Check Valve Leakage Detection Header Isolation Valve," was opened at 2330 hours to depressurize the header. At 2338 hours, the low pressure and level alarms were received for Accumulator "A." XVT-37 was closed at 2341 hours and Accumulator "A" pressure and level stabilized at 600 psig and 7354 gallons respectively. Action Statement (a) of Technical Specification 3.5.1.b applied since a minimum volume of 7368 gallons of borated water is required in each accumulator.

There were no adverse consequences resulting from this event. Normal level and pressure were re-established by 2356 hours on August 2, 1983.

CAUSE AND CORRECTIVE ACTIONS

The cause of the decrease in pressure and level in Accumulator "A" on August 2, 1983, is attributed to leakage through XVT-61A, "Accumulator 'A' Test Isolation Valve." The increase of pressure in the RHR System was apparently the result of leakage through XVT-61A and XVT-3B, "RH Header 'B' Drain Valves." Both of these valves are in the ECCS Check Valve Leak Detection System and are normally locked closed. The valves were verified to be in their correct positions.

The immediate corrective action taken by Operations personnel was to close XVT-37 and re-establish normal level and pressure in Accumulator "A." This action was completed at 2356 hours on August 2, 1983, and Action Statement (a) of Technical Specification 3.5.1 was exited at this time. The boron concentration was verified to be within the established Technical Specification limits at 0035 hours on August 3, 1983.

To prevent a future recurrence of this event, a blank flange was installed upstream of XVT-3B on August 18, 1983. This flange will remain in place until XVT-61A and XVT-3B can be repaired during the next Plant maintenance outage.