

LICENSEE EVENT REPORT

CONTROL BLOCK: [ ][ ][ ][ ][ ][ ] [1] (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

[01] T N S N P [1] [2] [0] [0] [ - ] [0] [0] [0] [0] [0] [0] [0] [0] [0] [0] [0] [0] [0] [0] [3] [4] [1] [1] [1] [1] [4] [ ] [ ] [5]

7 8 9 14 15 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45

LICENSEE CODE LICENSE NUMBER LICENSE TYPE JO CAT NO

CON'T  
[01] L [6] [0] [5] [1] [0] [1] [0] [3] [2] [7] [7] [0] [8] [0] [2] [8] [3] [8] [0] [9] [0] [1] [8] [3] [9]

7 8 60 61 68 69 74 75 80

REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES [10]  
[02] With unit 1 in mode 1 (100% power, 578 degrees F, 2235 psig) at 0745 CDT on  
[03] 08/02/83, the 480V shutdown board 2B2-B was made inoperable during the performance  
[04] of SMI-4A. The alternate supply was not energized when the board was transferred  
[05] from normal to alternate supply. There was no effect on public health or safety.  
[06] Previous occurrences - one (SQRO-50-327/83090).

[09] [E] [B] [11] [A] [12] [A] [13] [C] [K] [T] [B] [R] [K] [14] [A] [15] [Z] [16]

7 8 9 10 11 12 13 14 15 16 17 18 19 20

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

[17] [8] [3] [1] [0] [4] [ ] [ ] [0] [3] [L] [ ] [0]

21 22 23 24 26 27 28 29 30 31 32

LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

[H] [18] [Z] [19] [Z] [20] [Z] [21] [0] [0] [0] [0] [N] [23] [N] [24] [L] [25] [Z] [9] [9] [9]

33 34 35 36 37 40 41 42 43 44 47

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS [27]  
[10] DPSO had racked the breaker to test on the 6.9kV shutdown board feeding 2B2-B.  
[11] When the operator transferred from normal to alternate, he failed to observe the  
[12] voltages on the feeders. This was caused by the repetitiveness of several days of  
[13] board testing. The operator was instructed to read SOI-57.4H.4. The board was  
[14] returned to service at 0749 on 08/02/83. No further action will be taken.

[15] [E] [28] [1] [0] [0] [29] NA [B] [31] Operator observation

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

FACILITY STATUS % POWER OTHER STATUS [30] METHOD OF DISCOVERY [30] DISCOVERY DESCRIPTION [32]

[16] [Z] [33] [Z] [34] NA [ ] NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

ACTIVITY CONTENT AMOUNT OF ACTIVITY [35] LOCATION OF RELEASE [36]

[17] [0] [0] [0] [37] [Z] [38] NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION [39]

[18] [0] [0] [0] [40] NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PERSONNEL INJURIES NUMBER DESCRIPTION [41]

[19] [Z] [42] NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION [43]

[20] [N] [44] NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

ISSUED DESCRIPTION [45]

8309090326 830901 PDR ADOCK 05000327 S PDR  
Name of Preparer: G. E. Duggin / M. R. Harding  
Phone: (615) 870-6422  
NRC USE ONLY

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401  
1750 Chestnut Street Tower II

USNRC REGION II  
ATLANTA, GEORGIA

83 SEP 6 4 8: 56

September 1, 1983

Mr. James P. O'Reilly, Director  
U.S. Nuclear Regulatory Commission  
Suite 2900  
101 Marietta Street, NW  
Atlanta, Georgia 30303

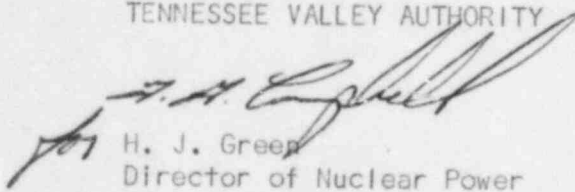
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET  
NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE  
REPORT SQRO-50-327/83104

The enclosed report provides details concerning the loss of power to  
480-volt shutdown board 2B2-B during a feeder breaker swap from normal to  
alternate. This report is submitted in accordance with Sequoyah unit 1  
Technical Specification 6.9.1.13.b.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
H. J. Green  
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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Institute of Nuclear Power Operations  
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Atlanta, Georgia 30339

NRC Inspector, Sequoyah

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