

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | A | L | B | R | F | 3 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

01 | L | 5 | 0 | 5 | 0 | 0 | 0 | 2 | 9 | 6 | 7 | 0 | 7 | 3 | 1 | 8 | 3 | R | 0 | 8 | 2 | 9 | 8 | 3 | 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
02 | While in steady state operation at 97% power, the accumulator alarm annunciated
03 | on control rod 54-47. Inspection showed a significant nitrogen leak through the
04 | "Star" valve on the accumulator. The control rod was considered inoperable per
05 | Technical Specification (T.S.) 3.3.A.2.e. There was no effect on the public
06 | health or safety. SLC was available and operable.

09 | R | B | 11 | E | 12 | B | 13 | V | A | L | V | E | X | 14 | G | 15 | D | 16

17 | 8 | 3 | 21 | 0 | 4 | 4 | 24 | 0 | 3 | 28 | L | 30 | 0 | 32

18 | A | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | 22 | Y | 23 | N | 24 | N | 25 | C | 4 | 8 | 7 | 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10 | The leak was caused by a worn packing on the combination "Star" valve (GE No.
11 | 158B7478P1). Control rod 54-47 was inserted to position "00" and the directional
12 | control valves were electrically disarmed. The packing was replaced and the
13 | control rod placed back in service. This is considered a random event and no
14 | further recurrence control is required.

15 | E | 28 | 0 | 9 | 7 | 29 | NA | 30 | A | 31 | Control Room Alarm | 32

16 | Z | 33 | Z | 34 | NA | 35 | NA | 36

17 | 0 | 0 | 0 | 37 | Z | 38 | NA | 39

18 | 0 | 0 | 0 | 40 | NA | 41

19 | Z | 42 | NA | 43

20 | N | 44 | NA | 45

8309070030 830829
PDR ADOCK 05000296
S PDR

NAME OF PREPARER D. A. Housley PHONE (205) 729-0845

LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 83044 Technical Specification Involved 3.3.A.2.e

Reported Under Technical Specification 6.7.2.b(2) * Date Due NRC 08/30/83

Event Narrative:

Unit 1 was in a refueling outage and unit 2 was operating at 94-percent power. Units 1 and 2 were unaffected by this event. Unit 3 was operating at 97-percent power when the accumulator alarm annunciated on control rod 54-47. Inspection of the hydraulic control unit for control rod 54-47 revealed a significant nitrogen leak through the star valve "O" ring on the accumulator. Due to the apparent size of the leak, the accumulator and control rod were considered inoperable per T.S. 3.3.A.2.e.

The leak was caused by a worn packing on the combination "Star" valve, (GE No. 158B7478P1.) In accordance with T.S. 3.3.A.2.f and T.S. 3.3.A.2.6, control rod 54-47 was inserted to position "00" and the directional control valves were electrically disarmed. The packing was replaced and the control rod placed back in service. Control rod 22-35 was also inoperable at this time due to a coupling problem. However, no more than one control rod in a 5x5 array was inoperable, and at least 4 operable control rods separated the inoperable controls rods as specified by T.S. 3.3.A.2.f.

There was no effect on the health and safety of the public. All technical specification requirements regarding operation with inoperable control rods were satisfied. Also, SLC was available and operable during this event.

* Previous Similar Events:

None

Retention: Period - Lifetime: Responsibility - Document Control Supervisor

*Revision: JRP

USNRC REGIONAL OFFICE
ATLANTA, GEORGIA
TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

83 AUG 31 P 1: 57
August 29, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET
NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE
REPORT BFRO-50-296/83044

The enclosed report provides details concerning an inoperable control rod
because of nitrogen leaking through a valve on the accumulator. This
report is submitted in accordance with Browns Ferry unit 3 Technical
Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



f H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

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U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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NRC Inspector, Browns Ferry

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