

EXHIBIT A

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10	
0 2	The Unit 1 once through steam generator (OTSG) had exhibited a small (approx. .007 gpm) primary to secondary
0 3	leak since plant startup from a recent refueling outage. On 7/7/83, while the unit was at 100% full power,
0 4	the leak slowly increased to approximately 0.1 gpm. The unit was shutdown for repair. During cooldown the
0 5	leak rate increased to approximately 0.2 gpm as estimated by volumetric analysis. A total of 11.87 curies of
0 6	activity was released; of this total, 11.85 curies were gaseous, predominately Xenon-133. No technical
0 7	specification (T.S.) limits on releases or leak rates were exceeded. This occurrence is reportable per T.S.
0 8	16.12.3.2.d. Similar occurrences were reported in LER's (50-313) 78-005, 80-026, 80-034, and 82-012.

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE		COMP SUBCODE		VALVE SUBCODE			
0 9	7 8	C C	11	E	12	C	13	H T E X C H	14	F	15	Z	16
LER/RO 17 REPORT NUMBER	EVENT YEAR B 3 21 22			SEQUENTIAL REPORT NO. 0 1 7 24 26				OCCURRENCE CODE 0 3 28 29		REPORT TYPE L 30		REVISION NO 0 32	
ACTION TAKEN X 18 33	FUTURE ACTION Z 19 34	EFFECT ON PLANT A 20 35	SHUTDOWN METHOD A 21 36	HOURS 0 4 6 4 22 37 40	ATTACHMENT SUBMITTED Y 23 41	NPRD-4 FORM SUB Y 24 42	PRIME COMP. SUPPLIER N 25 43	COMPONENT MANUFACTURER B 0 1 5 26 44 47					

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 | A single tube (77-18) was identified as the leaker. Eddy Current Testing confirmed the leak location to be

1 1 | at the interface of the tube and the upper OTSG tube sheet. The cause is believed to be intergranular stress

1 2 | corrosion cracking. The slight increase in leak rate during plant cooldown is consistent with increased OTSG

1 3 | tube tension and has been observed previously. An additional 533 steam generator tubes were inspected using

1 4 | eddy current methods. A total of 43 tubes were removed from service with plugs and upper end tube stabilizers.

7 8 | 9

FACILITY

FACILITY STATUS		% POWER	OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
1	5	1	1	0	0	1	0	1
7	8	9	10	12	13	14	15	16
ACTIVITY RELEASED		CONTENT	AMOUNT OF ACTIVITY		LOCATION OF RELEASE			
1	6	1	1	11.87 curies, pred. X3-133	RCS to secondary to condenser to atmosphere			
7	8	9	10	11	12	13	14	15

PERSONNEL EXPOSURES			
NUMBER		TYPE	DESCRIPTION
1	2	3	4
5	6	7	8
9	10	11	12
13	14	15	16
17	18	19	20
21	22	23	24
25	26	27	28
29	30	31	32
33	34	35	36
37	38	39	40
41	42	43	44
45	46	47	48
49	50	51	52
53	54	55	56
57	58	59	60
61	62	63	64
65	66	67	68
69	70	71	72
73	74	75	76
77	78	79	80
81	82	83	84
85	86	87	88
89	90	91	92
93	94	95	96
97	98	99	100

PERSONNEL INJURIES										
NUMBER					DESCRIPTION					
1	8	0	0	0	40	NA				

LOSS OF OR DAMAGE TO FACILITY		80
TYPE	DESCRIPTION	
11 9	2 42 NA	

PUBLICITY		ISSUED		DESCRIPTION		NRC USE ONLY											
2	0	Y	44	News Release													
7	8	9	10			145											
						68 69											

NAME OF PREPARER Patrick C. Rogers

PHONE: (501) 964-3100

8308150072 830806
PDR ADOCK 05000313
S PDR

LICENSEE EVENT REPORT

EXHIBIT A

LER No. 50-313/83-017/03L-0
Occurrence Date: 07/07/83

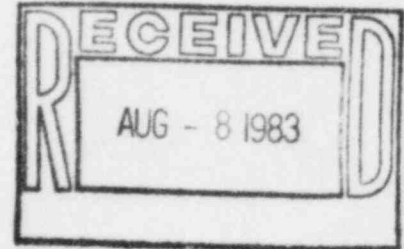
Cause Description and Corrective Actions (Continued)

Preliminary inspection results were discussed with the ANO-1 NRR Project Manager on July 15, 1983. Subsequently, more detailed inspection results were documented by letter 1CAN078312 on July 29, 1983. These inspection results and the results from the lab analysis of a previously pulled tube will be discussed in a future AP&L/NRC meeting.



ARKANSAS POWER & LIGHT COMPANY
POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

August 6, 1983



1CAN088303

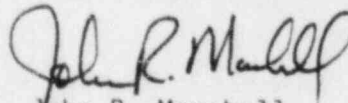
Mr. W. C. Seidle, Chief
Reactor Project Branch #2
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

SUBJECT: Arkansas Nuclear One - Unit 1
Docket No. 50-313
License No. DPR-51
Licensee Event Report
No. 83-017/03L-0

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 1 Technical Specification 6.12.3.2.d, attached is the subject report concerning a leak in the once through steam generator.

Very truly yours,


John R. Marshall
Manager, Licensing

JRM:RJS:rd

Attachment

cc: Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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