

PALISADES PLANT
Docket 50-255

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | M | I | P | A | L | 1 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 CAT 51 52

CON'T

0 1 | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 5 | 5 | 7 | 0 | 7 | 1 | 9 | 8 | 3 | 8 | 0 | 8 | 0 | 2 | 8 | 3 | 9

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

REPORT SOURCE 60 61 DOCKET NUMBER 66 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During the Palisades FSAR update process, reviewers discovered that the

0 3 | current resulting from a short-circuit in the 125 V DC System will result in

0 4 | the temperature of the associated conductors exceeding the value specified in

0 5 | the Palisades Plant FSAR, Section 8.5.2.3. On July 19, 1983, this condition

0 6 | was determined to be reportable per T.S. 6.9.2.A(8). No threat to public

0 7 | health or safety results.

0 8 | _____

7 8 9 80

0 9 | E | C | 11 | X | 12 | Z | 13 | E | L | E | C | I | O | N | 14 | Z | 15 | Z | 16

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

SYSTEM CODE 9 10 CAUSE CODE 11 12 CAUSE SUBCODE 13 14 COMPONENT CODE 15 16 COMP SUBCODE 17 18 VALVE SUBCODE 19 20

17 | L | E | R | R | O | R | E | P | O | R | T | N | U | M | B | E | R | 21 | 22 | 8 | 3 | 23 | 24 | 0 | 4 | 0 | 25 | 26 | 0 | 1 | 27 | 28 | 0 | 1 | 29 | 30 | T | 31 | 32 | 0 | 33

LEP/R/O REPORT NUMBER 21 22 EVENT YEAR 23 24 SEQUENTIAL REPORT NO. 25 26 OCCURRENCE CODE 27 28 REPORT TYPE 29 30 REVISION NO. 31 32

ACTION TAKEN 33 | X | 34 | X | 35 | Z | 36 | Z | 37 | 0 | 0 | 0 | 0 | 38 | Y | 39 | N | 40 | A | 41 | X | 42 | 9 | 9 | 9 | 43

FUTURE ACTION 33 34 EFFECT ON PLANT 35 36 SHUTDOWN METHOD 37 38 HOURS 39 40 ATTACHMENT SUBMITTED 41 42 NPRD-4 FORM SUB. 43 44 PRIME COMP. SUPPLIER 45 46 COMPONENT MANUFACTURER 47 48

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | Cause attributed to installation, in 1981, of larger capacity station

1 1 | batteries. Operability of the DC system during normal and accident

1 2 | conditions remains unaffected by this discrepancy, since the design load

1 3 | current does not result in unacceptable conductor temperatures.

1 4 | _____

7 8 9 80

1 5 | E | 28 | 0 | 8 | 9 | 29 | NA | 30 | C | 31 | FSAR Update Review | 32

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

FACILITY STATUS 28 29 % POWER 30 31 OTHER STATUS 32 33 METHOD OF DISCOVERY 34 35 DISCOVERY DESCRIPTION 36 37

1 6 | Z | 33 | Z | 34 | NA | 35 | NA | 36

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

ACTIVITY CONTENT 33 34 AMOUNT OF ACTIVITY 35 36 LOCATION OF RELEASE 37 38

1 7 | 0 | 0 | 0 | 37 | Z | 38 | NA | 39

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

PERSONNEL EXPOSURES 37 38 TYPE 39 40 DESCRIPTION 41 42

1 8 | 0 | 0 | 0 | 40 | NA | 41

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

PERSONNEL INJURIES 40 41 NUMBER 42 43 DESCRIPTION 44 45

1 9 | 2 | 42 | NA | 43

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

LOSS OF OR DAMAGE TO FACILITY 42 43 TYPE 44 45 DESCRIPTION 46 47

2 0 | N | 44 | NA | 45

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

PUBLICITY 44 45 ISSUED 46 47 DESCRIPTION 48 49

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NRC USE ONLY 68 69 70 71 72 73 74 75 76 77 78 79 80

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Consumers
Power
Company

General Offices: 1945 West Parnall Road, Jackson, MI 49201 • (517) 788-0550

August 2, 1983

James G Keppler, Administrator
Region III
US Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

DOCKET 50-255 - LICENSE DPR-20 -
PALISADES PLANT - LICENSEE EVENT REPORT 83-49 - DC SYSTEM SHORT-CIRCUIT
CURRENT IN EXCESS OF FSAR LIMIT

On the reverse please find Licensee Event Report 83-49 (DC System Short
Circuit Current in Excess of FSAR Limit), which is reportable to the NRC per
Technical Specification 6.9.2.a(8).

David J. Vandewalle
David J. Vandewalle

Nuclear Licensing Administrator

CC Administrator, Region III, USNRC
NRC Resident Inspector - Palisades

Attachment

AUG 05 1983

OC0883-0003A-NL02

IE 22
1/1

Attachment to LER 83-049
Consumers Power Company
Palisades Plant
Docket 50-255

While performing a review of the Palisades Plant FSAR for updating purposes, reviewers discovered that the current resulting from a short-circuit in the 125 V DC system (maximum DC system current) will result in the temperature of the associated conductors exceeding the values specified in the Palisades Plant FSAR Section 8.5.2.3. The condition results from the installation of larger capacity station batteries during the 1981 refueling outage.

Analysis shows that the FSAR specified conductor temperature limits will be exceeded for #4 AWG conductors and smaller assuming four cycles for system protection breakers to open.

Operation of the DC system under normal and accident conditions is not affected by the discrepancy because the design load current does not result in conductor temperatures that exceed the limits stated in the Palisades Plant FSAR.

Initial corrective measures under consideration are: 1) Evaluation of the DC loads that are fed by panels D11, D11A, D21, D21A and their feeder cables to determine the adequacy of the cables to withstand the higher short-circuit current capability of the new batteries. 2) Replacement of all inadequate cables with cables of sufficient current carrying capability to withstand the short-circuit current of the new batteries and maintain temperatures within the FSAR limits. 3) Provide a technical seminar for the Palisades Plant Technical Department to discuss system protection electrical considerations for future modifications.

Additional information on corrective measures will be provided when available.