APPROVED BY OME NRC FORM 366 (12-81) 10 CFR 50 U.S. NUCLEAR REGULATORY COMMISSION 3150 0011 LICENSEE EVENT REPORT CONTROL BLOCK (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) A S 0 S 2 0 0 - 0 0 0 0 0 0 0 3 4 1 1 1 0 CON'T REPORT L 60 5000 361 007148300729830 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (1) With the Unit in Mode 1 and following completion of a channel functional test on 0 2 [Core Protection Calculator (CPC) Channel A, a routine channel comparison was 0 3 Iconducted. During this comparison, it was discovered that certain CPC addressable 0 4 Iconstants in Channels B and D did not comply with the most recent values contained 0 5 in the CPC Addressable Constant Log. The incorrect values were assumed to be 0 6 inonconservative and CPC Channels B and D were declared inoperable in accordance 0 7 [with Technical Specification 2.2.2 at 1630. Public health and safety were not 0 8 affected by this event. COMP. VALVE CAUSE COD SUBCODE COMPONENT CODE CODE Z 15 A 1 12 X 13 |Z |Z |Z |Z | (14) Z 16 I | A | (1) ZZ 0 9 19 12 13 18 OCCURRENCE REVISION NO. REPORT SEQUENTIAL REPORT NO. CODE TYPE 17 HEPORT 0 7 3 T 0 8 3 | 01 31 28 30 22 29 32 24 HOURS 22 ATTACHMENT NPRD-4 PRIME COMP. COMPONENT 26 SHUTDOWN FUTURE EFFECT ON PLANT ACTION METHOD LY 23 N 24 Z 21 Z 25 X 1 (19) 01010101 Z 20 Z | 9 | 9 | 9 18 X CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) [Inoperability of CPC Channels B and D was due to addressable constants from the 10 I |channel functional test data base (i.e., "default" constants) being left in the 1 2] channels following completion of the 18-month channel calibration and functional 1 test. The correct constant values were entered into CPC Channels B and D within 3 4] one hour and the channels were declared operable. (See attachment). 80 METHOD OF FACILITY OTHER STATUS (30) DISCOVERY DESCRIPTICM (32) . POWER 5 B 28 0 5 0 29 A JOIL CPC Channel Comparison NA 1 80 8 9 10 ACTIVITY CONTENT RELEASED OF RELEASE 13 12 LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) Z 33 NA NA Z 34 6 1 80 5.5 10 PERSONNEL EXPOSURES NA 7 1 8.0 PERSONNEL INJURIES 180000 NA 80 1.1 12 . 64 8308080386 830729 LOSS OF OR DAMAGE TO FACILITY PDR ADOCK 05000361 NA Z (42) PDR 9 1 PUBLICITY ISSUED DESCRIPTION (45) NRC USE ONLY NA 2 0 N 44 6.8 HBPany/ NAMENAN PHONE (714) 492-7700 H. B. RAY NAME OF PPEPARER 27

ATTACHMENT TO LER 83-073 SOUTHERN CALIFORNIA EDISON COMPANY SAN ONOFRE NUCLEAR GENERATING STATION UNIT NO. 2, DOCKET NO. 50-361

SUPPLEMENTAL INFORMATION FOR CAUSE DESCRIPTION AND CORRECTIVE ACTION

A preliminary evaluation by Combustion Engineering, Inc. (CE) concluded that the use of default values for the addressable constants did not violate safety margins for 50% power operation. A final evaluation will be performed and if results to the contrary are obtained, a revision to this LER will be submitted.

As corrective action to prevent recurrence of this event, Computer Engineering is developing a program requiring a second verification of CPC constants on a periodic basis.

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION P.O. BOX 128 SAN CLEMENTE, CALIFORNIA 92672

July 29, 1983

1983 AUG -2 REGION VILLE TELEPHONE

17141 492-7700

TE-22 83-311

H. B. RAY STATION MANAGER

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U.S. Nuclear Regulatory Commission Office of Inspection and Enforcement Region V 1450 Maria Lane, Suite 210 Walnut Creek, California 94596-5368

Attention: Mr. J.B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361 14-Day Follow-Up Report Licensee Event Report No. 83-073 San Onofre Nuclear Generating Station, Unit 2

Letter, H B. Ray, (SCE) to J. B. Martin (NRC), Reference: dated July 15, 1982

The referenced letter provided you with confirmation of our prompt notification of a reportable occurrence involving the Core Protection Calculators (CPC).

Pursuant to Section 6.9.1.12.b of Appendix A, Technical Specifications to Facility Operating License NPF-10 for San Onofre Unit 2, this submittal provides the required 14-day follow-up report and a copy of Licensee Event Report (LER) No. 83-073 to address this event.

If there are any questions regarding the above, please contact me.

Sincerely,

HBRay Minnih

Enclosure: LER No. 83-073

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cc: A.E. Chaffee (USNRC Resident Inspector, Units 2 and 3)
J.P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U.S. Nuclear Regulatory Commission Office of Inspection and Enforcement

U.S. Nuclear Regulatory Commission Division of Technical Information and Document Control

Institute of Nuclear Power Operations (INPO)