NRC FORM 366 (12-81) 10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB 3150-0011

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE: A L B R F 2 (2) LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3) LICENSE TYPE: 4 1 1 1 1 (4) CAT: (5)
CON'T 0 1	REPORT SOURCE: L (6) DOCKET NUMBER: 0 5 0 0 0 2 6 0 (7) EVENT DATE: 0 6 0 7 8 3 (8) REPORT DATE: 0 7 0 6 8 3 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During normal operation while operating the drywell air compressor, FCV-64-34

0 3 torus exhaust valve bypass to SBGT, failed to meet its' five-second maximum

0 4 operating time per Technical Specification (TS) 3.7.D.1. Redundant valves

0 5 FCV-64-138, 64-140, and 64-31 were available and operable. In an accident

0 6 situation, FCV-64-34 would have isolated satisfactorily. There was no effect on

0 7 public health or safety.

0 8

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	S D (11)	X (12)	Z (13)	I N S T R U (14)	S (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 3 0	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER (26)
E (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	L (25)	N 0 0 7

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 FCV-64-34 failed to meet 5-second closure time due to indicating limit switches

1 1 being out of adjustment. FCV-64-34 was found operable and no work was performed

1 2 on valve. FCV-64-31, 139, and 141 were closed per TS 3.7.D.2. NAMCO (Model

1 3 D1200G-R) limit switches were adjusted and satisfactorily tested. This is

1 4 considered a random event and no recurrence control is required.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	E (28)	0 9 3 (29)	NA	B (31)	Surveillance Testing

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

PUBLICITY

	ISSUED	DESCRIPTION (45)
2 0	N (44)	

8307120418 830706
PDR ADOCK 05000260
S PDR

NRC USE ONLY

NAME OF PREPARER Glen D. Henry

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Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 83030 Technical Specification Involved 3.7.D.1

Reported Under Technical Specification 6.7.2.b(2) * Date Due NRC 07/07/83

Event Narrative:

Unit 1 was shut down for a refueling outage; unit 3 was operating at 99-percent power. These units were unaffected by this event. While unit 2 was operating normally at 93-percent power, a differential pressure check was made during operation of the drywell differential air compressor and FCV-64-34, torus exhaust valve bypass to standby gas treatment system, failed to meet its' five-second maximum operating time (Technical Specification 3.7.D.2 and Table 3.7.A.) FCV-64-31, 139, and 141 were closed to permit continued reactor power operation. In an accident situation, FCV-64-34 would have isolated. An urgent maintenance request was written to investigate and repair. FCV-64-34 was found to be operable; however, the limit switches which give control room indication were out of adjustment. The limit switches were adjusted to indicate proper valve position and satisfactorily tested. There was no effect on public health or safety. Redundant valves FCV-64-139, 64-140, and 64-31 were available and operable. This is considered a random event and no recurrence control is required.

* Previous Similar Events:

None

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY
CHATTANOOGA, TENNESSEE 37401
1750 Chestnut Street Tower II

July 6, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE REPORT BFRO-50-260/83030

The enclosed report provides details concerning a torus exhaust valve bypass to standby gas treatment which failed to operate within required time limits. This report is submitted in accordance with Browns Ferry unit 2 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. J. Green
Director of Nuclear Power

Enclosure
cc (Enclosure):
Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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Institute of Nuclear Power Operations
Suite 1500
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NRC Inspector, Browns Ferry

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