NRC FORM 366 (12-81) 10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB 3150-0011

LICENSEE EVENT REPORT

CONTROL BLOCK: ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | A L B R F 1 ② | 0 0 - 0 0 0 0 0 - 0 0 ③ | 4 1 1 1 1 ④ | ⑤
LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T 0 1 | REPORT SOURCE L ⑥ | 0 5 0 0 0 2 5 9 ⑦ | 0 4 1 8 8 3 ⑧ | 0 4 2 9 8 3 ⑨
DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 During refueling outage while performing SI 4.7.A.2.g-3 the combined leakage
0 3 of the "A" line MSIVs was found to be 4883.4 SCFH which exceeded the allowable
0 4 limit of 11.5 SCFH per valve (T. S. 4.7.A.2.i). (During further testing, B, C,
0 5 and D line MSIVs also exceeded 11.5 SCFA per valve). This resulted in the
0 6 total type B and C tests leakage rate exceeding 60% La (655.9 SCFH)
0 7 (T. S. 4.7.A.2.g). There was no release of radioactivity as a result of isolation
0 8 valve leakage. There was no effect on public health or safety.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	C D ⑪	B ⑫	B ⑬	V A L V E X ⑭	F ⑮	D ⑯

⑰ LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 2 0	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS ㉒	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER ㉖
B ⑱	F ⑲	Z ⑳	Z ㉑	0 0 0 0 0	Y ㉓	Y ㉔	N ㉕	A 5 8 5

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 The Atwood-Morrill 26-inch globe valves were leaking past the main seat.
1 1 Leakage was caused by deterioration of MSIV seating surfaces during operation.
1 2 Prior to startup the valves will be repaired and retested until satisfactory
1 3 leakage rates are obtained. A long-term MSIV improvement program is being
1 4 implemented (Reference BFRO 50-260/82032).

	FACILITY STATUS	% POWER	OTHER STATUS ㉚	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION ㉜
1 5	H ㉘	0 0 0 ㉙	NA	B ㉛	Surveillance Tests

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY ㉟	LOCATION OF RELEASE ㊱
1 6	Z ㉝	Z ㉞	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION ㊴
1 7	0 0 0 ㊲	Z ㊳	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION ㊶
1 8	0 0 0 ㊵	NA

LOSS OF OR DAMAGE TO FACILITY ㊸

	TYPE	DESCRIPTION
1 9	Z ㊷	NA

8305050272 830429
PDR ADOCK 05000259
S PDR

PUBLICITY

	ISSUED	DESCRIPTION ㊺	NRC USE ONLY
2 0	N ㊹	NA	

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Tennessee Valley Authority
Browns Ferry Nuclear Plant

Form BF 17
BF 15.2
2/12/82

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 83020 Technical Specification Involved 3.7.A.2 4.7.A.2.i, 4.7.A.2.g

Reported Under Technical Specification 6.7.2.a(3) * Date Due NRC 5/2/83

Event Narrative:

Units 2 and 3 were operating normally at 95-percent and 99-percent power respectively. Unit 1 was in a refueling outage and was the only unit affected by this event. During the performance of Surveillance Instruction 4.7.A.2.g-3, the combined leakage of the "A" line inboard and outboard MSIVs was 4883.4 SCFH which exceeded the allowable limit of 11.5 SCFH per valve (Technical Specification 4.7.A.2.i). (During further testing "B", "C", and "D" MSIVs also exceeded Technical Specification 4.7.A.2.i leakage limits, with line leakage at 30.3, 2633.2, and 2762.3 SCFH, respectively). Using conservative assumptions, the leak rate applicable to each valve was 2441.7 SCFH. This resulted in the type "B" and "C" tests leakage rate exceeding 60-percent La (655.9 SCFH) (Technical Specification 4.7.A.2.g). The 26-inch Atwood-Morrill globe valves were leaking past the main seat. This leakage was caused by deterioration of the MSIV valve seating surfaces during operation. Prior to startup these valves will be repaired and retested until satisfactory leak rates are obtained. There was no release of radioactivity as a result of the MSIVs leaking. There was no effect on the health and safety of the public.

A long term MSIV improvement program is being implemented which should improve the seating performance of these valves. (Reference BFRO 50-260/82032)

* Previous Similar Events:

BFRO 50-259/77023, 78034, 80003, 81014
BFRO 50-260/79077, 80092, 82030, 82037
BFRO 50-296/78025, 79014, 80058, 80059, 81073

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP