



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 21, 1997

Mr. Dwight C. Mims, Director
Nuclear Safety Division
Entergy Operations, Inc.
1448 SR 333
Russellville, Arkansas 72901

SUBJECT: ACCEPTANCE FOR REFERENCING OF TOPICAL REPORT SIR-94-080,
"RELAXATION OF REACTOR COOLANT PUMP FLYWHEEL INSPECTION
REQUIREMENTS"

Dear Mr. Mims:

We have completed our review of the subject topical report submitted in a letter dated April 4, 1995 by Entergy Operations, Inc. (Entergy) for Arkansas Nuclear One (ANO) -1 & -2 as lead plants. The report is acceptable for referencing in license amendment applications to the extent specified and under the limitations delineated in the report and the enclosed safety evaluation (SE). The staff concluded that the flywheel inspection period could be lengthened from the current 3 years to 10 years for the plants identified in the SER. Total elimination of flywheel inspections is not justified.

We do not intend to repeat our review of the matters described in the report when the report appears as a reference in license amendment applications, except to ensure that the material presented is applicable to the specific plant involved as indicated in the conclusion section of the SE. Our acceptance applies only to the matters described in the report.

In accordance with procedures established in NUREG-0390, it is requested that Entergy coordinate with the ABB Combustion Engineering Owners Group and publish this report within three months of receipt of this letter. The final version shall incorporate this letter, the enclosed evaluation, and Entergy's response dated December 9, 1996 to the NRC request for additional information, between the title page and the abstract. The final version shall include an -A (designating accepted) following the report identification symbol.

Licensees for ANO-2, Palisades, Millstone 2, Waterford 3, and St. Lucie 1 & 2 need to verify the reference temperature RT_{NDT} for their reactor coolant pump (RCP) flywheels and demonstrate, with plant-specific test results if possible,

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that the corresponding fracture toughness (K_{Ic}) values are equivalent to those reported in the topical report. ANO-1, which already has a 10-year inspection interval for its flywheels, will not be affected. The topical report indicated that flywheels for Waterford 3 could lose shrink fit at the accident speed. The staff will pursue this issue with Waterford 3 on a plant-specific basis.

Sincerely,

/s/

Brian W. Sheron, Director
Division of Engineering
Office of Nuclear Reactor Regulation

Enclosure: As stated

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corresponding fracture toughness (K_{Ic}) values are equivalent to those reported in the topical report. ANO-1, which already has a 10-year inspection interval for its flywheels, will not be affected. Licensees for Millstone 2 and Waterford 3 are denied the benefit of the 10-year interval for RCP flywheel inspections approved by the staff in this SE.

Sincerely,

Brian W. Sheron, Director
Division of Engineering
Office of Nuclear Reactor Regulation

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