



EDISON PLAZA
300 MADISON AVENUE
TOLEDO, OHIO 43652-0001

March 13, 1990
KB90-0119

Docket No. 50-346
License No. NPF-3

Document Control Desk
U. S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20555

Gentlemen:

Monthly Operating Report, February 1990
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit No. 1 for the month of February 1990.

During review of the NRC's "Licensed Operating Reactors Status Summary Report" (NUREG-0020/Vol. 13, No. 11/November 1989), Toledo Edison noted several errors in the data for the Davis-Besse Nuclear Power Station. These errors appear to have been propagated from a typographical error in the Gross Thermal Energy (Item 17). The figure listed in Item 17 for the month is 2,507,046. Davis-Besse records indicate that the correct figure is 2,057,046. This error was found to have affected the year-to-date and cumulative figures for Gross Thermal Energy, as well as, the year-to-date Unit Capacity Factor (MDC) listed in Item 22.

Toledo Edison requests that these corrections be made, and that they be reflected in future issuances of the "Licensed Operating Reactors Status Summary Report."

If you have any questions, please contact Bilal Sarsour at (419) 321-7384.

Very truly yours,

Louis F. Storz
Plant Manager
Davis-Besse Nuclear Power Station

BMS/mjb

Enclosures

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March 13, 1990
Distribution

cc: Mr. A. Bert Davis
Regional Administrator, Region III

Mr. Paul Byron
NRC Resident Inspector

Mr. T. V. Wambach
NRC Senior Project Manager

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346

UNIT Davis-Besse #1

DATE March 13, 1990

COMPLETED BY Bilal Sarsour

TELEPHONE 419-321-7384

MONTH February 1990

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	
30	
31	

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-346
 DATE March 13, 1990
 COMPLETED BY Bilal Sarsour
 TELEPHONE 419-321-7384

OPERATING STATUS

1. Unit Name: Davis-Besse Unit #1
 2. Report Period: February 1990
 3. Licensed Capacity (MWt): 2772
 4. Nameplate Rating (Gross MWe): 925
 5. Design Electrical Rating (Net MWe): 906
 6. Maximum Dependable Capacity (Gross MWe): 918
 7. Maximum Dependable Capacity (Net MWe): 874

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____

10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	672.0	1,416.0	101,545.0
12. Number Of Hours Reactor Was Critical	0.0	608.8	54,763.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	5,393.7
14. Hours Generator On-Line	0.0	608.8	52,809.2
15. Unit Reserve Shutdown Hours	0.0	0.0	1,732.5
16. Gross Thermal Energy Generated (MWH)	0.0	1,603,325	126,567,245
17. Gross Electrical Energy Generated (MWH)	0.0	536,477	41,886,778
18. Net Electrical Energy Generated (MWH)	0.0	508,740	39,295,898
19. Unit Service Factor	0.0	43.0	52.0
20. Unit Availability Factor	0.0	43.0	53.7
21. Unit Capacity Factor (Using MDC Net)	0.0	41.1	44.3
22. Unit Capacity Factor (Using DER Net)	0.0	39.7	42.7
23. Unit Forced Outage Rate	0.0	18.2	28.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: June 5, 1990

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346

UNIT NAME Davis-Besse #1DATE March 13, 1990COMPLETED BY Bilal SarsourTELEPHONE (419) 321-7384REPORT MONTH February 1990

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
2	90-02-01	S	672	C	9	NA	NA	NA	The refueling outage was entered following the reactor trip on January 26, 1990, to perform scheduled maintenance and refueling work.

¹F: Forced
S: Scheduled

²Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation from
Previous Month
5-Load Reduction
9-Other (Explain)

⁴Exhibit G - Instructions for Preparation of Data
Entry Sheets for Licensee Event Report (LER)
File (NUREG-0161)

⁵Exhibit I - Same Source
*Report challenges to Power Operated Relief Valves
(PORVs) and Pressurizer Code Safety Valves (PCSVs)

REFUELING INFORMATION

Date: February 1990

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling outage? September 1991
3. Scheduled date for restart from current refueling: June 1990
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: A license amendment request to remove cycle-specific values from Section 3 of the Technical Specifications was submitted to the NRC on June 16, 1989, based on Generic Letter 88-16, and was approved in January 1990. No Cycle 7 Technical Specification changes to the Technical Specification, except for Bases changes, will be submitted.

5. Scheduled date(s) for submitting proposed licensing action and supporting information: Changes to the bases were submitted on February 2, 1990.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
 - a. Sixty Batch Reload 3.38% enriched.
 - b. New fuel design Mark B8A (Reconstitutable, removable upper end fitting, Zircaloy grid spacer, debris resistant lower end cap, lower prepressurization, and annealed guide tubes).
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool, and (c) the new fuel storage areas.

(a) 177 (b) 328 (c) 0
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735 Increased size by: approximately 900 by 1994 is planned
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1996 - assuming ability to unload the entire core into the spent fuel pool is maintained