## CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

RR#1 • BOX 127E • EAST HAMPTON, CT 06424-9341

March 15, 1990 Re: Technical Specification 6.9.1d

Docket No. 50-213

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Dear Sir:

In accordance with reporting requirements of Technical Specification 6.9.1d, the Connecticut Yankee Haddam Neck Plant Monthly Operating Report 90-02, covering operations for the period February 1, 1990 to February 28, 1990 is hereby forwarded.

Very Truly yours,

Donald B. Miller, Jr. Station Superintendent

#### DBM/mdw

cc: (1) Regional Administrator, Region 1 U. S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406

> (2) John T. Shedlosky Sr. Resident Inspector Connecticut Yankee

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Connecticut Yankee Atomic Power Company
Haddam Neck Plant
Haddam, Connecticut

Monthly Operating Report No. 90-01

For The Month of

February 1990

## Plant Cperations Summary - February, 1990

The following is a summary of Plant Operations for February, 1990. The 15th refueling and maintenance outage continued for the month of February.

TION SPECIAL PRECAUTIONS TAKEN TO PROVIDE POR REACTOR SAFETY DURING REPAIR	
CORRECTIVE ACTION TAKEN TO PREVENT REPETITION	
EPFECT ON SAPE OPERATION	
3/90 TION RESULT	
IEC 3/ MALFUNCTION CAUSE RESULT	
SYSTEM OR COMPONENT	There were no reportable items for 1&C for the month of February, 1990.

SPECIAL PRECAUTIONS TAKEN TO PROVIDE POR REACTOR SAFETT DUBING REPAIR	
CORRECTIVE ACTION TAKEN TO PREVENT REPETITION	
EPPECT ON SAPE OPERATION	
3/90 TON RESULT	
Maintenance Maintenance CAUSE	
SYSTEM OR CONTPONENT	There were no reportable items for Maintenance for the month of February, 1990.

### AVERAGE DATLY UNIT POWER LEVEL

DOCKET NO.	50-213
UNIT	Conn. Yankee Haddam Neck
DATE	3-15-90
COMPLETED BY	K. C. Wall
TELEPHONE	(203) 267-3654

ONTH: February		
OAY AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1 0	17	0
2	18	
3	19	
4	20	
5	21	
5	22	
7	23	
8	24	
9	25	
10	26	
11	27	
12	28	Ö
13	29	
14	30	
15	31	
16		

### INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Complete the nearest whole megawatt.

## CONNECTICUT YANKEE REACTOR COOLANT DATA MONTH: FEBUARY 1990

REACTOR COOLANT ANALYSIS	MINIMUM	AVERAGE	MAXIMUM
***************************************		: 4.57E+00	4.70E+00 1
PH @ 25 DEGREES C	: 4.51E+00 : 7.93E+00	: 9.13E+00	
CONDUCTIVITY (UMHOS/CM) CHLORIDES (FPM)	: <5.00E-02	: <5.00E-02	: <5.00E-02 :
DISSOLVED OXYGEN (PPB)	: <5.00E+00	: <5.00E+00 : 2.66E+03	
BORON (PPM)	: 2.64E+03 : 0.00E-01	: 0.00E-01	
TOTAL GAMMA ACT. (UC/ML)	: 3.68E-03	: 4.11E-03	: 4.82E-03 :
IDDINE-131 ACT. (UC/ML)	: 0.00E-01		
I-131/I-133 RATIO	: 0.00E-01 : <1.00E-02		
TRITIUM (UC/ML)	: 1.83E-02		: 2.28E-02 :
HYDROGEN (CC/KG)	: 2.00E-03	: 2.00E-03	: 2.00E-03 :

WASTE LIGHTD PROC	FESED THROUG	WASTE PROCESS	ERY (	BALLONS):	5.07E+04 5.32E+04
AVERAGE PRIMARY TO SE	PRIMARY LEAK	RATE (GALLONS	FER	WINDLESS	0.005-01

<sup>\*</sup> Core off loaded to spent fuel pool. Accordingly Boron data is for the pool.

#### NRC OPERATING STATUS REPORT

#### Haddam Neck

1. Docket: 50-213

3. Utility Contact: J. Stanford (203) 267-3635

4. Licensed Thermal Power (MWt): 1825

5. Nameplate Rating (Gross MWe): 667 x 0.9 x 600.3

6. Design Electrical Rating (Net MWe): 582

7. Maximum Dependable Capacity (Gross MWe): 591.8

8. Maximum Dependable Capacity (Net MWe): 565

9. If changes occur above since last report, reasons are: NONE

10. Power level to which restricted, if any (Net MWe): N/A

11. Reasons for restriction, if any: N/A

	MONTH	YEAR-TO-DATE	CUMULATIVE
12. Report period hours:	672.0	1,416.0	194,280.0
13. Hours reactor critical:	0.0	0.0	158,249.2
14. Reactor reserve shutdown hours:	0.0	0.0	1,285.0
15. Hours generator on-line:	0.0	0.0	152,087.6
16. Unit reserve shutdown hours:	0.0	0.0	398.0
17. Gross thermal energy generated (MWtH):	0.0	0.0	262,867,318.0 *
18. Gross electrical energy generated (MWeH):	0.0	0.0	86,093,046.0 *
19. Net electrical energy generated (MWeH):	-1,774.3	-3,883.2	81,819,096.8 *
20. Unit service factor:	0.0	0.0	78.3
21. Unit eveilability factor:	0.0	0.0	78.5
22. Unit capacity factor using MDC net:	0.0	0.0	76.7
23. Unit capacity factor using DER net:	0.0	0.0	72.4
24. Unit forced outage rate:	0.0	0.0	5.6
25. Forced outage hours:	0.0	0.0	8,975.6

<sup>26.</sup> Shutdowns scheduled over next 6 months (type, date, duration): NONE

<sup>27.</sup> If currently shutdown, estimated startup date: 6/10/90

<sup>\*</sup> Cumulative values from the first criticality (97/24/67). (The remaining cumulative values are from the first date of commercial operation, 01/01/68).

# UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-213
UNIT NAME Connecticut Yank
DATE 3-15-90
COMPLETED BY K. C. Mall
TELEPHONE 203-267-3654

REPORT MONTH

February 1990

No.	Date	Type	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor	LER RPT.	System Code	Component S	Cause & Corrective Action to Prevent Recurrence
89-03	2/1/90	S	672	С	1	N/A	RC	Fuel XX	Continuation of Core XV-XVI Refueling.

F Forced S Scheduled Reason:

A-Equipment Failure (Explain)

H-Other(Explain

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Other(Explain)

4

Exhibit G-Instructions for Preparation of Data Entry Sheets for Licenses Event Report (LER) File (NUREG-0161)

5

Exhibit 1 Same Source

### Refueling Information Request

1. Name of facility

Haddam Neck

Scheduled date for next refueling shutdown.

June 27, 1991

Scheduled date for restart following refueling.

June 25, 1990

4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes

(b) If answer is yes, what, in general, will these be?

Incorporate the guidance provided in the NRC Generic Letter 88-16. The Generic Letter addresses removing cycle specific parameters from Technical Specifications and transferring them to the technical report supporting cycle operation.

(c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload? (Ref. 10 CFR section 50.59)

n/a

(d) If no such review has taken place, when is it scheduled?

n/a

Scheduled date(s) for submitting proposed licensing action and supporting information.

The TSCR was submitted to the NRC on July 28, 1989.

Important licensing considerations associated with refueling, e.g., new or different fuel design
or supplier, unreviewed design or performance analysis methods, significant changes in fuel
design, new operating procedures.

No

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 0 (b) 858

 The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

1168

 The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

1996