

John A. Bailey Vice President Nuclear Operations

March 5, 1990

NO 90-0064

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Mail Station P1-137 Washington, D. C. 20555

> Reference: Letter ET 88-0082 dated June 13, 1988 from J. A. Bailey, WCNOC, to Document Control Desk, NRC

Subject: Docket No. 50-482: Licensee Event Report 88-007-01

Gentlemen:

The Reference submitted Licensee Event Report 88-007-00 pursuant to 10 CFR 50.73 (a)(2)(1) concerning a violation of Technical Specifications. Subsequent review of the situation determined that the surveillance test procedure had satisfied Technical Specification 2.2.1. The Reference is being supplemented as a voluntary report to clarify the method of setting the trip setpoints.

Very truly yours,

John a. Barley

John A. Bailey Vice President Nuclear Operations

JAB/aem

9003150043 900305

PDR ADOCK 05000482

PDC

Attachment

cc: E. J. Holler (NRC), w/a
R. D. Martin (NRC), w/a
D. V. Pickett (NRC), w/a
M. E. Skow (NRC), w/a

1722

P.O. Box 411 / Burlington, KS 65839 / Phone: (316) 364-8831 An Equal Opportunity Employer M/F/HC/VET

| | | | LIC | ENSE | E EVE | NT RE | PORT | (LER) | U.S. NL | APPRON | | DRY COMMISSI NO. 2150-0106 | | |
|----------------------------------|--|---|--|---|--|---|-----------------------------------|--|--|------------------------|-------------------------|-------------------------------|--|--|
| AGILITY NAME (1) | | | | | | | | | DOCKET NUMBER | (2) | | PAGE (S | | |
| WO. | lf Creek | (Generatin | ng Stati | on | | _ | | | 0 5 0 0 | 1014 | 1812 | 1 OF O | | |
| Contraction of the second second | thodolox | y Of React | tor Trin | Sve | tom Se | ennin | + 0-1 | hention | | | | | | |
| EVENT DATE (| | LER NUMBER | The second second second second second | a second s | PORT DAT | contraction of the second state of | t car | Contractions, for any other states of the st | FACILITIES INVO | LVED (6) | | | | |
| MONTH DAY | YEAR YEAR | SEQUENTIAL NUMBER | REVIDION | MONTH | DAY | YEAR | | FACILITY NAM | AES | DOCKE | TNUMBER | R(S) | | |
| | | | | | | | | | | 0 15 | 1010 | 1011 | | |
| 0 5 1 3 | 8 8 8 8 | 0 0 0 7 | - 011 | 0 3 | DIE | olo | | | | | | | | |
| | | EPORT IS SUBMITTE | hand in hims | 0 0 | | 90 | CER 8. // | Charle one or more i | of the Initawinal 11 | Arrandores | 1010 | 1011 | | |
| MODE (B) | | 0.402(6) | | 20.405 | | | | 50.73(a)(2)(iv) | an and advertisingly an | | 3.71(6) | | | |
| POWER | | 0.405(4)(1)(0 | | 60.381 | 0)(1) | | | 50.73(a)(2)(v) | | , | 3.71(c) | | | |
| (10) 11 (| Condessand | 0.4051011111 | - | 60.361 | | | - | 50.73(a)(2)(vii) | | | | ecity in Abstract | | |
| | COST Comment | 0.405(a)(1)(iii) | | 50.73ie | | | | 50.73(e)(2)(viii)() | | 3 | 66A/ | | | |
| | | 0.408(s)(1)(v) | - | 80.734 | | | - | 60.73(a)(2)(viii)(| | | | | | |
| | | | L | ICENSEE | CONTACT | FOR THIS | LEN (12) | | | | | | | |
| AME | | Sara Pere | | | | | | | AREA CODE | TELEPH | ONE NUM | BER | | |
| Mer | rlin G. | Williams | | | | | | | 31116 | 3.6 | 14. | 18 18 12 | | |
| | | | ONE LINE FOR | EACH C | OMPONENT | FAILURE | DESCRIBE | D IN THIS REPOR | or the state of th | 1 310 | 141- | 18 18 13 | | |
| CAUSE SYSTEM | COMPONENT | MANUFAC TURER | REPORTABLE TO NPRDS | | | CAUSE | SYSTEM | COMPONENT | MANUFAC- TURER | | ATABLE | | | |
| | 111 | | | | | | | | | | | | | |
| | 111 | 1111 | | | | | 1 | 111 | 1111 | | | | | |
| | | BUPPLEME | INTAL REPORT | EXPECT | ED (14) | | | Lander Arrideen | EXPECT | | MONTH | DAY YE | | |
| | | D SUBMISSION DATE | | - | XI NO | | | | SUBMISSI DATE (1 | ON | | | | |
| me Ir pu | ethodolo nstrumer rocedure OPdT) ar | DT on May ogy, the ut at and Cont as for sett d Overtemp comply with | ility R crol (I& ing the perature | c) su trip Diff | or Eng upervi poin ferent Specif | incer: sor de t for ial Te icatio | overn overn mpera ons 2. | pervisor ined cons power Dif: ature (ON | and a ut arvativel ferential T) instr | ility y the Temp | at th perat t loo | ure ps | | |

| NRC (DRD 3664 | U.S. NUCLEAR REGULATORY COMMISSION | TORY COMMISSION APPROVED OME NO. 3150-0104 EXPIRES 4/30/92 | | | | | | | | |
|--|------------------------------------|---|----|---|----------|--|---|-------------------------------------|---|---|
| | ENT REPORT (LER) | | | D BURDEN PE IDN COLLECT SREGARDING RTS MANAGEI RV COMMISSI RWORK REDU EMENT AND B | ION BURD | REQUEST DEN ESTIM BRANCH VASHINGT N PROJEC | 50.0 H ATE TO (P-530). ON, DC T (3150 | RS. FC THE R U.S. N 20565. | RWARD ECORDS UCLEAR AND TO OFFICE | |
| FACILITY NAME (1) | DOCKET NUMBER (2) | | LE | R NUMBER 16 | 5 | | | PAGE | (3) | |
| | | VERR | 1 | NUMBER | 1 | NUMBER | | T | | - |
| Wolf Creek Generating S | tation 0 5 0 0 4 8 2 | 8 8 | - | 01017 | - | 011 | 013 | OF | 0 15 | ; |
| TEXT III more space is required, use additional NRC Form 305 | A'\$/ (17) | | | | 2008 | | | | | |

INTRODUCTION

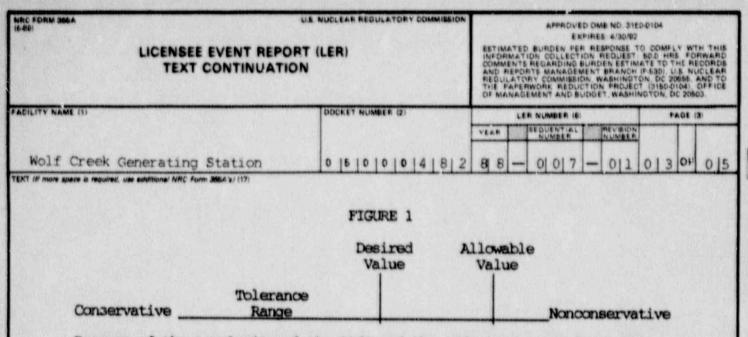
At 1230 CDT on May 13, 1988, while reviewing the surveillance test methodology, the utility Reactor Engineering supervisor and a utility Instrument and Control (I&C) supervisor conservatively determined that the procedures for setting the trip point for Overpower Differential Temperature (OPdT) and Overtemperature Differential Temperature (OTdT) instrument loops [JC-AS] contained an error. It was then determined that the surveillance test procedure for the Analog Channel Operational Test of the 7300 Process Instrumentation [JC-IA] did not comply with Technical Specifications 2.2.1, action statement "a", which states:

"a. With a Reactor Trip System Instrumentation or Interlock Setpoint less conservative than the value shown in the Trip Setpoint column but more conservative than the value shown in the Allowable Value column of Table 2.2-1, adjust the Setpoint consistent with the Trip Setpoint value."

The unit was operating in Mode 1, Power Operation, at 100 percent power when this problem was identified. Revision 0 of this Licensee Event Report (LER) was submitted pursuant to 10CFR 50.73(a)(2)(i)(B), as a violation of Technical Specification 2.2.1, based on the above determination. Upon further review of the situation in August, 1989, it was determined that the surveillance test procedure had satisfied Technical Specification 2.2.1, Action Statement 'a' prior to May, 1988. The original Licensee Event Report is being supplemented as a voluntary report to clarify the method of setting the trip setpoints.

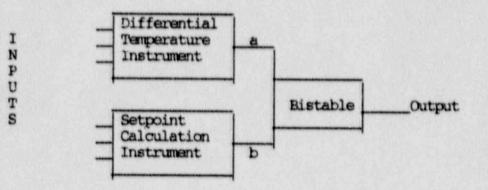
DESCRIPTION

The subject surveillance test procedure tests the setpoint for each of the reactor trip signals generated by the 7300 Process Instrumentation. The general methodology used therein is to inject a test signal at the detector and to note the trip point of the applicable bistable. For each trip function, there is listed a desired value, a tolerance range and an allowable value. The relationship of these three are shown in Figure 1. The desired value corresponds to the Trip Setpoint of Technical Specifications, Table 2.2-1; the tolerance range is conservative with respect to the desired value and the allowable value of the procedure corresponds to the Allowable Value Column of Table 2.2-1. During calibration the trip point of each bistable is reset to the tolerance range unless it is found already in the tolerance range.



Because of the complexity of the OPdT and the OTdT instrument loops the procedure calibrated each instrument loop in three sections. Figure 2 provides a simplified block diagram of one instrument loop. The bistables for the OPdT and OTdT instrument loops have two inputs; one corresponding to the differential temperature and the other corresponding to a calculated trip setpoint. The surveillance test procedure calibrated the OPdT and OTdT bistables by supplying a test signal at the two inputs to the bistables (labeled "a" and "b" on Figure 2) and setting the trip in the normal manner. Each of the input instruments feeding the bistable was calibrated to a plus or minus tolerance on either side of the nominal value.

FIGURE 2



MAY 1988 ANALYSIS

In May of 1988, it was believed that the procedure assumed that a perfect signal would be supplied at "a" and "b" and set the bistable trip point accordingly. It was further concluded that the procedure failed to recognize that instrument error in the two input instruments would affect the instrument loop trip point. The resultant error for the instrument loop is the sum of the bistable error plus the error in each input instrument. Since the resultant error could be nonconservative, this procedure allowed the instrument loop trip setpoint of OPdT and OTdT instruments to be less conservative than the value shown in the Trip Setpoint columns of Technical Specifications Table 2.2-1, resulting in violation of action statement "a".

| LICENSEE EVENT REPORT TEXT CONTINUATION | APPROVED DME NO. 3150-0104 EXPIRES 4/30/92 ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P.530). U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20505. AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104). OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503. | | | | | | | |
|---|---|------|--------------|-------------------|----------|-----|----|------|
| FACILITY NAME (1) | DOCKET NUMBER (2) | | ER NUMBER (6 | PAGE (3) | | | | |
| | | YEAR | L | SEQUENTIAL NUMBER | REVISION | | T | |
| Wolf Creek Generating Station | 0 6 0 0 4 8 2 | 8 8 | - | 0 0 7 | -011 | 014 | OF | 0 15 |
| TEXT III more space is required, use additional NRC Form 366A's/ (17) | | | | | | | | |

ALGUST 1989 ANALYSIS

Upon further review of the situation in August, 1989, it was recognized that the Bases for Technical Specification 2.2.1 states that "The Setpoint for a Reactor Trip System or interlock function is considered to be adjusted consistent with the nominal value when the 'as measured' setpoint is within the band allowed for calibration accuracy".

The calibration accuracy has been evaluated and specifically defined. It provides a tolerance range encompassing the Trip Setpoint value. Therefore, following performance of the calibration procedure, the bistable setpoint should be left at the Trip Setpoint value plus or minus the calibration accuracy for compliance with this Technical Specification 2.2.1, Action Statement "a".

Consequently it is planned to revise the method of calibration to the original method.

ADDITIONAL INFORMATION

This surveillance test procedure is done monthly when the unit is in Mode 1 (Power Operation) or Mode 2 (Startup). There are four OPdT instrument loops and four OTdT instrument loops that are calibrated each time the surveillance test procedure is accomplished.

It should be pointed out that the specific setpoint has always been within the Trip Setpoint value column of Technical Specifications Table 2.2-1. The OPdT and OTdP instrument loops are very stable. The amount of drift noted during the monthly surveillance test has never been enough to require any adjustment. Therefore, the instruments were always considered operable. If a condition had occurred that required these instruments to trip, they would have done so within the values analyzed in the Accident Analysis. Therefore, there was no adverse safety significance as a result of this event. There was no damage to plant equipment or release of radioactivity as a result of this event. At no time did conditions develop that may have posed a threat to the health and safety of the public.

ACITION TAKEN

Following the original determination that an error existed, the most recently completed surveillance test procedure was examined to determine which instruments were left with nonconservative trip points. It was determined that two OPdT and three OTdT instrument loops were affected. New setpoints were then calculated and a temporary procedure was approved for resetting the bistable trip setpoint for each of these. The affected

| LICENSEE EVENT REPORT TEXT CONTINUATION | | APPROVED DMB ND. 315 EXPIRES: 4/30/92 ESTIMATED BURDEN PER RESPONSE T INFORMATION COLLECTION REDUEST COMMENTS REGARDING BURDEN ESTIM AND REPORTS MANAGEMENT GRANCH REGULATORY COMMISSION, WASHINGT THE FAPERWORK REDUCTION PROJEC OF MANAGEMENT AND BUDGET, WASHI | 192 E TO COMPLY WTH THIS ST. 50.0 HRS. FORWARD IMATE TO THE RECORDS OF DESCI U.S. NUCLEAR | | | |
|---|---|---|---|--|--|--|
| FACILITY NAME (1) | DOCKET NUMBER (2) | LER NUMBER (6) | PAGE (3) | | | |
| Wolf Creek Generating Station | 0 6 0 0 4 8 2 | VEAR GEQUENTIAL NUMBER REVISION NUMBER 8 8 0 0 7 0 1 | 0 5 0 0 5 | | | |
| bistable trip setpoints were All procedures that verified Table 2.2-1 were reviewed to setpoints. The surveillance occurrences. | the other setpoints ensure this error di | of Technical Specific | ations | | | |
| Based on the formal definition determined that this procedure to revise the procedure accor | al revision was unne | uracy, it has been cessary and it is int | ended | | | |
| | | | | | | |
| | | | | | | |
| | | | | | | |
| | | | | | | |
| | | | | | | |
| | | | | | | |
| | | | | | | |
| | | | | | | |
| | | | | | | |
| | | | | | | |

. .

.