

# THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

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Serving The Best Location in the Nation PERRY NUCLEAR POWER PLANT

V'CE PRESIDENT

Al Kaplan

February 28, 1990 PY-CEI/NRR-1129 L

U.S. Nuclear Regulatory Commission Document Control Desk Washington D.C. 20555

> Perry Nuclear Power Plant Docket No. 50-440 1989 Annual Report

Dear Sir:

Accached is the 1989 Annual Report for Perry Unit 1. This report is submitted in accordance with Technical Specifications 6.9.1.4 and 6.9.1.5 and fulfills ongoing commitments associated with License Commitment 17 of USAR Appendix 18.

If there are any questions, please feel free to call.

Very truly yours,

Al Kaplan Vice President Nuclear Group

Enclosure

AK:njc

cc: T. Colburn P. Hiland USNRC, Region III

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#### CLEVELAND ELECTRIC ILLUMINATING COMPANY

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PERRY NUCLEAR POWER PLANT

JANUARY 1 - DECEMBER 31, 1989

ANNUAL REPORT

DOCKET NUMBER: 50-440 LICENSE NUMBER: NPF-58

Submitted By: Steven - Kensicki Plant Technical Director

General Manager of Operations

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#### I. Occupational Radiation Summary Report

Enclosed in this section is the 1989 Annual Occupational Exposure Summary Report required by Technical Specification 6.9.1.5.a of Appendix A to the Perry Unit 1 Operating License. The dose assignments to various duty functions provided herein were obtained from pocket dosimeter measurements recorded in Radiation Work Permit (RWP) documents. Thermoluminescent dosimeter (TLD) measurements for 1989 are provided under a separate report in accordance with 10 CFR 20.407. The following work categories were grouped as indicated below:

STATION - Operations Department, Technical Department, and Nuclear Support Department

UTILITY - Nuclear Engineering Department and Quality Assurance Department

OTHER - Contractor, Consultant, Centerior, Davis-Besse, Eastern District, and other support personnel.

#### PERRY RUCIEAR POWER PLANY - Calif 1 1989 ANNUAL EXPOSURE REPORT REGREATORY GUIDE 1.16 APPENDIX A SUMMARY

		# OF PERSONS > 100 MREM		TOTAL MAN-REM			
WORK CATEGORY	JOE CATEGORY	STATION	UTILITY	OTHER	STATION	JILITY	OTHER
Reactor Oper &	Maintenance	130	4	275	10.40°	0.520	5,909
Surveillance	Operations	79	1	5	28.41"	0.000	0.745
	Health Physics	48	0	172	22.514	0.000	57.464
	Supervisory	0	0	0	0.000	0.000	0.000
	Engineering	27	22	41	1.024	0.500	0.933
Routine	Maintenance	151	6	741	10.377	0.197	38.583
Maintenance	Operations	74	1	7	9.496	0.295	9.635
	Health Physics	45	0	186	2.559	0.000	19.604
	Supervisory	0	0	0	0.000	0.000	0.000
	Engineering	28	25	62	3.462	2.670	3.306
Inservice	Maintenance	100	2	592	4.780	0.070	66.124
Inspection	Operations	46	1	4	0.645	0.015	0.070
	Health Physics	18	0	102	0.360	0.000	2.637
	Supervisory	0	0	0	0.000	0.000	0.000
	Engineering	22	23	65	3.220	1.850	18.950
Special	Maintenance	151	6	768	43.489	1.540	368.653
Maintenance	Operations	70	1	10	5.461	0.025	2.625
	Health Physics	47	0	187	10.322	0.000	20.849
	Supervisory	0	0	0	0.000	0.000	0.000
	Engineering	28	25	57	3.238	5.596	9.267
Waste	Maintenance	45	0	216	0.680	0.000	11.098
Processing	Operations	22	0	3	2.220	0.000	1.380
	Health Physics	32	0	123	3.470	0.000	7.533
	Supervisory	0	0	0	0.000	0.000	0.000
	Engineering	0	5	3	0.000	0.045	0.000
Refueling	Maintenance	70	2	402	2.600	0.015	37.598
	Operations	35	1	4	0.480	0.000	2.825
	Health Physics	20	0	91	0.570	0.000	7.565
	Supervisory	0	0	0	0.000	0.000	0.000
	Engineering	15	11	23	1.260	0.520	0.770
Totale	Waintenance	617	20	2044	77 335		527 A/F
Iocars	Operations	326	5	33	46 715	0 235	\$ 280
	Health Dhursies	210	ñ	\$61	10.715	0.000	115 652
	Supervisory		0	000	0.000	0.000	0.000
	Engineering	120	111	246	12 204	11 101	22.226
	angineering	120		240	12.204	11.101	33.226
Grand Totals		1303	136	4084	171 049	13.050	485 133

### II. Safety/Relief Valve Challenges

The attached report includes all challenges made to the Safety/Relief Valves (SRVs) at the Perry Nuclear Power Flant in 1989. This information is submitted in accordance with Technical Specification 6.9.1.5.b of Appendix A to Operating License NPF-58 for Perry Nuclear Power Plant Unit 1. All nineteen safety/relief valves were replaced during the first refuel outage (February 22, through August 5, 1989). Following SRV replacement, all nineteen SRVs Jere stroked twice for initial operability in accordance with Technical Specification 4.0.5. Events subsequent to July 28, 1989, on the attached list were planned, manual actuations made to reseat the valves to minimize weepage. These actuations were performed in accordance with the surveillance test instruction, SVI-B21-T2005, "SRV Exercise Test."

Included as a SRV "challenge" are the following actions/events:

- -1- the SRV being physically operated by the manual control switches in the control room.
- -2- the SRV automatically lifts per design pressure limit when installed in the plant.

VALVE	DATE	TIME
1B21-F041A	7-28-89	0927
1821-F0418	7-28-80	1043
	/-20-09	1044
1B21-F041C	7-28-89	0929
1B21-F041D	7-28-89	1045 0930
		1046
1B21-F041E	7-28-89	0931
1B21-F041F	7-28-89	1047
	1-20-07	1048
1B21-F041G	7-28-90	0933
		1049
1 <b>B</b> 21-F041K	7-28-89	0934
1821-F047D	7 20 00	1050
1021-104/0	1-20-09	1040
1B21-F047F	7-28-89	0938
		1039
1B21-F047G	7-28-89	0939
1821 80470	7 00 00	1038
1021-FU4/H	7-28-89	0940
1B21-F051A	7-28-89	1037
	1-20-07	1036
1B21-F047B	7-28-89	0935
		1042
1B21-F047C	7-28-89	0936
1821_R0518	7 20 00	1041
1021-10510	7-20-09	1035
1B21-F051C	7-28-89	0943
		1034
1B21-F051D	7-28-89	0944
1001 00510	7 00 00	1033
1621-F0516	7-28-89	0945
1B21-F051G	7-31-89	1032
1B21-F047D	7-31-89	0512
1B21-F041G	7-31-89	0514
		0807
1B32-F041A	7-31-89	0519
1B21-F041F	8-09-89	1709
1B21-F051G	8-09-89	1706

# 1989 PERRY UNIT 1 SAPETY/RELIEF VALVE ACTUATION EVENTS

#### III. Reactor Coolant System Specific Activity Analysis

2. A.C. 4 4

During 1989 there were no occurrences in which Technical Specification 3.4.5, "Specific Activity," Limiting Condition for Operation was exceeded. This information is submitted in accordance with Technical Specification 6.9.1.5 c.

#### IV. Silicone Sealant Inspections

In accordance with commitments made in response to License Commitment 17 of USAR Appendix 1B (reference letter PY-CEI/NRR-0703L, dated August 27, 1987 from Murray R. Edelman) the following summary report is provided. During Perry Unit 1 first refueling outage (February 22-August 5, 1989) duct specimens were placed in an environmentally representative horizontal pipe chase in the Intermediate Building, 585' elevation. The duct specimens were leak tested, with acceptable results<sup>\*</sup>. An engineering walkdown of representative portions of the Annulus Exhaust Gas Treatment (M15), Control Room HVAC (M25) and Fuel Handling Building Ventilation (M40) systems was conducted and the exposed exterior silicone sealant was inspected, also with acceptable results.

\* These duct specimens were already aged to 40 years prior to their placement into the plant, therefore the ongoing leak tests only determine how long the sealant is good for past the 40 years qualified life, and to provide a means for recognizing degradation as it occurs. Since the sealant is already qualified for 40 years, failure of these test specimens during any future testing will not invalidate the results of the qualification report.

#### NJC/CODED/V3113

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