

SACRAMENTO MUNICIPAL UTILITY DISTRICT . P. O. Box 15830, Sacramento CA 95852-1830, (916) 452-3211 AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

NL 90-035

February 14, 1990

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

Docket No. 50-312 Rancho Seco Nuclear Generating Station License No. DPR-54 OPERATING PLANT STATUS REPORT

Attention: George Knighton

Enclosed is the January 1990 Monthly Operating Plant Status Report for the Rancho Seco Nuclear Generating Station. The District submits this report pursuant to Technical Specification 6.9.3.

Sincerely,

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Steve L. Crunk Nuclear Licensing Manager

Encl (5)

J. B. Martin, NRC, Walnut Creek cc w/encl: A. D'Angelo, NRC, Rancho Seco INPO R. Twilley, Jr.



RANCHO SECO NUCLEAR GENERATING STATION I 14440 Twin Cities Road, Herald, CA 95638-9799; (209) 333-2935

SUMMARY OF PLANT OPERATIONS

The plant has been in a defueled condition since the last of the fuel was removed and placed in the Spent Fuel Pool at approximately 1030 hours on Friday, December 8, 1989. The reactor vessel head was placed back on the reactor vessel on January 19. Installation and tensioning of the head closure studs was completed on January 30, 1990.

SUMMARY OF CHANGES IN ACCORDANCE WITH 10 CFR 50.59

The plant staff accepted documentation packages in January 1990 for the facility changes, procedure changes and tests described below which required detailed safety analyses. These changes were reviewed in accordance with the Technical Specifications by the Plant Review Committee (PRC) and the Management Safety Review Committee (MSRC).

Nuclear Training Area Administration Procedure TDAP-1010, Revision

 Nuclear Training Area Organization, eliminates the requirement
 for INPO accreditation for all training programs, but requires INPO
 Standards be maintained for RO and SRO Training and Requalification
 Programs. The Systematic Approach to Training continues to be the
 method used for development and implementation of all training
 programs. TDAP-1010 also reflects the training organization per
 Rancho Seco Administrative Procedure RSAP-0101, Nuclear Organization
 Responsibilities and Authorities.

INPO accreditation of the training program is administrative and not considered in USAR Chapter 14 evaluation of accidents. A regulatory based training program in accordance with 10 CFR 55 and the Rancho Seco Technical Specifications will be maintained. This change to training requirements does not increase the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the USAR, nor is the possibility for an accident or malfunction of a different type than any evaluated previously in the USAR created. This change does not involve an Unreviewed Safety Ouestion.

2. Special Test Procedure STP.1170, Special Test of the Bruce GM Diesel Generator Exciter Controls, was performed to demonstrate that both Bruce GM Diesel Generators will build up voltage and frequency to 3740-4580 Volts and 58.8-61.2 Hz, respectively, within 10 seconds after receipt of an emergency start signal while in the idle speed cooldown mode. All acceptance criteria were met and there were no outstanding test deficiencies.

STP.1170 required only one diesel generator train be out of service for testing. The test was performed with one train isolated and unloaded. After completion of the test, the system was established to its original configuration and SP.56A(B), Variable Diesel SUMMARY OF CHANGES IN ACCORDANCE WITH 10 CFR 50.59 (Continued)

Generator (G-886A[B]) Synchronization Surveillance Test, performed to verify system operability. During the test the second train was still capable of supplying sufficient load capabilities as described in the USAR. Thus, this test did not increase the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the USAR, nor was the possibility for an accident or malfunction of a different type than any evaluated previously in the USAR created. STP.1170 did not involve an Unreviewed Safety Question.

- The Auxiliary Fuel Handling Bridge (AFHB) was dismantled and the 3. parts moved to the Interim Onsite Storage Building for storage pending future disposal per DCP R89-0085. The ANHB is not associated with any of the fuel handling accidents evaluated in the USAR. Further, there is no way elimination of the AFHB could contribute to the consequences of any accident previously evaluated. Elimination of the AFHB does not create any new equipment failure modes and failure modes of the remaining Fuel Handling System equipment remains unchanged after removal of the AFHB. Therefore, this change does not increase the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the USAR, nor is the possibility for an accident or malfunction of a different type than any evaluated previously in the USAR created. This change does not involve an Unreviewed Safety Question.
- Physical Security Plan, Amendment 17, reflects the defueled 4. condition of the reactor and the fact that all nuclear fuel is now in the spent fuel storage pool. It also reflects the organization per Rancho Seco Administrative Procedure RSAP-0101, Nuclear Organization Responsibilities and Authorities. Amendment 17 pertains to the definition of Vital Areas and their protection in a defueled plant condition. Security protection shall be provided to those systems required to support the storage of spent fuel as though they were still vital. This administrative change to the Physical Security Plan does not increase the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the USAR, nor is the possibility for an accident or malfunction of a different type than any evaluated previously in the USAR created. This change does not involve an Unreviewed Safety Question.

MAJOR SAFETY-RELATED MAINTENANCE, TESTS AND MODIFICATIONS NOT REQUIRING DETAILED SAFETY ANALYSES

- Routine maintenance was performed on the "B" TDI diesel generator during January 1990.
- Special Test Procedure STP.1308 was conducted to estimate the corrosion activity of the Spent Fuel Building concrete reinforcing steel using the nondestructive Half-Cell Potential method. The acceptance criteria that measured half cell potentials are more positive than -0.20 volts was satisfied.

REFUELING INFORMATION REQUEST

1.	Name of Facility Rancho Seco					
2.	Scheduled date for next refueling shutdown: *					
з.	Scheduled date for restart following refueling: *					
4.	Technical Specification change or other license amendment required:					
5.	Scheduled date(s) for submitting proposed licensing action: *					
6.	Important licensing considerations associated with refueling: *					
7.	Number of fuel assemblies:					
	a) In the core:O					
	b) In the Spent Fuel Pool:493					
8.	Present licensed spent fuel capacity:1080					
9.	Projected date of the last refueling that can be discharged to					
	the Spent Fuel Pool: December 3, 2001					

* Plant shut down June 7 following negative outcome of public vote regarding continued operation of Rancho Seco by SMUD.

AVERAGE DAILY UNIT POWER LEVEL

	DOCKET NO.	50-312
	UNIT	Rancho Seco
	DATE	1/31/90
	COMPLETED BY	Marla Mueller
	TELEPHONE	(916) 452-3211
January 1990		
AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY AVERAG	E DAILY POWER LEVEL (MWe-Net)
0	17	0
0	18	0
0	19	0
0	20	
0	21	00
0	22	0
0	23	0
0	24	0
0	25	0
0	26	0
0	27	0
0	28	0
0	29	0

15 <u> 0</u> 16 <u> 0</u>

INSTRUCTIONS

MONTH

DAY

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. OPERATING DATA REPORT

DOCKET NO	50-312
DATE _	1/31/90
COMPLETED BY _	Marla Mueller
TELEPHONE _	(916) 452-3211

OPERATING STATUS

Unit Name: Ran Reporting Period: Janu Licensed Thermal Power (MWt): Nameplate Rating (Gross MWe): Design Electrical Rating (Net MWe): Maximum Dependable Capacity (Gross MWe): Maximum Dependable Capacity (Net MWe): If changes Occur in Capacity Ratings (It Give Reasons: Dever Level to Which Postricted If Any	cho Seco ary 1990 2.772 963 918 917 873 ems Number 3 Th (Not MWo):	Notes: hrough 7) Since La N/A	st Report,
Reasons For Restrictions, If Any:	N/A	N/A	
	This Month	Yr-to-Date	Cumulative
Hours in Reporting Period	744	744	129,672
Number of Hours Reactor Was Critical	0	0	62,221.5
Reactor Reserve Shutdown Hours	00	00	12,736.6
Hours Generator On-Line	0	0	57,811.1
Unit Reserve Shutdown Hours	0	0	3,647.5
Gross Thermal Energy Generated (MWH)	0	0	141,951,95.
Gross Electrical Energy Generated (MWH)	2 005	2 005	40,223,924
Net Electrical Energy Generated (MWH)	-3,905	-3,905	42,404,00
Unit Augilability Factor	0%	0%	A7 6°
Unit Canacity Factor (Using MDC Net)	0%	0%	37 5
Unit Capacity Factor (Using DER Net)	0%	0%	35.7
Unit Forced Outage Rate	0%	0%	42.7
Shutdowns Scheduled Over Next 6 Months (If Shut Down At End Of Report Period, Es Units In Test Status (Prior to Commercia	Type, Date, and timated Date of 1 Operation):	d Duration of Each f Startup: * Forecast): * Achie
INITIAL C INITIAL E	RITICALITY	<u>N/A</u> N/A	N/A
COMMERCIA	L OPERATION	N/A	N//

* Plant shut down June 7 following negative outcome of public vote regarding continued operation of Rancho Seco by SMUD.

UNIT SHUTDOWNS AND POWER REDUCTIONS

BOCKET NO.	50-312 Rancho Seco		
UNIT NAME			
DATE	1-31-90		
COMPLETED BY	M. Mueller		
TELEPHONE	(916) 452-3211		

REPORT MONTH January 1990

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code 4	Component Code 5	- Cause & Corrective Action to Prevent Recurrence
1	06-07-89	S	744	F	1	N/A	N/A	N/A	Plant shut down June 7, 1989 following negative outcome of public vote regarding continued operation of Rancho Seco by SMUD.
l F S	Forced Scheduled	2 Rea A- B- C- D- F-	son: Equipment F Maintenance Refueling Regulatory Operator Ir	ailure (Ex or Test Restrictio	plain) n icense Exami	3 nation	Method: 1- Manual 2- Manual 3- Automat 4- Other (Scram ic Scram Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

F- Administrative

G- Operational Error (Explain)

H- Other (Explain)

5 Exhibit 1 - Same Source