System Energy

System Energy
P.O. Box 756
Port Gibson, MS 39150
Tel 601 437 6470

John G. Cesare, Jr.
Director
Nuclear Licensing

February 14, 1990

U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

Attention: Document Control Desk

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-29
Monthly Operating Report
AECM-90/0037

In accordance with the requirements of Technical Specification 6.9.1.10, System Energy Resources, Inc. (SERI) is providing the Monthly Operating Report for Grand Gulf Nuclear Station Unit 1 for January ~~1989~~ 90 (attachment).

If you have any questions or require additional information, please contact this office.

JGC:cwg
Attachment

cc: Mr. D. C. Hintz (w/a)
Mr. T. H. Cloninger (w/a)
Mr. R. B. McGehee (w/a)
Mr. N. S. Reynolds (w/a)
Mr. H. L. Thomas (w/o)
Mr. H. O. Christensen (w/a)

Mr. Stewart D. Ebneter (w/a)
Regional Administrator
U.S. Nuclear Regulatory Commission
Region II
101 Marietta St., N.W., Suite 2900
Atlanta, Georgia 30323

Mr. L. L. Kintner, Project Manager (w/a)
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Mail Stop 14B20
Washington, D.C. 20555

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DOCKET NO. 50-416
DATE 02/14/90
COMPLETED BY G. A. Zinke
TELEPHONE (601)437-2459

OPERATING STATUS

1. Unit Name: GGNS UNIT 1
2. Reporting Period: January 1990
3. Licensed Thermal Power (MWt): 3833 MWt
4. Nameplate Rating (Gross MWe): 1372.5 MWe
5. Design Electrical Rating (Net MWe): 1250 MWe
6. Maximum Dependable Capacity (Gross MWe): 1190 MWe
7. Maximum Dependable Capacity (Net MWe): 1142 MWe

Notes:

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:
   N/A
9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr to Date	Cumulative
11. Hours In Reporting Period	744	744	46,306.0
12. Number of Hours Reactor Was Critical	744	744	35,799.7
13. Reactor Reserve Shutdown Hours	-0-	0-	-0-
14. Hours Generator On-Line	744	744	33,991.8
15. Unit Reserve Shutdown Hours	-0-	-0 -	-0-
16. Gross Thermal Energy Generated (MWH)	2,768,627	2,768,627	114,334,205
17. Gross Electrical Energy Generated (MWH)	906,280	906,280	36,230,240
18. Net Electrical Energy Generated (MWH)	871,346	871,346	34,609,339
19. Unit Service Factor	100.0	100.0	76.9
20. Unit Availability Factor	100.0	100.0	76.9
21. Unit Capacity Factor (Using MDC Net)	102.6	102.6	72.2
22. Unit Capacity Factor (Using DER Net)	93.7	93.7	65.2
23. Unit Forced Outage Rate	0	0	5.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    None
25. If Shut Down At End Of Report Period. Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation).

	Forecast	Achieved
INITIAL CRITICALITY		08/18/82
INITIAL ELECTRICITY		10/20/84
COMMERCIAL OPERATION		07/01/85

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MONTH January 1990

DAY	AVERAGE DAILY POWER LEVEL (MW$_e$-Net)	DAY	AVERAGE DAILY POWER LEVEL (MW$_e$-Net)
1	600.5	17	1190.2
2	1050.1	18	1197.0
3	966.5	19	1189.0
4	1195.2	20	1197.3
5	1220.5	21	1213.3
6	1220.0	22	1211.7
7	1213.4	23	1203.3
8	1218.5	24	1175.5
9	1183.5	25	1211.3
10	1208.7	26	1213.7
11	1203.6	27	1191.0
12	1218.6	28	1197.8
13	1204.0	29	1212.5
14	1213.8	30	1207.6
15	1178.0	31	1203.3
16	1196.8		

Attachment III

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-416
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TELEPHONE 601-437-2459

REPORT MONTH January 1990

No.	Date	Type (1)	Duration Hours	Reason (2)	Method of Shutting Down R	Licensee Event Report #	System Code (4)	Component Code (5)	Cause & Corrective Action To Prevent Recurrence
None									

1 2 3 4

F: Forced
S: Scheduled

Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Examinaticn
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continued
5-Reduced load
6-Other

5
Exhibit 1-Same Source

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

MAIN STEAM SAFETY RELIEF VALVE CHALLENGES

DOCKET NO. 50-416
UNIT 1
COMPLETED BY G. A. Zinke
TELEPHONE (601)437-2459

Date of Occurrence: January 15, 1990

Plant Operating Condition:

Rx Thermal Power 100% Rx Pressure (psig) 1025 Rx Mode 1

Rx Power (MWE) 1265 Rx Temperatures 532°F

Number of mainsteam line SRV's: 5

Number os SRV's affected by event: 5

Narrative:

On 01/15/90, five Main Steam Line SRVs automatically lifted due to a spurious low-low set actuation. Reactor power was reduced from 100 percent to approximately 93 percent to prevent a possible reactor scram on high neutron flux or high reactor pressure as the SRVs were manually closed. The cause of actuation is under investigation (IR90-01-6).

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