



EDISON PLAZA
300 MADISON AVENUE
TOLEDO, OHIO 43652-0001

December 11, 1989
KB89-00589

Docket No. 50-346
License No. NPF-3

Document Control Desk
U. S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20555

Gentlemen:

Monthly Operating Report, November 1989
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit No. 1 for the month of November 1989.

If you have any questions, please contact Bilal Sarsour at (419) 321-7384.

Very truly yours,

A handwritten signature in cursive script that reads 'Louis F. Storz'.

Louis F. Storz
Plant Manager
Davis-Besse Nuclear Power Station

BMS/mjb

Enclosures

cc: Mr. A. Bert Davis
Regional Administrator, Region III

Mr. Paul Byron
NRC Resident Inspector

Mr. T. V. Wambach
NRC Senior Project Manager

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346

UNIT Davis-Besse #1

DATE December 11, 1989

COMPLETED BY Bilal Sarsour

TELEPHONE (419) 321-7384

MONTH November 1989

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>882</u>	17	<u>886</u>
2	<u>884</u>	18	<u>885</u>
3	<u>885</u>	19	<u>864</u>
4	<u>885</u>	20	<u>884</u>
5	<u>878</u>	21	<u>885</u>
6	<u>878</u>	22	<u>886</u>
7	<u>876</u>	23	<u>885</u>
8	<u>877</u>	24	<u>886</u>
9	<u>877</u>	25	<u>883</u>
10	<u>882</u>	26	<u>882</u>
11	<u>882</u>	27	<u>881</u>
12	<u>882</u>	28	<u>881</u>
13	<u>878</u>	29	<u>884</u>
14	<u>879</u>	30	<u>884</u>
15	<u>877</u>	31	<u> </u>
16	<u>884</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-346
 DATE December 11, 1989
 COMPLETED BY Bilal Sarsour
 TELEPHONE (419) 321-7384

OPERATING STATUS

1. Unit Name: Davis-Besse Unit #1
2. Reporting Period: November 1989
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	8,016.0	99,385.0
12. Number Of Hours Reactor Was Critical	720.0	7,803.1	53,410.6
13. Reactor Reserve Shutdown Hours	0.0	89.0	5,393.7
14. Hours Generator On-Line	720.0	7,762.6	51,456.4
15. Unit Reserve Shutdown Hours	0.0	0.0	1,732.5
16. Gross Thermal Energy Generated (MWH)	1,990,518	21,038,816	122,908,205
17. Gross Electrical Energy Generated (MWH)	666,574	7,019,016	40,660,200
18. Net Electrical Energy Generated (MWH)	634,586	6,665,326	38,130,378
19. Unit Service Factor	100.0	96.8	51.8
20. Unit Availability Factor	100.0	96.8	53.5
21. Unit Capacity Factor (Using MDC Net)	100.8	95.1	43.9
22. Unit Capacity Factor (Using DER Net)	97.3	91.8	42.3
23. Unit Forced Outage Rate	0.0	2.0	29.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling - Start February 1, 1990 - 18 weeks - End June 5, 1990

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATIONAL SUMMARY

November 1989

Reactor power was maintained at approximately 100% full power until 2358 hours on November 18, 1989, when a manual power reduction to approximately 85% was initiated to perform turbine control valve testing.

After the completion of turbine control valve testing, reactor power was slowly increased to approximately 100% full power at 0757 hours on November 19, 1989, and maintained at this power level for the rest of the month.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346

UNIT NAME Davis-Besse #1

DATE December 11, 1989

COMPLETED BY Bilal Sarsour

TELEPHONE (419) 321-7384

REPORT MONTH November 1989

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
<div style="border: 1px solid black; padding: 10px; width: fit-content; margin: auto;"> NO SIGNIFICANT UNIT SHUTDOWNS OR POWER REDUCTIONS </div>									

¹F: Forced
S: Scheduled

²Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation from Previous Month
5-Load Reduction
9-Other (Explain)

⁴Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit I - Same Source
*Report challenges to Power Operated Relief Valves (PORVs) and Pressurizer Code Safety Valves (PCSVs)

REFUELING INFORMATION

Date: November 1989

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling outage? February 1990
3. Scheduled date for restart from current refueling: N/A
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: A license amendment request to remove cycle-specific values from Section 3 of the Technical Specifications was submitted to the NRC on June 16, 1989, based on Generic Letter 88-16. Assuming approval of this submittal in December 1989, no Cycle 7 Technical Specification changes are expected for the Technical Specifications though Bases changes will likely result.

5. Scheduled date(s) for submitting proposed licensing action and supporting information: For Bases, January, 1990.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
 - a. Sixty Batch Reload 3.38% enriched.
 - b. New fuel design Mark B8A (Reconstitutable, removable upper end fitting, Zircaloy grid spacer, debris resistant lower end cap, lower prepressurization, and annealed guide tubes.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool, and (c) the new fuel storage areas.

(a) 177 (b) 268 (c) 0
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735 Increased size by: approximately 900 by 1994 is planned
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1996 - assuming ability to unload the entire core into the spent fuel pool is maintained