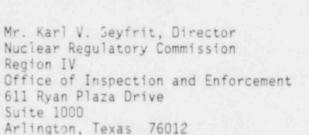
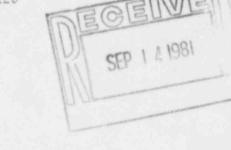


Public Service Company of Colorado

September 11, 1981 Fort St. Vrain Unit No. 1

P-81226





Reference: Facility Operating License

No. DPR-34

Docket No. 50-267

Dear Mr. Seyfrit:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/81-050, Final, submitted per the requirements of Technical Specification AC 7.5.2(b)2.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/81-050.

Very truly yours,

Don Warembourg Manager, Nuclear Production

Da Warentonny

DW/cls

Enclosure

cc: Director, MIPC

IEDA

OCCURRENCE REPORT DISTRIBUTION

OCCURRENCE REPORT DISTRIBUTION			
Number	Number of Copies		
Department of Energy	1	(P Letter)	
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Mail Station 8-107 Washington, D.C. 20545 Department of Energy	1	(P Letter)	
110 West A Street, Suite 460 San Diego, California 92101 Mr. Karl V. Seyfrit, Director	(1)	(Original of P Letter and Copy of LER)	
Nuclear Regulatory Commission 611 Ryan Plaza Drive Suite 1300 Arlington, Texas 76012			
Director Office of Management Information and Progress Control U. S. Nuclear Regulatory Commission Washington, D.C. 20555	1	(P Letter, LER)	
Mr. Richard Phelps	10	(Original of FPLG Letter plus Two Copies, One Copy of P Letter, One Copy of LER)	
Mr. 8ill Lavaiee	1	(P Letter, LER)	

1 (P Letter, LER)

NRC Resident Site Inspector -

REPORT DATE:

September 11, 1981

REPORTABLE OCCURRENCE 81-050

OCCURRENCE DATE: August 14, 1981

Determined

ISSUE 0 Page 1 of 2

FORT ST. VRAIN NUCLEAR GENERATING STATION PUBLIC SERVICE COMPANY OF COLORADO 16805 WELD COUNTY ROAD 19 1/2 PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/81-050/03-L-0

Final

IDENTIFICATION OF OCCURRENCE:

On August 14, 1981, it was determined that the total dissolved solids in the plant nonradioactive liquid effluent exceeded 2300 parts per million on August 9, 1981. This is reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)2 as a degraded mode of LCO NR 1.1.

EVENT DESCRIPTION:

The reactor power level was increased from 45% starting August 3, 1981, and reached approximately 70% on August 6, 1981. The increase in power level was responsible for increased evaporation of the circulating water resulting in increased solids in the circulating water tower basin and higher total dissolved solids in the blowdown water. The increase was monitored by the Chemistry Department, but the increase on midnight shift on August 9, 1981, resulted in the limit of 2300 parts per million being exceeded.

The corrective action taken was to reduce cold side blowdown and increase raw water blowdown, which brought the total dissolved solids within limits by the next shift's sample.

CAUSE DESCRIPTION:

The increased total dissolved solids in the plant nonradioactive effluent was due to increased evaporation caused by reactor power level increases and improper blowdown adjustment.

CORRECTIVE ACTION:

The main gooling water blowdown was adjusted to reflect the plant power levels.

No further corrective action is anticipated or required.

REPORTABLE OCCURRENCE 81-050 ISSUE 0 Page 2 of 2

Prepared By:

Cua B. Reed

Asa B. Reed

Technical Services Technician

Reviewed By:

Technical Services Manager

Reviewed By:

Edwin b. Hill Station Manager

Approved By: Un Warenburg

Manager, Nuclear Production