NRC FOR	M 366	U. S. NUCLEAR REGULATORY COMMISSION
	LICENSEE EVENT REPORT	Update Report - Previous Report
		E ALL REQUIRED INFORMATION
0 1 7 8	$ \begin{array}{ c c c c c } \hline N & C & B & E & F & 1 \\ \hline 9 & \text{LICENSEE CODE} \end{array} \begin{array}{ c c c c } \hline 14 & \hline 15 & \text{LICENSE NUMBER} \end{array} \begin{array}{ c c } \hline 0 & 0 & 0 & 0 & 0 \\ \hline 14 & \hline 15 & \text{LICENSE NUMBER} \end{array} \begin{array}{ c c } \hline 0 & 0 & 0 & 0 \\ \hline 25 & \hline 26 & \hline 26$	4 1 1 1 1 1 4 6 57 CAY 58 5
	REPORT LL 6 0 5 0 - 0 3 2 5 0 0 1 2 0 8 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE	1 8 1 8 1 4 8 1 9 74 75 REPORT DATE 80
0 2	During normal plant operation, reactor instrument penetrat	tion (RIP) valve, X53C,
03	shut, isolating the variable leg to level control instrumer	nts B21-LT-N004A&C, RPS
04	level instruments B21-L1S-N017A&B, and remote shutdown par	nel instrument B21-LT-3331.
0 5	This event caused a reactor scram on low level No. 1 due t	to feed flow fluctuations.
06	[This event did not affect the health and safety of the pub	blic.
0 7	Technical Specifications 3.3.1, 6.9.1.9b	
08	L	80
09 7 8	$\begin{array}{c c} S & S \\ 9 & 10 \end{array} \xrightarrow{\text{CAUSE}} & \begin{array}{c} CAUSE \\ S & DL \\ 9 & 10 \end{array} \xrightarrow{\text{CAUSE}} & \begin{array}{c} CAUSE \\ S & DL \\ 11 \end{array} \xrightarrow{\text{CAUSE}} & \begin{array}{c} CAUSE \\ S & DL \\ 12 \end{array} \xrightarrow{\text{CAUSE}} & \begin{array}{c} CAUSE \\ S & DL \\ 12 \end{array} \xrightarrow{\text{CAUSE}} & \begin{array}{c} CAUSE \\ COMPONENT CODE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \\ COMPONENT CODE \end{array} \xrightarrow{\text{CAUSE}} & \begin{array}{c} CAUSE \\ S & DL \\ 12 \end{array} \xrightarrow{\text{CAUSE}} & \begin{array}{c} CAUSE \\ COMPONENT CODE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \\ COMPONENT CODE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \\ S & DL \\ 12 \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \\ COMPONENT CODE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \\ COMPONENT CODE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \\ CODE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \\ COMPONENT CODE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \\ COMPONENT CODE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \\ CODE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}{c} CAUSE \end{array} \xrightarrow{\text{COMPONENT CODE}} & \begin{array}$	SUBCODE SUBCODE H 15 D (16) 20 BEVISION
	17 LER/RO REPORT EVENT YEAR REPORT NO. CODE 17 NUMBER 8 1 0 1 6 0 0 3 23 24 26 27 28 29	HEPOHI TYPE NO. L 31 L 32 PRD4 PRIME COMP COMPONENT
	TAKEN ACTION ON PLANT METHOD HOURS (22) SUBMITTED FOR X (B) Z (9) A (20) C (2) 0 (1 8) Y (2) 435 (2) 36 (2) 37 (2) 40 (2) $4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(2) 40 (2) 4(3) 40 (2) 4(3) 40 (2) 4(4) 40 (2) 40 (2) 40 (2) 40 (2) 40 (2) 40 (2)$	$\begin{array}{c ccccccccccccccccccccccccccccccccccc$
10	An exhaustive investigation failed to reveal a definite ca	ause for the RIP valve closure.
11	This investigation included a leak check on the valve cont	trol air supply, a timed
12	leak check of the valve bellows and a visual check of the	valve and the valve high
1 3	flow isolation switch. This is considered an isolated eve	ent, as system air pressure
14	was normal and no problems could be found with the valve.	80
7 8	$ \begin{array}{c c} 9 \\ FACILITY \\ STATUS \\ E \\ \hline 28 \\ 10 \\ 10 \\ 10 \\ 10 \\ 10 \\ 10 \\ 10 \\ 1$	DISCOVERY DESCRIPTION 32
1 6 7 8	CTIVITY CONTENT ELEASED OF RELEASE AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE 36
17 78	PERSONNEL EXPOSURES NUMBER 0 0 0 0 37 Z 38 NA	80
1		
7 8	9 11 12 LOSS OF OR DAMAGE TO FACILITY (43)	80
19	LZ 42 NA	80
20	PUBLICITY ISSUED DESCRIPTION (45)	NRC USE ONLY
B108240095 B10814 80.5 PDR ADDCK 05000325 ARER R. M. Poulk, Jr. PHONE 919-457-9521 0 S PDR		

LER 1-81-16 ATTACHMENT

Facility: Unit No. 1

Date: 1-20-81

The closure of α IP X53C isolated the selected feedwater level control input instrument N004A, creating a level error signal in the feed pump control circuitry. The nature of the initial error (decreasing level) caused the feed pumps to increase speed to restore level to the normal band (32-42"). This increase in feed flow actually caused vessel level to exceed the normal band and increase to approximately 60 inches. At this time, the isolated N004A instrument began indicating an increasing vessel level and thus decreased feed pump speed, thereby decreasing vessel level. Vessel level decreased to approximately 12.5 inches, where a low level scram was initiated from level instruments N 7C & D. At no time during this event did the isolated level instrument N004A incurate out of the normal operating band. A normal scram recovery was then conducted.

This event was reviewed by the Plant Nuclear Safety Committee. As no reason could be determined for the RIP valve closure and testing after the scram indicated no problems; therefore, no follow-up action was deemed necessary for the valve. To assist the operator in identifying when a level instrument important to plant safety and/or operation is affected by the operability or position of its RIP valve, selected RIP valve indication/control modules have been color coded to allow easy and rapid identification.

This event was also reviewed to determine if our level instrumentation complied with IEEE 279-1971, Section 4.7.3. This review concluded that the installed equipment was in compliance with this document. This review and its conclusions were discussed in detail with NRC personnel from both IE and ONRR.