U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 LICENSEE CODE: G A E I H 1 (2) LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3) LICENSE TYPE: 4 1 1 1 1 (4) CAT (5)

CON'T 01 REPORT SOURCE: L (6) DOCKET NUMBER: 0 5 0 0 0 3 2 1 (7) EVENT DATE: 0 7 2 9 8 1 (8) REPORT DATE: 0 8 0 6 8 1 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

On 7-29-81 with Unit 1 in steady state operation at 99% power it was determined by the AE, Bechtel, that the Unit 1 RHRSW piping in the intake structure was inadequately supported. The RHRSW system was therefore in a degraded condition violating Tech Specs 6.9.1.8.i. Unit 2 RHRSW is adequately supported. There was no significant occurrence due to this event. The health and safety of the public were not affected by this event.

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
C F (11)	B (12)	A (13)	Z Z Z Z Z Z (14)	Z (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
8 1	—	0 8 1	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
F (18)	F (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	Z (25)	Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The SCSI design for the Unit 1 RHRSW intake structure piping was inadequate resulting in a low (1.09) factor of safety in the worst postulated case. Additional supports have been designed and are being installed.

FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
E (28)	0 9 9 (29)	NA	D (31)	Notification from AE

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION (39)
0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

NUMBER	DESCRIPTION (41)
0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

TYPE	DESCRIPTION
Z (42)	NA

PUBLICITY

ISSUED	DESCRIPTION (45)
N (44)	NA

NRC USE ONLY

NAME OF PREPARER C. L. Coggin, Supt. Plt. Eng. Serv. PHONE: 912-367-7851

LER #: 50-321/1981-081
Licensee: Georgia Power Company
Facility Name: Edwin I. Hatch
Docket #: 50-321

Narrative Report
for LER 50-321/1981-081

On 7-29-81, with Hatch Unit 1 in steady state operations at 2414 megawatts thermal, it was determined by the AE, Bechtel, that the Unit 1 residual heat removal service water piping in the intake structure was inadequately supported in the horizontal plane in the vicinity of the RHRSW strainers. A computer analysis of the data obtained during a system walkdown conducted the previous week indicated that the worst case of combined pressure, weight, and DBE seismic stresses resulted in a low safety factor (1.09 based on actual material yield stress). Modifications to the Unit 1 RHRSW intake structure piping are being performed under PDCR B2M-165 and DCR 81-50 for both divisions I and II of Unit 1 RHRSW. Unit 2 RHRSW intake structure piping has been determined to be adequately supported.

There was no significant occurrence due to this event; the health and safety of the public were not affected by th event.