



Energy Measurements Group  
San Ramon Operations

EGG 1183-4219

MAY 1981

**TECHNICAL EVALUATION OF THE LICENSEE'S RESPONSE  
TO I&E BULLETIN 80-06  
CONCERNING ESF RESET CONTROLS FOR THE  
MILLSTONE NUCLEAR POWER STATION, UNIT 1**

(BUCKET NO. 80-245)

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REVISIONS

# INTERIM REPORT



NRC TAC No. 42755  
Report No. EGG 1183-4219

**Contract Program or Project Title:**

Electrical, Instrumentation, and Control System Support

**Subject of this Document:**

Technical Evaluation of the Licensee's Response to I&E Bulletin 80-06 Concerning ESF Reset Controls for the Millstone Nuclear Power Station, Unit 1

**Type of Document:**

Informal Report

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**Date of Document:**

May 1981

**Responsible NRC Individual and NRC Office or Division:**

P. Bender/R. Wilson, ICSB

**Preliminary**

This document was prepared primarily for preliminary or internal use. It has not received full review and approval. Since there may be substantive changes, this document should not be considered final.

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Prepared for the  
U.S. Nuclear Regulatory Commission  
Washington, D.C.  
Under DOE Contract No. B&R 201904031  
NRC FIN No. A-0250

INTERIM REPORT

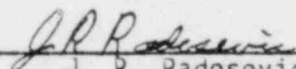
**TECHNICAL EVALUATION OF THE LICENSEE'S RESPONSE  
TO I&E BULLETIN 80-06  
CONCERNING ESF RESET CONTROLS FOR THE  
MILLSTONE NUCLEAR POWER STATION, UNIT 1**

(DOCKET NO. 50-245)

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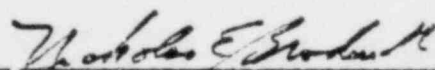
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Approved for Publication

  
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## INTRODUCTION

On March 13, 1980, the USNRC Office of Inspection and Enforcement (I&E), issued I&E Bulletin 80-06, entitled "Engineered Safety Feature (ESF) Reset Controls," to all PWR and BWR facilities with operating licenses. I&E Bulletin 80-06 requested that the following actions be taken by the licensees:

- (1) Review the drawings for all systems serving safety-related functions at the schematic/elementary diagram level to determine whether or not upon the reset of an ESF actuation signal all associated safety-related equipment remains in its emergency mode.
- (2) Verify that the actual installed instrumentation and controls at the facility are consistent with the schematic reviewed in Item 1 above by conducting a test to demonstrate that all equipment remains in its emergency mode upon removal of the actuating signal and/or manual resetting of the various isolating or actuation signals. Provide a schedule for the performance of the testing in your response to this bulletin.
- (3) If any safety-related equipment does not remain in its emergency mode upon reset of an ESF signal at your facility, describe proposed system modification, design change, or other corrective action planned to resolve the problem.
- (4) Report in writing within 90 days the results of your review, include a list of all devices which respond as discussed in Item 3 above, actions taken or planned to assure adequate equipment control, and a schedule for implementation of corrective action.

This technical evaluation addresses the licensee's response to I&E Bulletin 80-06 and the licensee's proposed system modification, design change, and/or other corrective action planned to resolve the problem. In evaluating the licensee's response to the four Action Item requirements of the bulletin, the following NRC staff guidance is also used:

Upon the reset of ESF signals, all safety-related equipment shall remain in its emergency mode. Multiple reset sequencing shall not cause the affected equipment to deviate from its emergency mode. Justification should be provided for any exceptions.

## EVALUATION AND CONCLUSIONS

In a letter dated June 13, 1980 [Ref. 1], Northeast Nuclear Energy Company (NNECO), the licensee for Millstone Nuclear Power Station, Unit 1, replied to I&E Bulletin 80-06. Following a February 15, 1981 telephone conference call [Ref. 2], a meeting was held on February 24, 1981 between the licensee and the NRC Project Manager [Ref. 3] during which additional information was provided.

NNECO reported [Ref. 1] that a review of the drawings for all systems serving safety-related functions had been completed to determine whether or not upon the reset of an ESF actuation signal all associated safety-related equipment remains in its emergency mode. The licensee identified four iso-condenser valves that could move from their emergency mode upon ESF reset. We conclude that the licensee has complied with the requirements of Action Item 1 of I&E Bulletin 80-06 by completing the drawing review of all systems serving safety-related functions.

NNECO has committed [Ref. 1] to perform a test prior to startup from the 1980 refueling outage to demonstrate that the installed instrumentation and controls at the facility are consistent with the schematics. We conclude that the licensee has complied with the requirements of Action Item 2 of I&E Bulletin 80-06 by providing a schedule for the performance of testing.

NNECO has committed to perform modifications to each of the components identified by the licensee, and has provided [Ref. 3] both a narrative description and marked-up drawings of all proposed modifications. This information was forwarded to EG&G/SRU by the NRC Technical Contact. The modifications, as presented in reference 3, will assure that each of the items identified will remain in its emergency mode position upon ESF reset; therefore, we conclude that the licensee has complied with the requirements of Action Item 3 of I&E Bulletin 80-06.

NNECO committed to install and test the modifications prior to startup from the current outage [Ref. 2]. We conclude that the licensee has complied with Action Item 4 of I&E Bulletin 80-06 by providing this schedule and in their response to Action Items 1 and 3 above.

## FINDINGS

Based on our review of the information and documents provided, we find the ESF reset controls for the Millstone Nuclear Power Station, Unit 1, comply with the requirements of I&E Bulletin 80-06.

#### REFERENCES

1. Northeast Utilities letter (W. G. Council) to NRC (B. H. Grier), "Millstone Nuclear Power Station, Unit No. 1, I&E Bulletin 80-06-- Engineered Safety Features Reset Control," dated June 13, 1980.
2. Telephone conference call, NKC (P. Bender) and EG&G/SRU (K. Jacoby) to NNECO (P. Biasioli), February 19, 1981.
3. Meeting between NRC and NNECO, Lyme, Connecticut, February 24, 1981.