YANKEE ATOMIC ELECTRIC COMPANY



1671 Worcester Road, Framingham, Massachusetts 01701

July 31, 1981 FYR 81-116

United States Nuclear Regulatory Commission Office of Nuclear Reactor Regulation Washington, D.C. 20555

Attention:

Mr. Darrell G. Eisenhut, Director

Division of Licensing

References:

(a) License No. DPR-3 (Docket No. 50-29)

(b) USNRC Letter to All SEP Licensees, dated July 7, 1981

Subject: Systematic Evaluation Program (SEP) Topic Assessments

Dear Sir:

This letter responds to your request that we review the schedule proposed in Enclosure 2 of Reference (b) for completion of safety assessment reports (SAR) for the remaining SEP topics. This review has been conducted and a modified schedule is presented in Table 1. We feel that this schedule is consistent with the SEP redirection, yet allows sufficient time for quality topic safety assessments to be developed.

We wish to note that Table 1 reflects only those items from your schedule designated as licensee topic SAR submittal dates. For other topics designated in Enclosure 2 as being developed by the NRC or its contractors, we commit to an expeditious review of the draft safety evaluation reports, consistent with other requirements imposed by the NRC.

Furthermore, in a meeting held on July 15, 1981, you requested additional commitments to provide sufficient resources to support the proposed SEP redirection approach. This letter will, therefore, serve to reconfirm our intention to complete the Systematic Evaluation Program in an expeditious manner.

We trust you will find this information satisfactory. However, if you have any questions, please contact us.

Very truly yours,

YANKEE ATOMIC ELECTRIC COMPANY

J. A. Kay

Senior Engineer - Licensing

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TABLE 1

Topic No.	<u>Title</u>	Schle
II-1.A	Exclusion Area Authority and Control	2/15/82
II-4.D	Stability of Slopes	2/15/82
11-4.F	Settlement of Foundations and Buried Equipment	8/31/81
III-2	Wind and Tornado Loadings	1/15/82
III-3.A	Effects of High Water Level on Structures	2/15/82
III-3.C	Inservice Inspection of Water Control Structures	12/31/81
III-4.A	Tornado Missiles	1/15/82
I'.I-4.C	Internally Generated Missiles	2/15/82
III-5.A	Effects of Pipe Break on Structures, Systems and	10/30/81
	Components Inside Containment	
III-5.B	Pipe Break Outside Containment	10/30/81
III-7.D	Containment Structural Integrity Test	2/15/82
IV-2	Reactivity Control Systems Including Functional	8/31/81
	Design and Protection Against Single Failures	
VI-1	Organic Material and Post Accident Chemistry	2/15/82
VI-2.D	Mass and Energy Release for Possible Pipe	12/15/81
	Break Inside Containment	
VI-3	Containment Pressure and Heat Removal Capability	12/15/81
IX-1	Fuel Storage	12/31/81
IX-3	Station Service and Cooling Water Systems	8/31/81
IX-4	Boron Addition System	8/31/81
IX-5	Ventilation Systems	9/30/81
XV-1	Decrease in Feedwater Temperature, Increase in	12/31/81
	Feedwater Flow, Increase in Steam Flow and	
	Inadvertent Opening of a Steam Generator Relief	
	or Safety Valve	
XV-4	Loss of Non-Emergency A-C Power to the Station	12/31/81
	Auxiliaries	
XV-6	Feedwater System Pipe Breaks I-side and Outside	12/31/81
	Containment (PWR)	
XV-8	Control Rod Misoperation (System Malfunction or	12/31/81
	Operator Error)	

Topic No.	<u>Title</u>	Schedule
XV-16	Radiological Consequences of Failure of Small Lines	12/31/81
	Carrying Primary Coolant Outside Containment	
XV-17	Steam Generator Tube Failure (PWR)	12/31/81
XV-19	Loss-of-Coolant Accident Resulting from Spectrum	9/30/81
	of Postulated Piping Breaks within the Reactor	
	Coolant Pressure Boundary	