NRC FORM 366 U. S. NUCLEAR REGULATORY COMMISSION (7.77) LICENSEE EVENT REPORT CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) 0 H 0 0 0 01 0 0 0 03 $\frac{4 | 1 | 1 | 1 | 1 | 1}{5 \text{ LICENSE TYPE 30}} 4$ J (5) CON'T REPORT 6 6 6 0 7 1 4 8 1 0 1 510 0 0 2 1 8 0 L(6) 0SOURCE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10 0 2 On 7/14/81 Licensee received information from the contractor conducting IEB 80-11 "Masonry Block Malls" analyses that stresses in four walls in 0 3 the control room and two walls in the battery room would exceed the 0 4 revised acceptance criteria in the event of an SSE.Failure of these 0 5 walls could impact safety related equipment including the plant bat-0 6 teries, electrical distribution panels, and control boards. This event is 0 reportable under T.S. 15.6.9.2.A.9. PB Unit 2 is also affected by this event. SYSTEM CAUSE CAUSE COMP. SUBCODE VALVE COMPONENT CODE Z (13) ZZ X (12) Z ZZ ZZ (14 Z (15 Z (16 SEQUENTIAL OCCURRENCE REPOR REVISION EVENT YEAR LER/RO REPORT REPORT NO CODE TYPE NO. 0 0 9 01 1 T 0 1 NUMBER ACTION FUTURE SHUTDOWN NPRD-4 FORM SUB PRIME COMP ATTACHMENT SUBMITTED COMPONENT HOURS MANUFACTURER F Z Z (21 0 0 0 0 0 YI IN 91 919 Z CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Overstress was discovered during reanalysis of masonry block walls in accordance with IEB 80-11. Corrective action for these walls is described in detail in Licensee's letter dated July 17, 1981. Modifications included installation of clip angles and structural steel supports All wall repairs will be completed in October 1981. 80 FACILITY METHOD OF DISCOVERY OTHER STATUS (30) POWER 80-11 G (28 01 01 01 N/A Bulletin DI work (31 ACTIVITY CONTENT 80 AMOUNT OF ACTIVITY (35 RELEASED OF RELEASE LOCATION OF RELEASE (36) Z (33) Z (34) N/A N/A 80 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 0 (37) Z (38) N/A 01 0 PERSONNEL INJURIES 80 DESCRIPTION (41) 0 0 0 (40) N/A 80 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION Z (42) N/A DESCRIPTION (45 NRC USE ONLY (44) N/A 8108050149 810728 PDR ADDCK 05000266 PDR 414-277-2811 C. W. Fay PHONE ..

Attachment to Licensee Event Report No. 81-009/01T-0

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Wisconsin Electric Power Company Point Beach Nuclear Plant, Units 1 and 2 Docket Nos. 50-266 and 50-301

During the afternoon of July 15, 1981, the Wisconsin Electric Power Company Nuclear Engineering Safety Review Committee met to consider information received on July 14, 1981 from the Bechtel Power Corporation regarding the investigation of masonry block walls at the Point Beach Nuclear Plant. Bechtel has been retained by Wisconsin Electric to conduct masonry wall investigations and analyses in accordance with the requirements of IE Bulletin 80-11. On July 14, 1981 Bechtel reported by telephone that their analyses of several of the walls in the Point Beach Nuclear Plant control room and battery rooms indicated that in the event of an SSE stresses in these walls exceeded the revised acceptance criteria stress limits as established during the meeting between Licensee and the NRC Staff on June 9-11, 1981. These walls are identified as wall numbers 19 and 20 in the battery room and walls 111-1, 111-3N, 113, and 114 in the control room.

Details of the wall locations and the proposed corrective modifications were provided to the NRC in Licensee's letter to Mr. Denton dated July 17, 1981. These modifications include installation of positive shear transfer mechanisms (clip angles, along the wall boundaries and upgrading with structural steel supports to relieve the flexural tension overstress. Block wall 20, the north-south divider wall in the battery room, has already been modified to eliminate the SSE-induced overstress condition. Modifications to the remaining walls will be completed in October 1981.

On July 15, 1981, the Wisconsin Electric Nuclear Engineering Safety Review Committee evaluated this information and determined that continued operation of the Point Beach Nuclear Plant pending modifications to these walls was justified. This determination was based on the following considerations:

For walls 19 and 20 the degree of SSE overstress indicated by the analysis was small with respect to the factor of safety of approximately four between the stress allowed by the acceptance criteria and the ultimate strength.

The remaining walls, 111-1, 111-3N, 113, and 114 had higher SSE overstress conditions; however, it was concluded that a failure of these walls would not preclude the ability of plant personnel to safel, shut down the plant using the remote shut-down panels as necessary. It was further observed that the

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Point Beach Nuclear Plant is located in a relatively inactive earthquake area and the probability of seismic activity with ground accelerations equaling the SSE design for the plant within the three months necessary to complete the wall modifications is exceedingly remote.

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This event is reportable in accordance with Technical Specification 15.6.9.2.A.9.