3. Licensee Identified Construction Deficiency Reports

(Closed) Byron Jackson Pump Seal Leakage. A deficiency involving a portion of the pump seal cooling water piping not tested to the pressure required by the ASME specifications for five ECCS pumps as manufactured by Byron Jackson for River Bend was reported to the NRC by GSU in a letter dated December 2, 1977 (RBG-4757). A December 30, 1977 letter (RBG-4813) from GSU to the NRC concluded that, as a result of their evaluation, this deficiency was not reportable under the requirements of 10 CFR 50.55(e). Furthermore, a March 13, 1978 letter from Byron Jackson to the NRC stated that the evaluation revealed that the deficiency was not reportable under the requirements of 10 CFR 21 as it does not meet the criteria for "Determination of Creation of a Substantial Safety Hazard." This letter further stated that all subject piping will be hydrostatically tested to the pressure required by the appropriate ASME specification. This item is considered closed.

(Closed) Second Pop Phenomenon. In a November 4, 1977 letter (RBG-4688), GSU notified the NRC Region IV Office of a 10 CFR 50.55(e) reportable deficiency regarding the safety relief valve second pop phenomenon. It was determined that more than one valve will reactuate after the initial pressure transient from a reactor isolation instead of a single valve predicted for Mark III containment design. General Electric has likewise reported this design deficiency in accordance with the requirements of 10 CFR 21. A letter explaining that the final analysis of this reportable deficiency will be included in the River Bend safety analysis report has been sent to the Region IV Office (RBG-10,269). GE's generic resolution will incorporate a modification to the SRV control system logic as well.

(Open) Miscellaneous Steel Provided by CIVES Steel. The NRC Region IV Office was notified on December 6, 1979 of a potentially reportable deficiency with regard to miscellaneous steel provided by CIVES Steel Company. The problem involved CIVES procurement of steel from suppliers who did not meet applicable quality assurance requirements. In addition, a small amount of the questionable material was inadvertently installed.

A review of Stone and Webster Specification 210.320, Revision 1, "Miscellaneous Steel and Embedments, Category I," indicated four nonconformance and disposition (N&D) reports were initiated indicating low yield strength readings of questionable material for plates from one of the suppliers for CIVES Steel.

N&D 9209 documents that a test specimen obtained from the same heat as that used on the mat anchor ring exhibited a yield strength of 48.3 ksi versus the specified 50 ksi of ASTM 588-74. These were confirmatory tests. The manufacturer's test indicated 52 ksi and 52.5 ksi. N&D 9220 documents a 47.3 ksi for this same test performed by the independent testing agency. In addition, the element nickel composition test performed by the agency conflicted with the manufacturer's test results.

N&D 9225 and N&D V009 document other suspect material from this same manufacturer and supplier to CIVES steel.

These conditions were evaluated by the licensee and found not to constitute a reportable deficiency under the requirements of 10 CFR 50.55(e).

The RRI, however, requires assurance that the design calculations for the mat anchor ring reflect that the actual stresses are less than the maximum allowed and that the appropriate design criteria as defined in the safety analysis report has not been violated by the low yield strength readings obtained during tensile testing. Additionally, the RRI requires assurance that none of the suspect material performs an actual structural function and has little or no structural significance.

This item will remain open.

(Closed) Incorrect input to the finite element model used to calculate the amplified floor response spectra of the Reactor Building structures due to the hydrodynamic forces in the suppression pool. This subject was reported to the Region IV Office as a potentially reportable construction deficiency (10 CFR 50.55(e)) on August 14, 1980. Stone and Webster concluded that by correcting the discrepancy of the input in the calculations, there was no adverse impact on the design of the structures and had this problem remained uncorrected, it would not have affected the safety of the plant. A letter from the licensee to the Region IV Office dated April 1, 1981 (RBG-10.034) concluded that this item is not a reportable deficiency under 10 CFR 50.55(e).

This item is considered closed