

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
REGION IV

Report No. 50-445/81-08; 50-446/81-08

Docket No. 50-445; 50-446

Category A2

Licensee: Texas Utilities Generating Company
2001 Bryan Tower
Dallas, Texas 75201

Facility Name: Comanche Peak, Units 1 and 2

Inspection at: Comanche Peak Steam Electric Station, Glen Rose, Texas

Inspection conducted: May 26-29, and June 1-5, 1981

Inspector: *R. C. Stewart*
R. C. Stewart, Reactor Inspector
Project Section 3, Reactor Projects Branch

6/24/81
Date

W. A. Crossman
W. A. Crossman, Chief
Project Section 3, Reactor Projects Branch

6/24/81
Date

Inspection Summary

Inspection on May 26-29 and June 1-5, 1981 (Report No. 50-445/81-08;
50-446/81-08)

Areas Inspected: Routine, unannounced inspection of construction activities and observation of work activities related to the review of the pressurizer safety and relief valves QA/QC Documentation, including the review of the applicable sections of the FSAR and ASME codes. In addition, observations were made of the in process work activities related to the installation of the Unit 1 pressurizer safety and relief valves, and a general overall review of construction progress of Units 1 and 2 as related to NRC inspection activities. The inspection involved 40 inspector-hours by one NRC inspector.

Results: No violations or deviations were identified.

DETAILS1. Persons Contacted

R. G. Tolson, TUGCO, Site QA Supervisor
J. T. Merrit, TUSI, Manager, Engineering and Construction
C. Osborne, TUGCO, Document Control Supervisor
*B. Scott, TUGCO, QE Supervisor
W. Cromeans, TUGCO

The NRC inspector also interviewed other licensee and contractor personnel during the course of the inspection.

*Denotes those attending the exit interview.

2. Site Tour

The NRC inspector walked through various construction and storage areas to observe construction activities in progress and to inspect the general state of cleanliness and adherence to housekeeping requirements. The tour included the Reactor Building, Reactor Auxiliary Building and the Fuel Handling Building.

No violations or deviations were identified.

3. Plant Construction Progress

The licensee's construction schedules indicate that the Unit 1 construction progress is 86% complete and that Unit 2 is 55% complete. During the period May 26-29, 1981, the NRC inspector reviewed the on-site NRC inspection records relative to the status and observation of construction activities to assure that the NRC inspection program requirements are concurrent with the work progress. The results of the review indicate that the NRC inspection program requirements are being maintained commensurate with the pace of construction progress.

4. Safety Related Components - Review of Quality Control Records

The NRC inspector selected the QA/QC documentation related to the Unit 1 Pressurizer safety and relief valves for a record review. Specific valves selected and documents reviewed were as follows:

Valves

Safety Relief Valves (Crosby) Serial Nos. N56964-00-0006, N56964-00-0007, and N56964-00-0008.

Power Operated Relief Valves (Copes-Vulcan) Serial Nos. 7620-95372-204-1 and 7620-95372-204-2.

Documents Reviewed

CPSES, FSAR, Sections 5.0, 5.4.13, 3.9N, and 17A.

B&R Procedure QI-QAP-11.1-28, Revision 7, "Installation Inspections of ASME Component Supports, Class 1, 2, and 3," dated May 28, 1981.

Microfiche Files, 252012 - TCX-220-35199, and 178368-TBX-220-31738, which include:

B&R "QA Department Receiving Inspection Reports"

B&R "Prior to Off-Loading Check"

B&R "Receiving Inspection Checklist for Westinghouse Supplied Q Equipment"

B&R "Material Receiving Records"

Westinghouse documents which include:

Material Certifications

Heat Treat Records

PT & UT Test Reports

Approved Stress Report CL.I

Approved Design Analysis CL.II

Pressure Test Records

Seat Tightness Test Records

Welding Personnel Qualification Certifications

NDE Personnel Qualification Certifications

ASME "NPV-1 Manufacturers Data Report for Nuclear Pumps or Valves."

Visual Inspection Reports

Dimensional Inspection Reports

Cleanliness, Painting, and Packaging Reports.

The review also included the review of certified test reports on 1.25-8 UNX 5 3/4" long ASME SA453 Grade 660B, Class I bolt studs and 1.25-8 UNX Heavy Duty Hex Head Nuts, ASME SA 194 Grade 6.

No violations or deviations were identified.

5. Safety Related Components - Observation of Work and Work Activities

The NRC inspector conducted a visual examination of the two installed Power Operated Relief Valves (PORV), Serial Nos. 7620-95371-204-1 and 204-2, and one installed Safety Relief Valve (SRV), Serial No. N56964-00-0006. It was observed by the NRC inspector that no work was in progress at the time of the examination; however, welding was completed on one of the PORVs and the SRV was positioned on the pipe flange but the bolt studs were not torqued. A followon inspection in this area will be conducted during a subsequent NRC site inspection.

No violations or deviations were identified.

6. Exit Interview

The NRC inspector met with the licensee representatives (denoted in paragraph 1) and Mr. R. G. Taylor (NRC Resident Reactor Inspector) at the conclusion of the inspection on June 5, 1981. The NRC inspector summarized the purpose, scope and findings of the inspection.