UNITED STATES OF AMERICA

NUCLEAR REGULATORY COMMISSION

In the Matter of

8107220050 810710 PDR ADOCK 0500034 Docket No. 50-344

PORTLAND GENERAL ELECTRIC COMPANY THE CITY OF EUGENE, OREGON PACIFIC POWER AND LIGHT COMPANY

(Trojan Nuclear Plant)

ORDER CONFIRMING LICENSEE COMMITMENTS

Ι.

Portland General Electric Company, <u>et al.</u> (the licensee) is the holder of Facility Operating License No. NPF-1, which authorizes the operation of the Trojan Nuclear Plant (the facility) at steady-state power levels not in excess of 3411 megawatts thermal. The facility consists of a pressurized water reactor (PWR) located at the licensee's site in Columbia County, Oregon.

II.

Following the accident at Three Mile Island Unit No. 2 (TMI-2) on March 28, 1979, the Nuclear Regulatory Commission (NRC) staff developed a number of proposed requirements to be implemented on operating reactors and on plants under construction. These requirements include Operational Safety, Siting and Design, and Emergency Preparedness and are intended to pro.ide substantial additional protection for the operation of nuclear Sacilities based on the

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experience from the accident at TMI-2 and the official studies and investigations of the accident. The staff's proposed requirements and schedule for implementation are set forth in NUREG-0737, "Clarification of TMI Action Plan Requirements." Among these requirements are a number of items, consisting of hardware modifications, administrative procedure implementation and specific information to be submitted by the licensee, scheduled to be completed on or before June 30, 1981 (see the Attachment to this Order). NUREG-0737 was transmitted to each licensee and applicant by an NRC letter from my office dated October 31, 1980, which is hereby incorporated by reference. In that letter, it was indicated that although the NRC staff expected each requirement to be implemented in accordance with the schedule set forth in NUREG-0737, the staff would consider licensee requests for relief from staff proposed requirements and their associated implementation dates.

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III.

The licensee's submittal dated December 23, 1980 and the references stated therein, which are incorporated herein by reference, committed to complete each of the actions specified in the Attachment. The licensee's submittals included a modified schedule for submittal of certain information. The staff has reviewed the licensee's submittal and determined that the licensee's modified schedule is acceptable based on the following: The licensee's schedule for submittal of information in some instances does not meet the staff's specified submittal dates. Most of the information requested by the staff describes how the licensee is meeting the guidance of NUREG-0737. Therefore, this deferral of the licensee submittal will not alter the implementation of plant modifications. Therefore, plant safety is not affected by this modification in schedule for the submittal of information.

I have determined that these commitments are required in the interest of public health and safety, and therefore, should be confirmed by ORDER.

IV.

Accordingly, pursuant to Sections 103, 161i, 161o, and 182 of the Atomic Energy Act of 1954, as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, IT IS HEREBY ORDERED EFFECTIVE IMMEDIATELY THAT the licensee shall comply with the following conditions:

The licensee shall satisfy the specific requirements described in the Attachment to this Order (as appropriate to the licensee's facility) as early as practicable but no later than 30 days after the effective date of the ORDER.

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Any person who has an interest affected by this Order may request a hearing within 20 days of the date of publication of this Order in the <u>Federal Register</u>. Any request for a hearing shall be addressed to the Director, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. A copy shall also be sent to the Executive Legal Director at the same address. If a hearing is requested by a person other than the licensee, that person shall describe, in accordance with 10 CFR 2.714(a)(2), the nature of the person's interest and the manner in which the interest is affected by this Order. A REQUEST FOR HEARING SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

If a hearing is requested by the licensee or other persons who have an interest affected by this Order, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether, on the basis of the information set forth in Sections II and III of this Order, the licensee should comply with the conditions set forth in Section IV of this Order.

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This request for information was approved by OMB under clearance number 3150-0065 which expires June 30, 1983. Comments on burden and duplication may be directed to the Office of Management and Budget, Reports Management, Room 3208, New Executive Office Building, Washington, D. C.

This Order is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dar D irector

Division of Licensing Office of Nuclear Reactor Regulation

Dated at Bethesda, Maryland this 10th day of July , 1981

Attachment: NUREG-0737 Requirements

ATTACHMENT

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NUREG 0737 REQUIREMENTS

Item	<u>Title</u> A	pplicability	Specific Requirement Subm	Imp'/Doc n Due Dates
I.A.ï.1	STA	ALL	 (a) Provide STA coverage by degreed individuals (or equivalent), trained to licensee's program. 	1/1/81
			(b) Submit description of current training program and demonstrate compliance with 10-30-79 letter.	1/1/81
			(c) Submit description of long term training program.	1/1/81
1.4.2.1	Reactor Operator Qualification	ALL	Submit description of and imple- ment Upgraded Training Program.	8/1/80
I.C.1	Accident Procedures	ALL	 (a) Submit reanalyses of inade- quate core cooling and propose guidelines for emergency procedures, OR propose schedule and justify delays. 	1/1/81
			(b) Submit reanalysis of transients and accidents for emergency procedures, OR propose schedule and justify delays.	1/1/81
1.C.5	Feedback of Operating Experience	ALL	Implement procedures for feedback of operating experience (no documentation submittal is required).	1/1/81
1.C.6	Correct Performance or Operating Activiti	ALL es	Implement procedures to verify correct performance of operating activities (no documentation submittal is required).	1/1/81

Item	<u>Title</u>	Applicability	Specific Requirement	Imp1/Doc Subm Due Dates
II.B.2	Shielding	ALL	 (a) Have available design details for vital area modifications. 	1/1/81
			(b) Submit technical deviations to staff positions.	1/1/81
II.B.3	Post-Accident Sampling	ALL	Submit description of deviations from staff positions.	1/1/81
II.B.4	Training to Mitigate Core Damage	ALL	 (a) Have available for review a training program for mitigating core damage (no documen- tation submittal required 	1/1/81
			(b) Implement training program (no documentation submittal required).	4/1/81
°I.D.1	Performance Testing of RV/SRV's	ALL	<pre>(a) Submit test program (both GWR/PWR's).</pre>	7/1/80 (PWRs) 10/1/80 (BWRs)
			(b) Submit qualification program for PWR's block valve.	1/1/81 (PWRs)
II.E.1.2	Aux Feed Initiation and Flow	PWRs	 (a) Submit final design and documentation on safety grade flow indication. 	1/1/81
			(b) Submit final design and documentation on safety grade flow initiation.	1/1/81
II.E.4.2	Containment Isolation	ALL	 (a) Submit documentation justifying minimum contain ment pressure setpoint for isolation of non-essential penetrations. 	
			(b) Submit statement that purge valves not meeting CSS 6-4 (or interim posi- tion) are sealed and verific cation is performed every	1/1/81
			31 days.	

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Item	Title	Applicability	Specific Requirement	Imp1/Doc Subm Due Dates
II.F.1	Post-Accident Monitoring	ALL	a. For noble gas monitor and lodine/particulate sampling and analysis submit description and justification for deviations from staff requirements.	1/1/81
			b. Have available the final design infor- mation for noble gas monitor and iodine/ particulate sampling and analysis.	1/1/81
11.F.2	Inst. for Inadequa Core Cooling	ate ALL	Submit a report detailing the planned instrumentation system for monitoring inadequate core cooling.	
II.K.2.10	Antic. Trip on LJFW and TT	B&W	Submit final design for anticipatory trip as described in NUREG 0737.	1/1/81
.i.K.2.13	Thermal Mechanical Report	B&W	Submit report on effects of Hi Pressure Injection on vessel integrity for SB LOCA with no Aux Feedwater.	1/1/81
Ш.К.З.2	PORV/SV Failures	PWR	Submit report on SB LOCA and probability of failure of PORV/SV/RV.	1/1/81
II.K.3.3	SRV/SV Failures & Challenges	ALL	Submit report (historical and annually thereafter) of SRV/SV failures and chailenges.	1/1/81
II.K.3.7	. PORV Opening Probability	B&W	Submit report on the probability of a PORV opening during an over-pressurization transient.	1/1/81
II.K.3.9	PID Controller Modification	Selected <u>W</u> plants	Modify the Proportional Integral Derivative Controller (as recommended by \underline{W}). Advise NRC when modification is complete	12/1/80 ted.
II.K.3.12	Anticipatory Trip on Turbine Trip	¥	Submit confirmation of Anticipatory Trip. If nct currently implemented, submit modification design and schedule for implementation.	1/1/81

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Item	Title	Applicability	Specific Requirement	Imp1/Doc Subm Date
I1.K.3.13	Separation of HPCI/RCIC Initiation Levels; Auto Restart of RCIC	BWRs w/HPCI/RCIC	Submit results of evaluation and proposed modification as appropriate.	1/1/81
11.K.3.16	Reduce Challenges to RV's	BWR	Submit report on actions planned to reduce RV challenges	4/1/81
II.K.3.17	ECCS System Outages	ALL	Submit report on ECCS outages and propose changes to reduce outages.	1/1/81
11.K.3.18	ADS Logic Modifications	BWR	Submit report of feasibility of ADS system logic changes to eliminate need for manual actuation.	4/1/81
II.K.3.21	CSS/CPCI Restart	BWR	Submit report of evaluation, proposed modifications and analysis to satisfy staff positions.	1/1/81
11.K.3.22	RCIC Suction	BWR W/RCIC	Implement procedures and document verification of this change.	1/1/81
II.K.3.27	Common Reference for H ₂ O Level Instr	BWR uments	Implement change and submit documentation of changes.	1/1/81
II.K.3.29	Isolation I Condensor Performance	BWR w/ICs	Submit evaluation of I.C. performance.	4/1/81
11.K.3.30	SB_LOCA Methods	ALL	Submit outline of program for model.	11/15/80
II.K.3.44	Fuel Failure	SWR	Submit evaluation to verify no fuel fail	1/1/81
II.K.3.45	Manual Depressur- ization		Submit evaluation on other than ADS method for depressurization.	1/1/81

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Item	Title	Applicability	Specific Requirement	· Impl/Doc Subm Due Dates
III.D.3.3	Improved Inplant Iodine Monitoring	ALL	Have available means to accurately measure airborne radioiodine i n plant during an accident.	1/1/81
111.0.3.4	Control Room Habit	ability ALL	 Submit control room habitability evaluation information. 	1/1/81
			 Submit modifications necessary to assure CR habitability with a schedule for completion. 	1/1/81

Item	Title	1pplicability	Spe	cific Requirement	Subm Due Dates
111.0.3.3	Improved Inplant Iodine Monitoring	ALL .	acc rad	e available means to urately measure airborne lioiodine i n plant during accident.	1/1/81
111.0.3.4	Control Room Habit	ability ALL	a.	Submit control room habitability evaluation information.	1/1/81
			b.	Submit modifications necessary to assure CR habitability with a schedule for completion.	1/1/81