

LICENSEE EVENT REPORT

CONTROL BLOCK: \_\_\_\_\_ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 8 9 | F | L | T | P | S | 4 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | \_\_\_\_\_ | 5  
7 8 9 | LICENSEE CODE | 14 | 15 | LICENSE NUMBER | 25 | 26 | LICENSE TYPE | 30 | 37 | 38 | 39

CON'T  
7 8 9 | 0 | 1 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 5 | 1 | 7 | 0 | 1 | 1 | 2 | 8 | 1 | 3 | 0 | 2 | 1 | 1 | 8 | 1 | 9  
7 8 9 | 50 | 51 | DOCKET NUMBER | 56 | 59 | EVENT DATE | 74 | 75 | REPORT DATE | 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During startup, for a period of about 4 hours, the condensate storage tank  
0 3 | (CST) contained less than the 185,000 gallons required by TS 3.8.1.c.  
0 4 | Operation in this mode is permitted by TS 3.8.3. The CST inventory  
0 5 | during this interval was not reduced below approximately 180,000 gallons.  
0 6 | The only effect of having a reduced volume in the CST is illustrated  
0 7 | in FSAR Figure 9.11-1. Reference similar occurrence LER 250-80-2.

0 9 | SYSTEM CODE | W | F | 11 | CAUSE CODE | X | 12 | CAUSE SUBCODE | X | 13 | COMPONENT CODE | Z | Z | Z | Z | Z | Z | 14 | COMP SUBCODE | Z | 15 | VALVE SUBCODE | Z | 16 |  
17 | LER/RO REPORT NUMBER | 8 | 1 | EVENT YEAR | 21 | 22 | SEQUENTIAL REPORT NO. | 0 | 0 | 1 | OCCURRENCE CODE | 0 | 3 | REPORT TYPE | L | REVISION NO. | 0 |  
ACTION TAKEN | FUTURE ACTION | EFFECT ON PLANT | SHUTDOWN METHOD | HOURS | ATTACHMENT SUBMITTED | NPRO-A FORM SUB. | PRIME COMP. SUPPLIER | COMPONENT MANUFACTURER  
Z | 18 | X | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | Y | 23 | N | 24 | Z | 25 | Z | 9 | 9 | 9 | 9 | 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | There were no equipment functional problems associated with this event.  
1 1 | Increased water demand during low power physics testing exceeded water  
1 2 | treatment plant output and resulted in CST level decreasing to less than  
1 3 | the TS 3.8.1.c limit. Plant changes and modifications currently scheduled  
1 4 | will reduce the potential exposure to events of this type.

1 5 | FACILITY STATUS | C | 28 | % POWER | 0 | 0 | 2 | 29 | OTHER STATUS | NA | 30 | METHOD OF DISCOVERY | A | 31 | DISCOVERY DESCRIPTION | Operator Observation | 32

1 6 | ACTIVITY CONTENT RELEASED OF RELEASE | Z | 33 | Z | 34 | AMOUNT OF ACTIVITY | NA | 35 | LOCATION OF RELEASE | NA | 36

1 7 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | NA | 39

1 8 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | NA | 41

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | NA | 43

2 0 | PUBLICITY ISSUED DESCRIPTION | N | 44 | DESCRIPTION | NA | 45 | NRC USE ONLY

Additional Event Description and Probable Consequences

During startup, for a period of about 4 hours, the condensate storage tank (CST) contained less than the 185,000 gallons required by TS 3.8.1.c. Unit was undergoing low power physics testing after refueling. Operation in this mode is permitted by TS 3.8.3. The CST inventory during this interval was not reduced below approximately 180,000 gallons. The only effect of having a reduced volume in the CST is illustrated in FSAR Figure 9.11-1. The event had no effect on public health and safety. Previous occurrences of low CST levels from various causes have been reported as: LER 250-78-4, LER 250-78-11, LER 250-78-12, LER 250-78-13, LER 250-79-11, LER 250-79-29, LER 250-80-2, LER 251-78-7, LER 251-78-13, LER 251-78-14, LER 251-79-17, LER 251-80-2, LER 251-80-4.

Additional Cause Description and Corrective Actions

There were no equipment functional problems associated with this event. Increased water demand during low power physics testing exceeded water treatment plant output and resulted in CST level decreasing to less than the TS 3.8.1.c limit.

Plant changes and modifications currently scheduled will reduce the potential exposure to events of this type. Additional water storage capacity is being provided as a part of the planned upgrade of the demineralized water/deaeration system. This modification is currently scheduled to be completed during the second quarter of 1981. A steam generator blowdown recovery system will be installed on each of the nuclear units. This modification is currently scheduled to be completed during the steam generator replacement outage on Unit 4 during the second quarter of 1982 and on Unit 3 during the second quarter of 1983.