

NUREG-0748
Vol. 1, No. 4

OPERATING REACTORS LICENSING ACTIONS SUMMARY



UNITED STATES NUCLEAR REGULATORY COMMISSION



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OPERATING REACTORS LICENSING ACTIONS SUMMARY

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Prepared by: OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS
U.S. NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555



FOREWORD

THE OPERATING REACTORS LICENSING ACTIONS SUMMARY IS DESIGNED TO PROVIDE THE MANAGEMENT OF THE NUCLEAR REGULATORY COMMISSION (NRC) WITH AN OVERVIEW OF LICENSING ACTIONS DEALING WITH OPERATING POWER AND NONPOWER REACTORS. THESE REPORTS UTILIZE DATA COLLECTED FROM THE DIVISION OF LICENSING IN THE OFFICE OF NUCLEAR REACTOR REGULATION AND ARE PREPARED BY THE OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS.

THIS SUMMARY REPORT IS PUBLISHED PRIMARILY FOR INTERNAL NRC USE IN MANAGING THE OPERATING REACTORS LICENSING ACTIONS PROGRAM. ITS CONTENT WILL CHANGE BASED ON NRC MANAGEMENT INFORMATIONAL REQUIREMENTS.

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Definitions

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I. Licensing Actions

- A. Active Licensing Action - an item which has been initiated but not completed.
 - 1. Multi-Plant Licensing Action - a licensing action topic that applies to more than one operating reactor.
 - 2. Plant Specific Licensing Action - a licensing action associated with a certain facility because of the uniqueness of the technical specifications or situation, e.g., reduce the number of incore detectors.
- B. Types of Licensing Actions
 - 1. Amendment - action that results in changes to facility licenses including technical specification changes.
 - 2. Hearings - Effort associated with public hearings that occur after an OL has been issued, including responses to interrogatories or discovery requests, preparation of testimony, etc.
 - 3. Letters to Licensees - letters sent to licensee requesting information that will resolve or aid in the resolution of technical issues. Examples of letters sent to licensees are: fuel handling inside containment, asymmetric loads or reactor vessel materials surveillance.
 - 4. Orders/Exemptions - actions on operating plant that result in an Order for Modification of a License or an exemption to the regulation.
- C. Sources of Licensing Actions
 - 1. New Information - action required as a result of current data which shows that previously accepted requirements are no longer acceptable or are overly restrictive, e.g., ECUS input errors, asymmetric loads, equipment qualification. This new information may result from technological advancements, research or advanced analytical studies.
 - 2. Regulation - actions directly attributed to a specific regulation requirements, e.g., inservice inspection/testing requirements 10 CFR 50.55a(q).
 - 3. Operating Experience - action directly attributed to an occurrence of accumulative experience at licensed reactors e.g., fire protection.
 - 4. Licensee Requirement - action requested by the licensee that does not fit any of the other sources of licensing actions. Usually these actions are requested because of the licensee's desire to improve operational performance, e.g., core reloads, spent fuel pool modifications, or (N-1) loop operation.
 - 5. Internal Review - action resulting from NRC staff reviews of license applications, generic topics and operating reactor licensing actions, e.g., reporting requirements, fuel handling accidents, or filter tech spec requirements.

II. Significant Dates

- A. Initiation - for a hearing, it is the date that the hearing is actually ordered. For an amendment, it is either the date the amendment request is received from the licensee, or, in cases where DOR initiates the action, the date that authority is received to submit a letter requiring the amendment. For letters to the licensee, it is the date that authority is received to initiate a letter to the licensee.
- B. PM Target - project manager's anticipated completion date for active items.
- C. Licensing Action Complete - date on which all work, both technical and administrative was completed and sent out of the agency.

II. Other

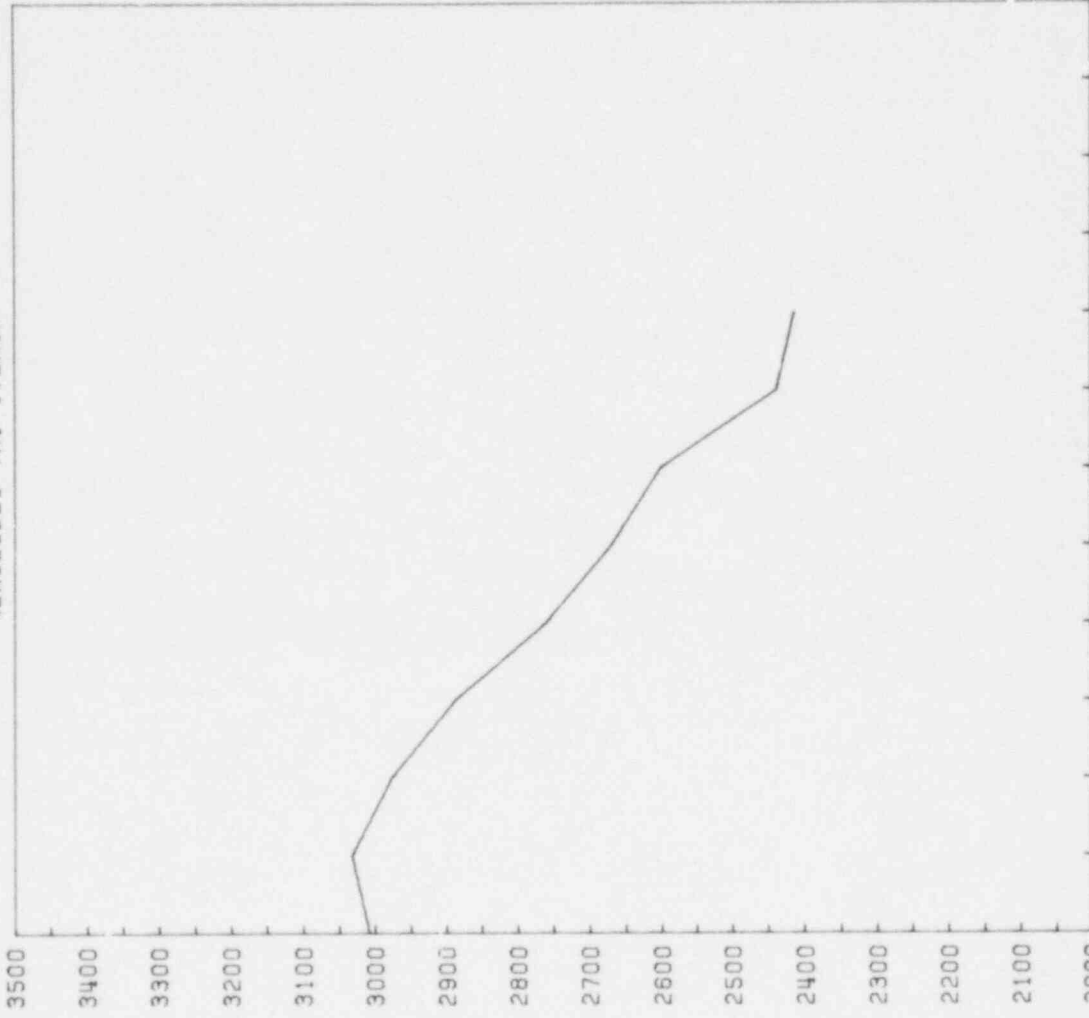
- A. Fee Class - for action items received since March 27, 1978 where licensing fee classifications have been made by NRC and/or proposed by the licensee, it is one of the six fee classifications. If the amendment action item came in prior to that date the fee class is not applicable. (See 10 CFR, 170.22)
- B. New Licensing Action - item which was initiated or implementation actually began on the plant during the current reporting period (month and fiscal year).
- C. Complete Licensing Action - item on which all work, both technical and administrative was completed during the current reporting period (month and fiscal year).

DEFINITION OF PRIORITIESAPPLICABLE TO MULTI-PLANTS OR PLANT SPECIFIC ACTIONS

- HIGH (1)
- Required by Rule, Order or NRC letter
 - Safety-related items that use definite improvements in safety or would reduce the likelihood of transients or accidents
 - Hearing-related matters
 - High public interest in issue
 - Actions which, if not completed, will affect power level or scheduled restart of a facility
 - § 2.206 petitions
- MEDIUM (2)
- Items which, if not completed, could result in an otherwise unnecessary IE enforcement action
 - Spent fuel pool expansions
 - Task should be completed within six months
- LOW (3)
- Licensee requests for amendments that have low safety significance
 - Licensee requests for amendments which may have safety significance but for which NRC action is not required during the next six months (e.g., in early submittal of a core reload)

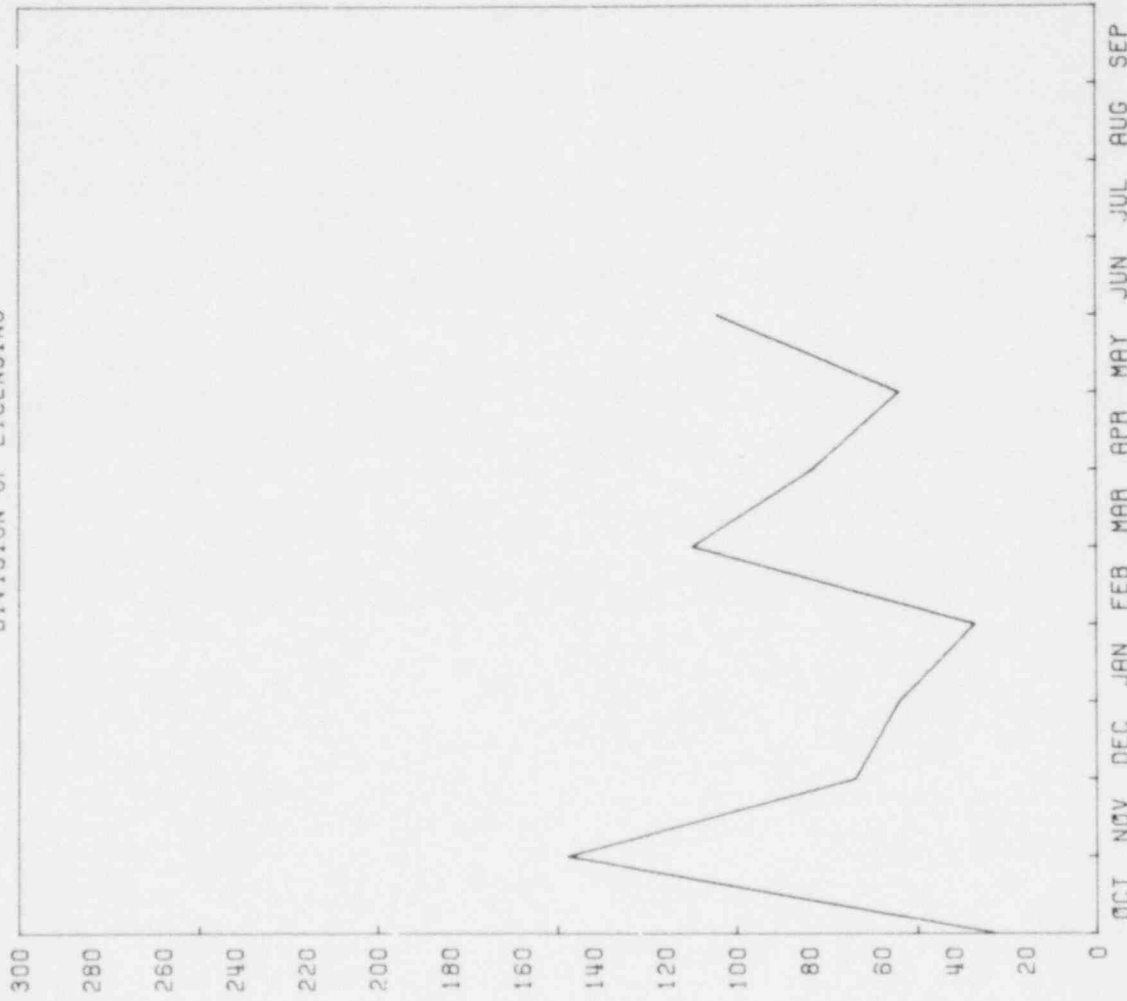
SECTION 1
MANAGEMENT OVERVIEW REPORT

TOTAL ACTIVE ROUTINE ACTIONS
 DIVISION OF LICENSING
 (EXCLUDES TMI ITEMS)



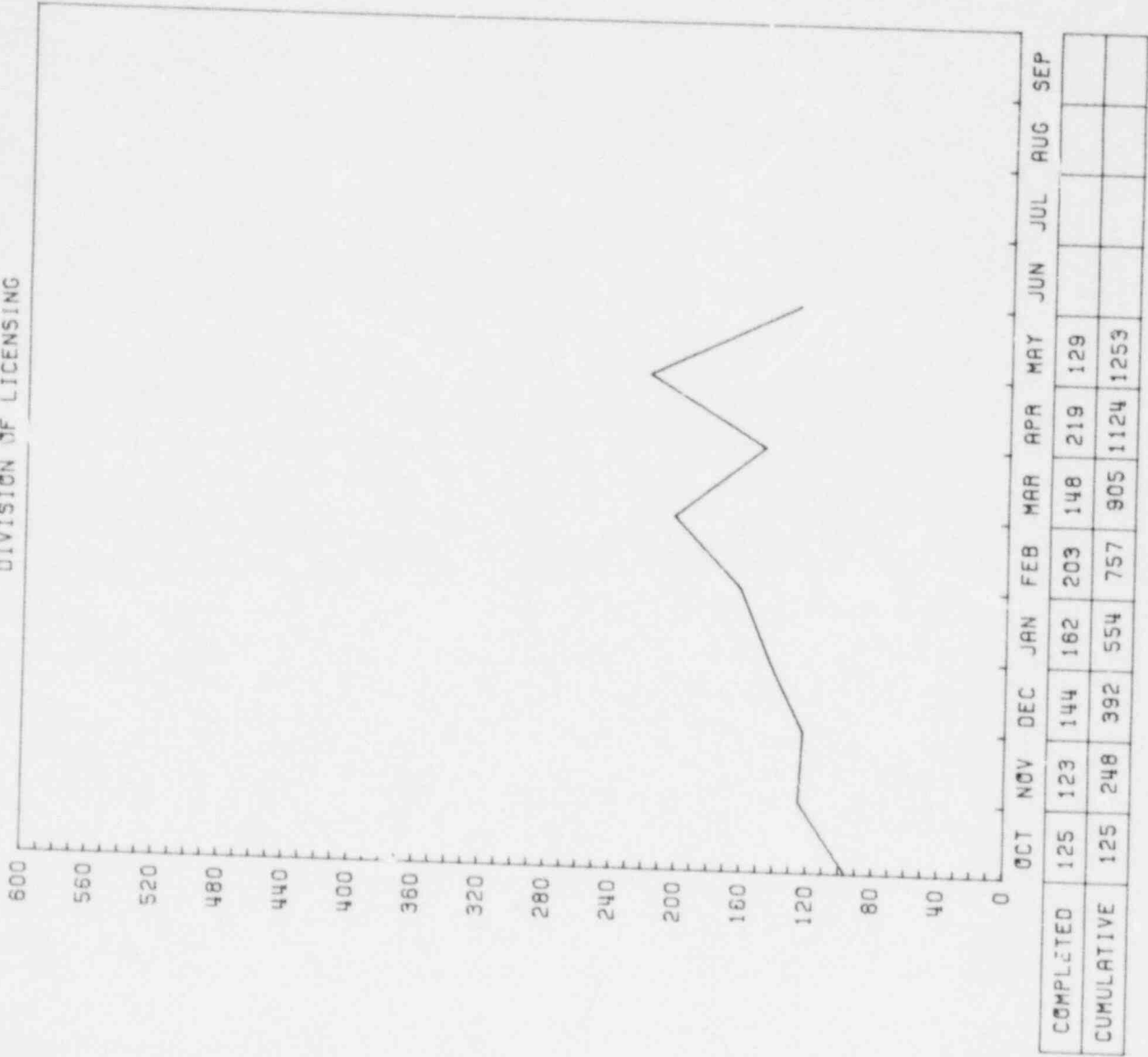
| | OCT | NOV | DEC | JAN | FEB | MAR | APR | MAY | JUN | JUL | AUG | SEP |
|----------|------|------|------|------|------|------|------|------|-----|-----|-----|-----|
| TOTAL | 3032 | 2976 | 2887 | 2759 | 2668 | 2599 | 2437 | 2413 | | | | |
| ACTIONS | 3013 | 2957 | 2868 | 2740 | 2653 | 2584 | 2422 | 2398 | | | | |
| HEARINGS | 19 | 19 | 19 | 19 | 15 | 15 | 15 | 15 | | | | |

NEW ROUTINE ACTIONS BY MONTH
DIVISION OF LICENSING



| | OCT | NOV | DEC | JAN | FEB | MAR | APR | MAY | JUN | JUL | AUG | SEP |
|-------------|-----|-----|-----|-----|-----|-----|-----|-----|-----|-----|-----|-----|
| NEW ACTIONS | 147 | 67 | 55 | 34 | 112 | 79 | 55 | 105 | | | | |
| CUMULATIVE | 147 | 214 | 269 | 303 | 415 | 494 | 549 | 654 | | | | |

COMPLETED ROUTINE ACTIONS BY MONTH
DIVISION OF LICENSING



TECHNICAL ASSIGNMENT CONTROL SYSTEM

FISCAL YEAR: 1981

ROUTINE LICENSING ACTIONS BY MONTH

REPORT DATE: 06/12/81

| <u>FACILITY NAME</u> | <u>OCT.</u> | <u>NOV.</u> | <u>DEC.</u> | <u>JAN.</u> | <u>FEB.</u> | <u>MAR.</u> | <u>APR.</u> | <u>MAY</u> | <u>JUNE</u> | <u>JULY</u> | <u>AUG.</u> | <u>SEPT.</u> |
|----------------------|-------------|-------------|-------------|-------------|-------------|-------------|-------------|------------|-------------|-------------|-------------|--------------|
| ARKANSAS 1 | 75 | 77 | 74 | 54 | 59 | 59 | 57 | 56 | | | | |
| ARKANSAS 2 | 62 | 59 | 58 | 57 | 54 | 48 | 47 | 48 | | | | |
| BEAVER VALLEY 1 | 49 | 44 | 44 | 43 | 36 | 34 | 32 | 31 | | | | |
| BIG ROCK POINT 1 | 47 | | 46 | 39 | 34 | 31 | 31 | 29 | | | | |
| BROWNS FERRY 1 | 60 | 60 | 58 | 55 | 51 | 51 | 55 | 55 | | | | |
| BROWNS FERRY 2 | 54 | 52 | 50 | 49 | 47 | 48 | 50 | 49 | | | | |
| BROWNS FERRY 3 | 56 | 56 | 55 | 49 | 45 | 45 | 46 | 46 | | | | |
| BRUNSWICK 1 | 36 | 38 | 36 | 31 | 31 | 34 | 37 | 39 | | | | |
| BRUNSWICK 2 | 38 | 40 | 38 | 34 | 34 | 36 | 39 | 40 | | | | |
| CALVERT CLIFFS 1 | 55 | 58 | 57 | 56 | 58 | 58 | 55 | 53 | | | | |
| CALVERT CLIFFS 2 | 44 | 46 | 49 | 49 | 44 | 46 | 42 | 42 | | | | |
| COOK 1 | 39 | 39 | 41 | 36 | 35 | 35 | 33 | 31 | | | | |
| COOK 2 | 42 | 42 | 44 | 40 | 38 | 37 | 35 | 34 | | | | |
| COOPER STATION | 41 | 35 | 34 | 31 | 29 | 29 | 30 | 30 | | | | |
| CRYSTAL RIVER 3 | 59 | 60 | 58 | 58 | 53 | 47 | 44 | 45 | | | | |
| DAVIS-BESSE 1 | 53 | 54 | 59 | 58 | 54 | 55 | 53 | 55 | | | | |
| DRESDEN 2 | 43 | 41 | 36 | 33 | 28 | 24 | 24 | 28 | | | | |
| DRESDEN 3 | 46 | 45 | 38 | 38 | 34 | 33 | 32 | 33 | | | | |
| DUANE ARNOLD | 48 | 47 | 47 | 44 | 47 | 47 | 45 | 44 | | | | |
| FARLEY 1 | 38 | 37 | 34 | 35 | 34 | 33 | 30 | 31 | | | | |
| FARLEY 2 | 3 | 4 | 4 | 4 | 4 | 5 | 6 | 10 | | | | |
| FITZPATRICK | 42 | 42 | 42 | 44 | 41 | 44 | 37 | 32 | | | | |
| FORT CALHOUN 1 | 34 | 32 | 33 | 33 | 31 | 30 | 28 | 28 | | | | |
| FORT ST VRAIN | 22 | 22 | 22 | 22 | 22 | 22 | 22 | 23 | | | | |
| GINNA | 48 | 51 | 49 | 50 | 48 | 44 | 36 | 36 | | | | |
| HADDAM NECK | 45 | 42 | 42 | 42 | 39 | 39 | 38 | 39 | | | | |
| HATCH 1 | 32 | 31 | 31 | 31 | 32 | 33 | 31 | 32 | | | | |
| HATCH 2 | 37 | 33 | 35 | 34 | 31 | 32 | 30 | 31 | | | | |
| INDIAN POINT 2 | 45 | 45 | 44 | 44 | 43 | 43 | 37 | 36 | | | | |
| INDIAN POINT 3 | 35 | 35 | 34 | 33 | 32 | 32 | 30 | 29 | | | | |
| KEWAUNEE | 44 | 45 | 45 | 45 | 50 | 48 | 42 | 42 | | | | |
| LA CROSSE | 37 | 37 | 35 | 35 | 31 | 31 | 32 | 33 | | | | |
| MAINE YANKEE | 46 | 46 | 47 | 46 | 44 | 41 | 40 | 39 | | | | |
| MCGUIRE 1 | 4 | 4 | 4 | 4 | 4 | 4 | 4 | 5 | | | | |
| MILLSTONE 1 | 48 | 48 | 48 | 45 | 42 | 40 | 37 | 38 | | | | |
| MILLSTONE 2 | 44 | 41 | 41 | 41 | 42 | 41 | 37 | 37 | | | | |
| MONTICELLO | 43 | 42 | 43 | 40 | 45 | 42 | 40 | 40 | | | | |
| NINE MILE POINT 1 | 49 | 50 | 47 | 46 | 42 | 41 | 35 | 29 | | | | |
| NORTH ANNA 1 | 39 | 34 | 30 | 29 | 29 | 26 | 24 | 20 | | | | |
| NORTH ANNA 2 | 18 | 16 | 14 | 14 | 12 | 9 | 7 | 5 | | | | |

R-1208612-001
FISCAL YEAR: 1981

TECHNICAL ASSIGNMENT CONTROL SYSTEM
ROUTINE LICENSING ACTIONS BY MONTH

PAGE: D-2
REPORT DATE: 06/12/81

| <u>FACILITY NAME</u> | <u>OCT.</u> | <u>NOV.</u> | <u>DEC.</u> | <u>JAN.</u> | <u>FEB.</u> | <u>MAR.</u> | <u>APR.</u> | <u>MAY</u> | <u>JUNE</u> | <u>JULY</u> | <u>AUG.</u> | <u>SEPT.</u> |
|----------------------|-------------|-------------|-------------|-------------|-------------|-------------|-------------|------------|-------------|-------------|-------------|--------------|
| OCONEE 1 | 54 | 52 | 48 | 43 | 43 | 39 | 31 | 28 | | | | |
| OCONEE 2 | 50 | 48 | 45 | 40 | 40 | 36 | 28 | 27 | | | | |
| OCONEE 3 | 49 | 47 | 45 | 40 | 40 | 36 | 28 | 27 | | | | |
| OYSTER CREEK 1 | 39 | 39 | 38 | 38 | 32 | 32 | 31 | 32 | | | | |
| PALISADES | 57 | 58 | 58 | 57 | 53 | 51 | 49 | 50 | | | | |
| PEACH BOTTOM 2 | 33 | 31 | 29 | 29 | 29 | 29 | 29 | 27 | | | | |
| PEACH BOTTOM 3 | 34 | 32 | 30 | 29 | 29 | 30 | 29 | 28 | | | | |
| PILGRIM 1 | 37 | 37 | 37 | 36 | 33 | 29 | 27 | 29 | | | | |
| POINT BEACH 1 | 37 | 37 | 36 | 36 | 36 | 36 | 32 | 32 | | | | |
| POINT BEACH 2 | 40 | 39 | 38 | 37 | 37 | 38 | 33 | 33 | | | | |
| PRAIRIE ISLAND 1 | 49 | 46 | 44 | 43 | 41 | 41 | 39 | 37 | | | | |
| PRAIRIE ISLAND 2 | 47 | 45 | 44 | 43 | 42 | 40 | 38 | 37 | | | | |
| QUAD CITIES 1 | 52 | 48 | 43 | 42 | 38 | 36 | 30 | 30 | | | | |
| QUAD CITIES 2 | 49 | 45 | 39 | 38 | 36 | 33 | 27 | 28 | | | | |
| RANCHO SECO 1 | 54 | 55 | 55 | 58 | 55 | 52 | 47 | 48 | | | | |
| ROBINSON 2 | 39 | 37 | 37 | 35 | 36 | 41 | 39 | 40 | | | | |
| SALEM 1 | 46 | 41 | 38 | 35 | 34 | 33 | 32 | 32 | | | | |
| SALEM 2 | 4 | 4 | 4 | 4 | 5 | 5 | 5 | 6 | | | | |
| SAN ONOFRE 1 | 56 | 56 | 57 | 53 | 53 | 46 | 43 | 44 | | | | |
| SEQUOYAH 1 | 2 | 2 | 3 | 3 | 4 | 4 | 4 | 5 | | | | |
| ST LUCIE 1 | 53 | 53 | 52 | 49 | 45 | 46 | 43 | 40 | | | | |
| SURRY 1 | 48 | 50 | 47 | 43 | 42 | 42 | 40 | 38 | | | | |
| SURRY 2 | 44 | 46 | 43 | 40 | 41 | 40 | 38 | 36 | | | | |
| THREE MILE ISLAND 1 | 47 | 47 | 48 | 45 | 42 | 39 | 34 | 37 | | | | |
| TROJAN | 35 | 37 | 33 | 33 | 35 | 34 | 30 | 24 | | | | |
| TURKEY POINT 3 | 44 | 43 | 40 | 37 | 40 | 43 | 41 | 39 | | | | |
| TURKEY POINT 4 | 46 | 45 | 43 | 39 | 41 | 44 | 43 | 39 | | | | |
| VERMONT YANKEE 1 | 50 | 46 | 42 | 39 | 40 | 37 | 34 | 34 | | | | |
| YANKEE-ROWE 1 | 41 | 39 | 37 | 37 | 34 | 34 | 32 | 33 | | | | |
| ZION 1 | 48 | 48 | 44 | 38 | 41 | 38 | 36 | 35 | | | | |
| ZION 2 | 48 | 48 | 44 | 39 | 42 | 38 | 36 | 35 | | | | |
| GRAND TOTAL | 3,032 | 2,976 | 2,887 | 2,759 | 2,668 | 2,599 | 2,437 | 2,413 | | | | |

TECHNICAL ASSIGNMENT CONTROL SYSTEM

REPORT DATE: 06/12/81

ROUTINE LICENSING ACTIONS

POWER REACTORS

FOR 05/01/81 - 05/31/81

| FACILITY | NEW ACTIONS FOR THIS REPORTING PERIOD | NEW ACTIONS TO DATE IN FY 81 | COMPLETED ACTIONS FOR THIS REPORTING PERIOD | COMPLETED ACTIONS TO DATE IN FY 81 | NUMBER OF ROUTINE ACTIONS | NUMBERS OF HEARINGS | TOTAL ACTIONS |
|------------------|---------------------------------------|------------------------------|---|------------------------------------|---------------------------|---------------------|---------------|
| ARKANSAS 1 | 1 | 26 | 2 | 37 | 56 | 0 | 56 |
| ARKANSAS 2 | 2 | 8 | 1 | 19 | 48 | 0 | 48 |
| BEAVER VALLEY 1 | 2 | 13 | 3 | 28 | 31 | 0 | 31 |
| BIG ROCK POINT 1 | 1 | 7 | 3 | 24 | 28 | 1 | 29 |
| BROWNS FERRY 1 | 0 | 11 | 0 | 15 | 55 | 0 | 55 |
| BROWNS FERRY 2 | 0 | 10 | 1 | 15 | 49 | 0 | 49 |
| BROWNS FERRY 3 | 0 | 8 | 0 | 14 | 46 | 0 | 46 |
| BRUNSWICK 1 | 2 | 12 | 0 | 10 | 39 | 0 | 39 |
| BRUNSWICK 2 | 1 | 10 | 0 | 9 | 40 | 0 | 40 |
| CALVERT CLIFFS 1 | 3 | 22 | 3 | 23 | 53 | 0 | 53 |
| CALVERT CLIFFS 2 | 1 | 15 | 1 | 16 | 42 | 0 | 42 |
| COOK 1 | 1 | 10 | 3 | 15 | 31 | 0 | 31 |
| COOK 2 | 4 | 12 | 5 | 17 | 34 | 0 | 34 |
| COOPER STATION | 1 | 2 | 1 | 13 | 30 | 0 | 30 |
| CRYSTAL RIVER 3 | 1 | 6 | 0 | 21 | 45 | 0 | 45 |
| DAVIS-BESSE 1 | 2 | 15 | 0 | 17 | 55 | 0 | 55 |
| DRESDEN 2 | 5 | 8 | 1 | 25 | 28 | 0 | 28 |
| DRESDEN 3 | 2 | 6 | 4 | 19 | 33 | 0 | 33 |
| DUANE ARNOLD | 2 | 8 | 3 | 11 | 44 | 0 | 44 |
| FARLEY 1 | 2 | 11 | 1 | 16 | 31 | 0 | 31 |
| FARLEY 2 | 4 | 8 | 0 | 0 | 9 | 1 | 10 |
| FITZPATRICK | 2 | 9 | 7 | 18 | 32 | 0 | 32 |
| FORT CALHOUN 1 | 1 | 11 | 1 | 18 | 28 | 0 | 28 |
| FORT ST VRAIN | 1 | 1 | 0 | 0 | 22 | 1 | 23 |
| GIHNA | 2 | 12 | 2 | 22 | 36 | 0 | 36 |
| HADDAM NECK | 1 | 9 | 0 | 14 | 39 | 0 | 39 |
| HATCH 1 | 3 | 11 | 3 | 11 | 31 | 1 | 32 |
| HATCH 2 | 1 | 9 | 0 | 15 | 30 | 1 | 31 |
| INDIAN POINT 2 | 1 | 11 | 2 | 20 | 36 | 0 | 36 |
| INDIAN POINT 3 | 1 | 4 | 2 | 10 | 29 | 0 | 29 |
| KEWAUNEE | 1 | 13 | 1 | 11 | 42 | 0 | 42 |
| LA CROSSE | 1 | 3 | 0 | 11 | 31 | 0 | 31 |
| MAINE YANKEE | 1 | 7 | 2 | 13 | 39 | 2 | 33 |
| MCGUIRE 1 | 1 | 1 | 0 | 0 | 39 | 0 | 39 |
| MILLSTONE 1 | 1 | 5 | 0 | 21 | 5 | 0 | 5 |
| | | | | | 38 | 0 | 38 |

REPORT DATE: 06/12/81

ROUTINE LICENSING ACTIONS

POWER REACTORS

FOR 05/01/81 - 05/31/81

| FACILITY | NEW ACTIONS FOR THIS REPORTING PERIOD | NEW ACTIONS TO DATE IN FY 81 | COMPLETED ACTIONS FOR THIS REPORTING PERIOD | COMPLETED ACTIONS TO DATE IN FY 81 | NUMBER OF ROUTINE ACTIONS | NUMBERS OF HEARINGS | TOTAL ACTIONS |
|---------------------|---|---------------------------------------|---|---|------------------------------------|---------------------------|------------------|
| MILLSTONE 2 | 2 | 9 | 2 | 20 | 37 | 0 | 37 |
| MONTICELLO | 0 | 8 | 0 | 11 | 40 | 0 | 40 |
| NINE MILE POINT 1 | 1 | 6 | 7 | 25 | 29 | 0 | 29 |
| NORTH ANNA 1 | 1 | 6 | 5 | 24 | 19 | 1 | 20 |
| NORTH ANNA 2 | 1 | 2 | 3 | 15 | 5 | 0 | 5 |
| OCONEE 1 | 2 | 15 | 5 | 41 | 28 | 0 | 28 |
| OCONEE 2 | 2 | 12 | 3 | 37 | 27 | 0 | 27 |
| OCONEE 3 | 2 | 13 | 3 | 36 | 27 | 0 | 27 |
| OYSTER CREEK 1 | 1 | 6 | 0 | 13 | 31 | 1 | 32 |
| PALISADES | 1 | 5 | 0 | 15 | 49 | 1 | 50 |
| PEACH BOTTOM 2 | 1 | 5 | 3 | 12 | 27 | 0 | 27 |
| PEACH BOTTOM 3 | 1 | 5 | 2 | 12 | 28 | 0 | 28 |
| PILGRIM 1 | 3 | 8 | 1 | 16 | 29 | 0 | 29 |
| POINT BEACH 1 | 1 | 13 | 1 | 16 | 32 | 0 | 32 |
| POINT BEACH 2 | 1 | 10 | 1 | 15 | 33 | 0 | 33 |
| PRAIRIE ISLAND 1 | 1 | 5 | 3 | 17 | 37 | 0 | 37 |
| PRAIRIE ISLAND 2 | 1 | 3 | 2 | 15 | 37 | 0 | 37 |
| QUAD CITIES 1 | 1 | 4 | 1 | 26 | 30 | 0 | 30 |
| QUAD CITIES 2 | 1 | 3 | 0 | 24 | 28 | 0 | 28 |
| RANCHO SECO 1 | 1 | 16 | 0 | 19 | 47 | 1 | 48 |
| ROBINSON 2 | 3 | 15 | 2 | 14 | 40 | 0 | 40 |
| SALEM 1 | 1 | 10 | 1 | 22 | 32 | 0 | 32 |
| SALEM 2 | 1 | 2 | 0 | 0 | 6 | 0 | 6 |
| SAN ONOFRE 1 | 1 | 10 | 0 | 20 | 42 | 2 | 44 |
| SEQUOYAH 1 | 1 | 3 | 0 | 0 | 5 | 0 | 5 |
| ST LUCIE 1 | 1 | 12 | 4 | 21 | 40 | 0 | 40 |
| SURRY 1 | 2 | 8 | 4 | 17 | 38 | 0 | 38 |
| SURRY 2 | 2 | 9 | 4 | 16 | 36 | 0 | 36 |
| THREE MILE ISLAND 1 | 6 | 11 | 3 | 22 | 37 | 0 | 37 |
| TROJAN | 1 | 14 | 7 | 25 | 24 | 0 | 24 |
| TURKEY POINT 3 | 2 | 18 | 4 | 23 | 38 | 1 | 39 |
| TURKEY POINT 4 | 1 | 16 | 5 | 23 | 38 | 1 | 39 |
| VERMONT YANKEE 1 | 0 | 9 | 0 | 25 | 34 | 0 | 34 |
| YANKEE-ROWE 1 | 1 | 11 | 0 | 19 | 33 | 0 | 33 |
| ZION 1 | 1 | 9 | 2 | 24 | 35 | 0 | 35 |

REPORT DATE: 06/12/81

ROUTINE LICENSING ACTIONS

POWER REACTORS

FOR 05/01/81 - 05/31/81

| FACILITY | NEW ACTIONS FOR THIS REPORTING PERIOD | NEW ACTIONS TO DATE IN FY 81 | COMPLETED ACTIONS FOR THIS REPORTING PERIOD | COMPLETED ACTIONS TO DATE IN FY 81 | NUMBER OF ROUTINE ACTIONS | NUMBERS OF HEARINGS | TOTAL ACTIONS |
|-------------|---|---------------------------------------|---|---|------------------------------------|---------------------------|------------------|
| ZION 2 | 1 | 9 | 2 | 24 | 35 | 0 | 35 |
| GRAND TOTAL | 105 | 654 | 129 | 1,253 | 2,398 | 15 | 2,413 |

REPORT DATE: 06/12/81

ACTIVE LICENSING ACTIONS

NON-POWER REACTORS

FOR 05/01/81 - 05/31/81

| FACILITY | NEW ACTIONS FOR THIS REPORTING PERIOD | NEW ACTIONS TO DATE IN FY 81 | COMPLETED ACTIONS FOR THIS REPORTING PERIOD | COMPLETED ACTIONS TO DATE IN FY 81 | NUMBER OF ROUTINE ACTIONS | NUMBERS OF HEARINGS | TOTAL ACTIONS |
|-----------------------|---------------------------------------|------------------------------|---|------------------------------------|---------------------------|---------------------|---------------|
| AEROTEST | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| AFRRI, TRIGA | 0 | 0 | 0 | 0 | 3 | 0 | 3 |
| B & W, CRITICAL FACI | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| B&W, LPR | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| BRIGHAM YOUNG, L-77 | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| CATHOLIC U., AGN-201 | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| COLUMBIA U TRIGA MK | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| CORNELL U., TRIGA | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| CORNELL U., ZPR | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| DOW CHEM., TRIGA MAR | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| GA. TECH | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| GA. TECH, AGN-201 | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| GA. TRIGA MARK I | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| GA, TRIGA MARK F | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| GE TEST REACTOR | 0 | 0 | 0 | 0 | 7 | 0 | 7 |
| GE, NTR | 0 | 0 | 0 | 0 | 3 | 0 | 3 |
| IDAHO STATE, AGN-201 | 0 | 1 | 0 | 0 | 1 | 0 | 1 |
| IOWA STATE, UTR-10 | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| KANSAS (T.), TRIGA MA | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| MANHATTAN COLLEGE | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| MICH. STATE, TRIGA M | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| MIT, MITR | 0 | 0 | 0 | 0 | 4 | 0 | 4 |
| N. C. STATE, PULSTAR | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| NBSR | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| NORTH. CORP, TRIGA M | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| OHIO STATE | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| OREGON ST, TRIGA MK | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| PENN STATE, TRIGA MA | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| PURDUE U. | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| REED INST, TRIGA MK | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| RENSSELAER | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| RHODE ISLAND | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| ROCKWELL INT., L-85 | 0 | 0 | 0 | 0 | 3 | 0 | 3 |
| STATE U OF NY PULSTA | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| TEXAS A&M, AGN-201M, | 0 | 0 | 0 | 0 | 2 | 0 | 2 |

REPORT DATE: 06/12/81

ACTIVE LICENSING ACTIONS

NON-POWER REACTORS

FOR 05/01/81 - 05/31/81

| FACILITY | NEW ACTIONS FOR THIS REPORTING PERIOD | NEW ACTIONS TO DATE IN FY 81 | COMPLETED ACTIONS FOR THIS REPORTING PERIOD | COMPLETED ACTIONS TO DATE IN FY 81 | NUMBER OF ROUTINE ACTIONS | NUMBERS OF HEARINGS | TOTAL ACTIONS |
|----------------------|---------------------------------------|------------------------------|---|------------------------------------|---------------------------|---------------------|---------------|
| TEXAS A&M, TRIGA | 1 | 0 | 0 | 0 | 4 | 0 | 4 |
| U OF CAL IRV, TRIGA | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U OF CAL SANTA BAR L | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U OF CAL, BERK TRIGA | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U OF MICH PHOENIX ME | 0 | 0 | 0 | 0 | 3 | 0 | 3 |
| U. OF ARIZONA, TRIGA | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U. OF FLA., ARGONAUT | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| U. OF ILL., ADV TRIG | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U. OF ILLINOIS, LOPR | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| U. OF KANSAS | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| U. OF LOWELL | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U. OF MD., TRIGA | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U. OF MD. | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| U. OF MO., ROLLA | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| U. OF N. M., AGN-2-1 | 0 | 1 | 0 | 0 | 2 | 0 | 2 |
| U. OF TEXAS, TRIGA | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U. OF UTAH, AGN-201 | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| U. OF UTAH, TRIGA MA | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| U. OF VA. | 0 | 0 | 0 | 0 | 3 | 0 | 3 |
| U. OF VA., CAVALIER | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U. OF WASH. | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| U. OF WISC., TRIGA | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| U.S.G.S., TRIGA MARK | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| UCLA, ARGONAUT | 0 | 0 | 0 | 0 | 5 | 0 | 5 |
| UNION CARBIDE | 0 | 0 | 0 | 0 | 3 | 0 | 3 |
| VA HDSP., TRIGA | 0 | 0 | 0 | 0 | 3 | 0 | 3 |
| VA. POLY. INST., UTR | 0 | 0 | 0 | 0 | 3 | 0 | 3 |
| WASH. STATE, TRIGA | 0 | 0 | 0 | 0 | 2 | 0 | 2 |
| WEST., TRAINING REAC | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| WORCHESTER POLY TECH | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| GRAND TOTAL | 0 | 2 | 0 | 0 | 122 | 0 | 122 |

REPORT DATE: 06/13/81

MANPOWER SUMMARY BY FACILITY

FOR 05/01/81 - 05/31/81

| FACILITY | MANHOURS EXPENDED LAST MONTH ON | | | | MANHOURS EXPENDED TO DATE FY 1981 | AVG HOURS/COMPLETED ACTION FOR | | |
|------------------|---------------------------------|----------|------|-------|--|--------------------------------|-------------------|-------|
| | OR ROUTINE ACTIONS | HEARINGS | TMI | TOTAL | | GENERIC | PLANT SPECIFIC | TOTAL |
| ARKANSAS 1 | 178.3 | 0.0 | 3.0 | 181.3 | 2,239.8 | 56.4 | 26.3 | 37.7 |
| ARKANSAS 2 | 833.4 | 0.0 | 0.0 | 833.4 | 2,980.1 | 35.4 | 63.0 | 47.0 |
| BEAVER VALLEY 1 | 190.2 | 0.0 | 4.0 | 194.2 | 2,043.5 | 83.7 | 52.1 | 70.1 |
| BIG ROCK POINT 1 | 308.5 | 6.0 | 36.0 | 350.5 | 1,952.1 | 125.0 | 47.0 | 82.7 |
| BROWNS FERRY 1 | 160.8 | 0.0 | 0.0 | 160.8 | 1,031.3 | 31.0 | 44.7 | 41.0 |
| BROWNS FERRY 2 | 60.5 | 0.0 | 12.0 | 73.0 | 378.5 | 7.0 | 19.0 | 17.4 |
| BROWNS FERRY 3 | 241.3 | 0.0 | 0.0 | 241.3 | 1,335.8 | 4.5 | 40.0 | 35.0 |
| BRUNSWICK 1 | 145.1 | 0.0 | 0.0 | 145.1 | 772.7 | 13.7 | 13.3 | 13.6 |
| BRUNSWICK 2 | 106.5 | 0.0 | 0.0 | 106.5 | 561.5 | 9.5 | 19.8 | 14.1 |
| CALVERT CLIFFS 1 | 284.2 | 0.0 | 0.0 | 284.2 | 2,788.2 | 199.1 | 32.6 | 97.8 |
| CALVERT CLIFFS 2 | 74.0 | 0.0 | 0.0 | 74.0 | 641.0 | 49.6 | 37.5 | 43.5 |
| COOK 1 | 278.5 | 0.0 | 0.0 | 278.5 | 959.5 | 33.8 | 30.3 | 33.1 |
| COOK 2 | 158.0 | 0.0 | 0.0 | 158.0 | 984.0 | 29.4 | 63.0 | 41.2 |
| COOPER STATION | 164.6 | 0.0 | 18.0 | 182.9 | 978.4 | 256.5 | 64.5 | 153.2 |
| CRYSTAL RIVER 3 | 114.7 | 0.0 | 5.0 | 119.7 | 1,566.0 | 218.3 | 56.5 | 125.8 |
| DAVIS-BESSE 1 | 129.6 | 0.0 | 0.0 | 129.6 | 1,785.9 | 71.3 | 65.9 | 67.8 |
| DRESDEN 2 | 131.0 | 0.0 | 4.0 | 135.0 | 1,484.0 | 46.9 | 45.5 | 46.5 |
| DRESDEN 3 | 178.0 | 0.0 | 11.0 | 189.0 | 797.2 | 84.2 | 30.8 | 67.3 |
| DUANE ARNOLD | 85.0 | 0.0 | 0.0 | 85.0 | 710.0 | 91.5 | 19.2 | 58.6 |
| FARLEY 1 | 235.0 | 0.0 | 0.0 | 235.0 | 1,507.9 | 103.7 | 41.5 | 76.5 |
| FARLEY 2 | 80.3 | 0.0 | 0.0 | 80.3 | 276.8 | 0.0 | 0.0 | 0.0 |
| FITZPATRICK | 24.0 | 0.0 | 8.0 | 32.0 | 241.0 | 55.3 | 9.5 | 34.9 |
| FORT CALHOUN 1 | 33.0 | 0.0 | 0.0 | 33.0 | 878.9 | 117.6 | 23.5 | 107.2 |
| FORT ST VRAIN | 224.2 | 0.0 | 0.0 | 224.2 | 1,488.5 | 0.0 | 0.0 | 0.0 |
| GINNA | 338.9 | 0.0 | 0.0 | 338.9 | 1,425.0 | 44.6 | 30.6 | 38.9 |
| HADDAM NECK | 140.6 | 0.0 | 0.0 | 140.6 | 916.0 | 118.1 | 54.7 | 92.7 |
| HATCH 1 | 149.7 | 0.0 | 0.0 | 149.7 | 630.5 | 17.2 | 24.7 | 22.2 |
| HATCH 2 | 1.0 | 0.0 | 0.0 | 1.0 | 497.0 | 21.4 | 21.2 | 21.3 |
| INDIAN POINT 2 | 203.1 | 0.0 | 0.0 | 203.1 | 2,439.1 | 109.9 | 106.8 | 107.9 |
| INDIAN POINT 3 | 41.5 | 0.0 | 0.0 | 41.5 | 581.5 | 26.6 | 71.5 | 49.0 |
| KEWAUNEE | 247.8 | 0.0 | 2.0 | 249.8 | 1,759.3 | 40.7 | 58.3 | 48.7 |
| LA CROSSE | 23.0 | 0.0 | 0.0 | 23.0 | 1,315.5 | 47.8 | 322.5 | 147.7 |
| MAINE YANKEE | 283.5 | 0.0 | 1.0 | 284.5 | 2,252.8 | 57.8 | 113.3 | 74.8 |
| MCGUIRE 1 | 0.0 | 0.0 | 0.0 | 0.0 | 353.5 | 0.0 | 0.0 | 0.0 |
| MILLSTONE 1 | 200.7 | 0.0 | 0.0 | 200.7 | 1,751.7 | 65.1 | 97.1 | 81.8 |

TECHNICAL ASSIGNMENT CONTROL SYSTEM
MANPOWER SUMMARY BY FACILITY

REPORT DATE: 06/13/81

FOR 05/01/81 - 05/31/81

| FACILITY | MANHOURS EXPENDED LAST MONTH ON | | | | MANHOURS EXPENDED TO DATE FY 1981 | AVG HOURS/COMPLETED ACTION FOR | | |
|---------------------|---------------------------------|----------|------|-------|--|--------------------------------|-------------------|-------|
| | OR ROUTINE ACTIONS | HEARINGS | TMI | TOTAL | | GENERIC | PLANT SPECIFIC | TOTAL |
| MILLSTONE 2 | 225.4 | 0.0 | 0.0 | 225.4 | 1,271.6 | 84.5 | 148.9 | 119.9 |
| MONTICELLO | 89.2 | 0.0 | 0.0 | 89.2 | 1,762.0 | 81.4 | 162.0 | 118.0 |
| NINE MILE POINT 1 | 175.0 | 0.0 | 18.0 | 193.0 | 978.0 | 48.8 | 138.2 | 86.6 |
| NORTH ANNA 1 | 201.8 | 1.0 | 2.0 | 204.8 | 2,116.2 | 38.9 | 78.8 | 67.2 |
| NORTH ANNA 2 | 30.0 | 0.0 | 0.0 | 30.0 | 534.0 | 287.0 | 52.2 | 67.9 |
| OCONEE 1 | 231.7 | 0.0 | 88.0 | 319.7 | 2,941.2 | 44.4 | 67.6 | 58.0 |
| OCONEE 2 | 84.8 | 0.0 | 0.0 | 84.8 | 479.3 | 30.0 | 6.5 | 16.4 |
| OCONEE 3 | 47.0 | 0.0 | 0.0 | 47.0 | 543.0 | 17.4 | 11.1 | 13.6 |
| OYSTER CREEK 1 | 113.3 | 12.0 | 0.0 | 125.3 | 907.0 | 56.8 | 32.8 | 47.5 |
| PALISADES | 196.2 | 0.0 | 0.0 | 196.2 | 651.7 | 19.5 | 131.1 | 41.8 |
| PEACH BOTTOM 2 | 116.3 | 0.0 | 0.0 | 116.3 | 925.4 | 117.5 | 50.2 | 76.1 |
| PEACH BOTTOM 3 | 35.4 | 0.0 | 0.0 | 35.4 | 312.8 | 26.8 | 6.4 | 15.8 |
| PILGRIM 1 | 155.0 | 0.0 | 5.0 | 160.0 | 1,042.8 | 165.9 | 94.4 | 143.5 |
| POINT BEACH 1 | 226.5 | 0.0 | 2.0 | 228.5 | 1,199.6 | 74.5 | 30.8 | 55.4 |
| POINT BEACH 2 | 150.5 | 0.0 | 2.0 | 152.5 | 649.1 | 19.8 | 22.3 | 20.8 |
| PRAIRIE ISLAND 1 | 140.3 | 0.0 | 9.0 | 149.3 | 1,426.0 | 73.7 | 424.9 | 177.0 |
| PRAIRIE ISLAND 2 | 8.0 | 0.0 | 0.0 | 8.0 | 228.5 | 5.2 | 0.0 | 4.5 |
| QUAD CITIES 1 | 191.4 | 0.0 | 8.0 | 199.4 | 850.5 | 35.2 | 48.7 | 43.4 |
| QUAD CITIES 2 | 88.7 | 0.0 | 8.0 | 96.7 | 478.3 | 31.5 | 19.8 | 24.7 |
| RANCHO SECO 1 | 163.5 | 16.0 | 0.0 | 179.5 | 1,685.3 | 10.7 | 70.2 | 57.7 |
| ROBINSON 2 | 158.8 | 0.0 | 0.0 | 158.8 | 1,244.8 | 29.8 | 17.6 | 25.5 |
| SALEM 1 | 116.1 | 0.0 | 0.0 | 116.1 | 1,513.1 | 62.5 | 105.5 | 84.0 |
| SALEM 2 | 10.0 | 0.0 | 0.0 | 10.0 | 95.4 | 0.0 | 0.0 | 0.0 |
| SAN ONOFRE 1 | 608.3 | 17.0 | 0.0 | 625.3 | 3,684.6 | 57.6 | 54.6 | 56.1 |
| SEQUOYAH 1 | 94.0 | 0.0 | 0.0 | 94.0 | 391.5 | 0.0 | 0.0 | 0.0 |
| ST LUCIE 1 | 287.4 | 0.0 | 19.0 | 306.4 | 1,470.7 | 54.7 | 57.4 | 55.9 |
| SURRY 1 | 69.0 | 0.0 | 15.0 | 84.2 | 987.2 | 45.2 | 72.2 | 51.6 |
| SURRY 2 | 6.0 | 0.0 | 13.0 | 19.2 | 412.2 | 44.5 | 1.0 | 33.6 |
| THREE MILE ISLAND 1 | 492.7 | 0.0 | 0.0 | 492.7 | 2,306.7 | 169.2 | 73.4 | 108.2 |
| TROJAN | 209.0 | 26.5 | 0.0 | 235.5 | 1,843.0 | 56.2 | 111.6 | 86.0 |
| TURKEY POINT 3 | 136.3 | 160.5 | 0.0 | 297.3 | 3,604.9 | 63.6 | 238.2 | 147.1 |
| TURKEY POINT 4 | 6.5 | 3.0 | 0.0 | 10.0 | 1,150.2 | 45.6 | 67.8 | 56.2 |
| VERMONT YANKEE 1 | 184.0 | 0.0 | 0.0 | 184.0 | 1,364.2 | 122.5 | 60.9 | 85.6 |
| YANKEE-ROWE 1 | 110.9 | 0.0 | 0.0 | 110.9 | 1,053.0 | 166.4 | 12.0 | 142.0 |
| ZION 1 | 319.9 | 0.0 | 4.0 | 323.9 | 1,946.9 | 56.3 | 28.9 | 39.1 |

REPORT DATE: 06/13/81

MANPOWER SUMMARY BY FACILITY

FOR 05/01/81 - 05/31/81

| FACILITY | MANHOURS EXPENDED LAST MONTH ON | | | | MANHOURS EXPENDED TO DATE FY 1981 | AVG HOURS/COMPLETED ACTION FOR | | |
|-------------|---------------------------------|----------|-------|----------|--|--------------------------------|-------------------|-------|
| | OR ROUTINE ACTIONS | HEARINGS | TMI | TOTAL | | GENERIC | PLANT SPECIFIC | TOTAL |
| ZION 2 | 62.5 | 0.0 | 0.0 | 62.5 | 361.5 | 24.2 | 9.9 | 15.3 |
| GRAND TOTAL | 11,589.9 | 242.0 | 299.2 | 12,131.1 | 88,717.7 | 67.2 | 60.1 | 63.7 |

TECHNICAL ASSIGNMENT CONTROL SYSTEM
MULTI-PLANT ISSUES SUMMARY REPORT

06/12/81

(LEAD PROJECT MANAGER - TELEPHONE NUMBER)

| CODE | GENERIC SUBJECT | ACTIVE | FUTURE | COMPL. CURRENT FY (81) | COMPL. PRIOR FY | TOTAL |
|------|--|--------|--------|------------------------|-----------------|-------|
| A-01 | 10 CFR 50.55 A(G) - ISI (D. Chaney - X27110) | 44 | 0 | 11 | 14 | 69 |
| A-02 | APPENDIX I - ALARA (P. Wagner - X27072) | 64 | 0 | 0 | 1 | 65 |
| A-03 | SECURITY REVIEWS-MODIFIED AMENDMENT PLANS (J. Miller - X27014) | 0 | 0 | 3 | 66 | 69 |
| A-04 | APPENDIX J - CONTAINMENT LEAK TESTING (W. Paulson - X27214) | 51 | 0 | 6 | 3 | 60 |
| A-05 | GE MARK I CONTAINMENT TECH SPECS-SHORT TERM (B. Siegel - X29409) | 0 | 0 | 0 | 20 | 20 |
| A-06 | RESPIRATORY PROTECTION SYSTEM (J. Shea - X27231) | 1 | 0 | 0 | 11 | 12 |
| A-07 | APPENDIX G - FRACTURE TOUGHNESS (R. Caruso - X27232) | 0 | 0 | 1 | 8 | 9 |
| A-08 | ECCS EVALUATION-GENERIC PER 50.46 COMPLIANCE (D. Garner - X28430) | 2 | 0 | 0 | 6 | 8 |
| A-09 | PRESSURE VESSEL BELTLINE MATERIAL SURVEILLANCE (P. Erickson - X29564) | 3 | 0 | 16 | 46 | 65 |
| A-10 | CONTINGENCY PLANNING (J. Miller - X27014) | 9 | 0 | 44 | 11 | 64 |
| A-11 | GUARD TRAINING PLANS (J. Miller - X27014) | 33 | 0 | 32 | 1 | 66 |
| A-12 | VITAL AREA ANALYSIS (J. Miller - X27014) | 55 | 0 | 5 | 7 | 67 |
| A-13 | NON POWER REACTOR SAFEGUARDS PLANS (J. Miller - X27014) | 0 | 0 | 0 | 0 | 0 |
| A-14 | 10CFR 50.55 A(G) - INSERVICE TESTING (D. Chaney - X27110) | 59 | 0 | 1 | 1 | 61 |
| A-15 | QUALITY ASSURANCE REQUEST REGARDING DIESEL GENERATOR FUEL OIL (G. Vissing - X28136) | 3 | 0 | 2 | 30 | 35 |
| B-01 | DIESEL GENERATOR LOCKOUT (T. Wambach - X27038) | 7 | 0 | 6 | 13 | 26 |
| B-02 | FIRE PROTECTION (T. Wambach - X27038) | 3 | 0 | 2 | 62 | 67 |
| B-03 | PWR MODERATOR DILUTION (C. Nelson - X27563) | 7 | 0 | 3 | 31 | 41 |
| B-04 | REACTOR VESSEL OVERPRESSURE PROTECTION (D. Garner - X28430) | 15 | 0 | 1 | 22 | 38 |
| B-05 | STRESS CORROSION CRACKING - BWR RCS/B (D. Clark - X29797) | 2 | 0 | 3 | 18 | 23 |
| B-06 | BWR RELIEF VALVE (J. Hannon - X29796) | 1 | 0 | 1 | 21 | 23 |
| B-07 | STEAM GENERATOR FEEDWATER FLOW INSTABILITY (S. Nowicki - X27218) | 3 | 0 | 0 | 23 | 26 |
| B-08 | PWR HPSI-LPSI FLOW RESISTANCE (P. Wagner - X27072) | 0 | 0 | 1 | 15 | 16 |
| B-09 | CHARGING SYSTEMS PIPE VIBRATIONS (D. DiLanni - X27793) | 0 | 0 | 0 | 15 | 15 |
| B-10 | BURNABLE POISON ROD FAILURE - B&W (P. Erickson - X29564) | 0 | 0 | 0 | 2 | 2 |
| B-11 | FLOOD OF EQUIPMENT IMPORTANT TO SAFETY (D. Verrelli - X27110) | 2 | 0 | 5 | 3 | 10 |
| B-12 | STEAM GENERATOR TUBE INSPECTION (M. Fairtile - X29196) | 0 | 0 | 2 | 10 | 12 |
| B-13 | FUEL ROD BOW (W. Ross - X27134) | 1 | 0 | 0 | 7 | 8 |
| B-14 | CCA GUIDE TUBE WEAR (M. Conner - X27564) | 14 | 0 | 7 | 17 | 38 |
| B-15 | C-E POISON ROD GROWTH () | 1 | 0 | 0 | 1 | 2 |
| B-16 | EMERGENCY PLANNING AND REVISIONS () | 28 | 0 | 40 | 7 | 75 |
| B-17 | TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS (L. Engle - X28349) | 49 | 0 | 9 | 3 | 61 |
| B-18 | WORTHINGTON RHR PUMP SHAFT INTEGRITY (C. Nelson - 27563) | 0 | 0 | 0 | 2 | 2 |
| B-19 | NEUTRON SHIELDING - CE REACTORS (L. Engle - X28349) | 0 | 0 | 0 | 2 | 2 |
| B-20 | CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATION (E. Reeves - X27050) | 8 | 0 | 23 | 24 | 55 |
| B-21 | LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM (S. Nowicki - X27218) | 56 | 0 | 6 | 3 | 65 |
| B-22 | TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS (L. Engle - X28349) | 45 | 0 | 17 | 3 | 65 |
| B-23 | DEGRADED GRID VOLTAGE (S. Nowicki - X27218) | 34 | 0 | 7 | 21 | 62 |
| B-24 | VENTING AND PURGING CONTAINMENTS WHILE AT FULL POWER AND EFFECT ON LOCA (E. Reeves -) | 36 | 0 | 23 | 34 | 93 |
| B-25 | BWR FEEDWATER NOZZEL CRACKING (R. Snaider - X27876) | 5 | 0 | 6 | 12 | 23 |
| B-26 | INADVERTANT SAFETY INJECTION DURING COOLDOWN (R. Martin - 24727) | 1 | 0 | 1 | 38 | 40 |
| B-27 | REVIEW RESPONSES TO IE BULLETIN 78-03 (OFFGAS EXPLOSIONS) (J. Shea - X27231) | 0 | 0 | 3 | 23 | 26 |
| B-28 | BWR JET PUMP FLOW INDICATION ELIMINATION (R. Bevan - X29784) | 0 | 0 | 2 | 3 | 5 |
| B-29 | BWR FEEDWATER PUMP TRIP (P. O'Connor - X21215) | 0 | 0 | 0 | 4 | 4 |
| B-30 | STEAM GENERATOR REPLACEMENT PROGRAM (R. Martin - X24727) | 3 | 0 | 0 | 6 | 9 |
| B-31 | LONG SHAFT LHSI & OUTSIDE RECIRC. PUMP DEGRADATION (D. Neighbors - X27037) | 3 | 0 | 0 | 0 | 3 |
| B-32 | BLOCKED SI SIGNAL DURING COOLDOWN (M. Grotenhuis - X27128) | 4 | 0 | 0 | 1 | 5 |
| B-33 | IODINE SPIKING | 0 | 0 | 0 | 0 | 0 |
| B-34 | BWR-WELD FAILURE OF JET PUMP RETAINER BOLT (R. Bevan - X29784) | 0 | 0 | 0 | 0 | 0 |
| B-35 | ORIFICE ROD ASSEMBLY INTEGRITY - B&W (G. Vissing - X28136) | 0 | 0 | 0 | 6 | 6 |
| B-36 | RESISTANCE TEMPERATURE DETECTOR (RTD) RESPONSE - CE (M. Conner - X27564) | 8 | 0 | 0 | 0 | 8 |

06/12/81

MULTI-PLANT ISSUES SUMMARY REPORT
(LEAD PROJECT MANAGER - TELEPHONE NUMBER)

| CODE | GENERIC SUBJECT | ACTIVE | FUTURE | COMPL. CURRENT FY (81) | COMPL. PRIOR FY | TOTAL |
|------|---|--------|--------|------------------------|-----------------|-------|
| B-37 | STEAM GENERATOR TUBE DENTING AND SUPPORT PLATE MODIFICATIONS - CE (G.Requa - X28478) | 3 | 0 | 1 | 7 | 11 |
| B-38 | TENDON SURVEILLANCE - BECHTEL CONFINEMENTS (G. Viseing - X28136) | 3 | 0 | 0 | 1 | 4 |
| B-39 | PWR PRESSURE - TEMPERATURE LIMIT TECH SPECS (M. Fairtile - X29196) | 6 | 0 | 29 | 5 | 40 |
| B-40 | PIPE SUPPORT BASE PLATES (L. Olshan - X27144) | 0 | 0 | 1 | 0 | 1 |
| B-41 | FIRE PROTECTION - FINAL TECH SPECS (INCLUDES SER SUPPLEMENTS)(T. Wambach - X27038) | 36 | 0 | 34 | 10 | 80 |
| B-42 | TMI FOLLOW UP - ALL PLANTS (M. Fairtile - X29196) | 17 | 0 | 39 | 30 | 86 |
| B-43 | PWR FEEDWATER LINE CRACKS (S. Miner - X27266) | 33 | 0 | 8 | 30 | 71 |
| B-44 | LESSONS LEARNED IMPLEMENTATION (| 5 | 0 | 11 | 790 | 806 |
| B-45 | WASH 1400 EVENT V, "PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES"(P.Polk -X29777) | 5 | 0 | 38 | 23 | 66 |
| B-46 | ANALYSIS OF TURBINE DISC CRACKS (W. Ross - X27134) | 60 | 0 | 14 | 13 | 87 |
| B-47 | ECCS; CLAD SWELLING AND RUPTURE (L. Olshan - X27144) | 1 | 0 | 3 | 40 | 44 |
| B-48 | ADEQUACY OF STATION ELECTRIC DISTRIBUTION VOLTAGE (Lombardo - X27356) | 41 | 0 | 7 | 0 | 48 |
| B-49 | PWR CONTROL ROD MISALIGNMENT (P. Wagner - X27072) | 3 | 0 | 2 | 1 | 6 |
| B-50 | AUXILIARY FEEDWATER SYSTEM EVALUATION (G. Requa - X28478) | 0 | 0 | 0 | 0 | 0 |
| B-51 | EVALUATION OF BULLETIN 79-06 AND 79-08 (M. Fairtile - X29196) | 3 | 0 | 0 | 3 | 6 |
| B-52 | REVIEW OF SAFETY ASPECT OF INADVERTENT SAFETY ACTIONS DURING SUR. TEST(C.Trammell) | 2 | 0 | 25 | 1 | 28 |
| B-53 | LESSONS LEARNED CATEGORY B (ITEMS) | 2 | 0 | 1 | 2 | 5 |
| B-54 | LESSONS CATEGORY A TECH SPEC | 1 | 0 | 5 | 0 | 6 |
| B-55 | B&O REPORT ON BWRs (D. Verrelli - X27462) | 1 | 0 | 0 | 3 | 4 |
| B-56 | CONTROL RODS FAILURE TO INSERT. BWR (K Eccleston - X29799) | 5 | 0 | 0 | 0 | 5 |
| B-57 | DHR CAPABILITY (D. Garner - X28430) | 0 | 0 | 5 | 0 | 5 |
| B-58 | SDR CAPABILITY (J. Hannon - 29796) | 15 | 0 | 12 | 0 | 27 |
| B-59 | MASONRY WALL DESIGN (C.Trammell - X27070) | 63 | 0 | 2 | 0 | 65 |
| B-60 | ENVIRONMENTAL QUALIFICATION (M. Williams - X29798) | 69 | 0 | 0 | 1 | 70 |
| B-61 | LOSS OF NON CLASS IE I&C POWER (M. Fairtile - X29196) | 64 | 0 | 0 | 0 | 64 |
| B-62 | CLASS IE 120 VAC VITAL INST (M. Padovan - X29778) | 59 | 0 | 6 | 0 | 65 |
| B-63 | INTERIM PROCEDURES FOR SHORT TERM BLACKOUT (C. Nelson - X27563) | 64 | 0 | 0 | 0 | 64 |
| B-64 | B&C INDUCED FLUX ERRORS (M. Fairtile - X29196) | 8 | 0 | 0 | 0 | 8 |
| B-65 | SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN BWR SCRAM SYSTEM(J. Hannon - X29796) | 24 | 0 | 0 | 0 | 24 |
| C-01 | PWR SECONDARY WATER CHEMISTRY MONITORING REQUIREMENTS (R. Licciardo - X28362) | 14 | 0 | 5 | 19 | 38 |
| C-02 | BWR-RECIRC. PUMP TRIP (ATWS) (V. Rooney - X29787) | 0 | 0 | 0 | 23 | 23 |
| C-03 | QUALIFICATIONS OF RADIATION PROTECTION MANAGER (D. DiIanni - X27793) | 1 | 0 | 0 | 12 | 13 |
| C-04 | FILTER TECH SPECS (R. Caruso - X27232) | 8 | 0 | 1 | 6 | 15 |
| C-05 | CONVEKSION TO STANDARD TECH SPECS (P. Wagner - X27072) | 2 | 0 | 2 | 1 | 5 |
| C-06 | PUMP SUPPORT-LAMELLAR TEARING (R. Snaider - X27876) | 29 | 0 | 9 | 1 | 39 |
| C-07 | FUEL HANDLING ACCIDENT INSIDE CONTAINMENT (W. Paulson - X27214) | 8 | 0 | 1 | 28 | 37 |
| C-08 | BWR POST LOCA H2 CONTROL (J. Hannon - X29796) | 4 | 0 | 4 | 2 | 10 |
| C-09 | PWR AUX FW PUMPS (M. Grotenhuis - X27128) | 2 | 0 | 1 | 4 | 7 |
| C-10 | CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (G. Requa - X28478) | 53 | 0 | 10 | 6 | 69 |
| C-11 | RPS POWER SUPPLY (J. VanVliet - X29795) | 16 | 0 | 6 | 1 | 23 |
| C-12 | BORON SOLUBILITY DURING LONG TERM COOLING FOLLOWING LOCA (T. Colburn - 28129) | 5 | 0 | 1 | 0 | 6 |
| C-13 | LOSS OF OFFSITE POWER (Lombardo - X27356) | 0 | 0 | 0 | 2 | 2 |
| C-14 | AUXILIARY FEEDWATER SEISMIC QUALIFICATION (K. Herring - X28140) | 45 | 0 | 0 | 0 | 45 |
| D-01 | GE MARK I CONTAINMENT EVALUATION - LONG TERM (B. Siegel - X29409) | 18 | 0 | 4 | 0 | 22 |
| D-02 | ECCS ZIRC CLAD MODEL ERROR-COMPLIANCE WITH 10 CFR-46 (D. Neighbors - X27037) | 0 | 0 | 0 | 20 | 20 |
| D-03 | PRESSURIZER HEATUP RATE ERROR (D. Neighbors - X27037) | 0 | 0 | 0 | 15 | 15 |
| D-04 | PWR REACTOR VESSEL CAVITY SEAL RING MISSILE POTENTIAL (C. Trammell - X27070) | 4 | 0 | 2 | 12 | 18 |
| D-05 | PLANT UPI MODEL PROBLEM (R. Licciardo - X28362) | 6 | 0 | 0 | 0 | 6 |
| D-06 | PEAKING MODEL CHANGE FOR GE REACTOR CORE (M. Conner - X27564) | 0 | 0 | 0 | 6 | 6 |
| D-07 | BWR POWER LEVEL FOR RWM (R. Clark - X29797) | 0 | 0 | 0 | 3 | 3 |
| D-08 | DEFICIENCY IN CHEM ADDITION TO CONTAINMENT SPRAYS (R. Caruso - X27232) | 0 | 0 | 0 | 3 | 3 |

TECHNICAL ASSIGNMENT CONTROL SYSTEM
MULTI-PLANT ISSUES SUMMARY REPORT
(LEAD PROJECT MANAGER - TELEPHONE NUMBER)

06/12/81

| CODE | GENERIC SUBJECT | ACTIVE | FUTURE | COMPL. CURRENT FY (81) | COMPL. PRIOR FY | TOTAL |
|------|---|--------|--------|------------------------------|-----------------------|-------|
| D-09 | GE ECCS INPUT ERRORS (V. Rooney - X29787) | 0 | 0 | 0 | 1 | 1 |
| D-10 | ASYMMETRIC LOCA LOADS (J. Rajan - X29475) | 37 | 0 | 1 | 0 | 38 |
| D-11 | FISSION GAS RELEASE | 2 | 0 | 5 | 57 | 64 |
| D-12 | NON-JET PUMP BWR CORE SPRAY PERFORMANCE (P. Polk - X29777) | 3 | 0 | 1 | 1 | 5 |
| D-13 | B&W SMALL BREAK ERROR (M. Fairtile - X29196) | 0 | 0 | 0 | 7 | 7 |
| D-14 | REACTOR VESSEL WELD - WIRE DEFICIENCY ((M. Fairtile - 29196) | 0 | 0 | 1 | 10 | 11 |
| D-15 | HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE (M. Grotenhuis - 27128) | 44 | 0 | 14 | 4 | 62 |
| D-16 | REVIEW OF CORPORATE MANAGEMENT CAPABILITY (S. Miner - X27266) | 8 | 0 | 5 | 2 | 15 |
| D-17 | DEFINITION OF OPERABLE (S. Miner - X27266) | 31 | 0 | 24 | 5 | 60 |
| E-01 | SPENT FUEL POOL EXPANSIONS (R. Clark - X29797) | 5 | 0 | 7 | 29 | 41 |
| E-02 | FUEL CASK DROP (D. Neighbors - 27037) | 2 | 0 | 0 | 5 | 7 |
| E-03 | CORE RELOADS REQUIRING PRIOR NRC APPROVAL (W. Ross - 27134) | 1 | 0 | 4 | 29 | 34 |
| E-04 | BWR SINGLE LOOP OPERATION (R. Clark - X29797) | 8 | 0 | 1 | 2 | 11 |
| E-05 | W N-1 LOOP OPERATION (L. Olshan - X27144) | 3 | 0 | 3 | 2 | 8 |
| E-06 | CEA POSITION INDICATION FAILURES - CE (M. Conner - X27564) | 6 | 0 | 0 | 1 | 7 |
| E-07 | REACTOR PROTECTION SYSTEM LOGIC - CE (M. Conner - X27564) | 5 | 0 | 0 | 0 | 5 |
| F-01 | I.A.1.1 SHIFT TECHNICAL ADVISOR | 70 | 0 | 0 | 0 | 70 |
| F-02 | I.A.1.3 SHIFT MANNING | 68 | 0 | 1 | 0 | 69 |
| F-03 | I.A.2.1 UPGRADING OF RO AND SRO TRAINING | 69 | 0 | 1 | 0 | 70 |
| F-04 | I.C.1.2 INADEQUATE CORE COOLING GUIDELINES AND PROCEDURES | 70 | 0 | 0 | 0 | 70 |
| F-05 | I.C.1.3 ABNORMAL TRANSIENT OPERATOR GUIDELINES AND PROCEDURES | 70 | 0 | 0 | 0 | 70 |
| F-06 | I.C.5 FEEDBACK OF OPERATING EXPERIENCE | 0 | 0 | 0 | 0 | 0 |
| F-07 | I.C.6 CORRECT PERFORMANCE OF OPERATING ACTIVITIES | 0 | 0 | 0 | 0 | 0 |
| F-08 | I.D.1 CONTROL ROOM DESIGN REVIEW | 0 | 0 | 0 | 0 | 0 |
| F-09 | I.D.2 SAFETY PARAMETER DISPLAY SYSTEM | 0 | 0 | 0 | 0 | 0 |
| F-10 | II.B.1 RCS HIGH POINT VENTS | 67 | 0 | 4 | 0 | 71 |
| F-11 | II.B.2 PLANT SHIELDING | 0 | 0 | 0 | 0 | 0 |
| F-12 | II.B.3 POST ACCIDENT SAMPLING | 69 | 0 | 1 | 0 | 70 |
| F-13 | II.B.4 TRAINING FOR MITIGATING CORE DAMAGE | 70 | 0 | 0 | 0 | 70 |
| F-14 | II.D.1 RV AND SV TRAINING | 71 | 0 | 0 | 0 | 71 |
| F-15 | II.E.1.1 AFW SYSTEM EVALUATION | 59 | 0 | 1 | 0 | 60 |
| F-16 | II.E.1.2.1 AFW SYSTEM INITIATION | 43 | 0 | 3 | 0 | 46 |
| F-17 | II.E.1.2.2 AFW SYSTEM FLOW INDICATION | 43 | 0 | 1 | 0 | 44 |
| F-18 | II.E.4.1 DEDICATED HYDROGEN PENETRATION | 70 | 0 | 0 | 0 | 70 |
| F-19 | II.E.4.2 CONTAINMENT ISOLATION DEPENDABILITY | 69 | 0 | 1 | 0 | 70 |
| F-20 | II.F.1.1 NOBLE GAS MONITOR | 70 | 0 | 0 | 0 | 70 |
| F-21 | II.F.1.2 IODINE/PARTICULATE SAMPLING | 70 | 0 | 0 | 0 | 70 |
| F-22 | II.F.1.3 CONTAINMENT HIGH RANGE MONITOR | 70 | 0 | 0 | 0 | 70 |
| F-23 | II.F.1.4 CONTAINMENT PRESSURE INSTRUMENT | 0 | 0 | 0 | 0 | 0 |
| F-24 | II.F.1.5 CONTAINMENT WATER LEVEL INSTRUMENT | 0 | 0 | 0 | 0 | 0 |
| F-25 | II.F.1.6 CONTAINMENT HYDROGEN MONITOR | 0 | 0 | 0 | 0 | 0 |
| F-26 | II.F.2 INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING | 68 | 0 | 2 | 0 | 70 |
| F-27 | II.K.2.9 FMEA ON ICS | 3 | 0 | 0 | 0 | 3 |
| F-28 | II.K.2.10 SAFETY GRADE ARTS | 7 | 0 | 0 | 0 | 7 |
| F-29 | II.K.2.11 CONTINUED OPERATOR TRAINING AND DRILLING | 7 | 0 | 0 | 0 | 7 |
| F-30 | II.K.2.13 THERMAL-MECHANICAL REPORT | 7 | 0 | 3 | 0 | 10 |
| F-31 | II.K.2.14 LIFT FREQUENCY OF PORVS AND SVS | 8 | 0 | 0 | 0 | 8 |
| F-32 | II.K.2.16 RCP SEAL DAMAGE | 7 | 0 | 0 | 0 | 7 |
| F-33 | II.K.2.17 POTENTIAL FOR VOIDING IN RCS | 7 | 0 | 0 | 0 | 7 |
| F-35 | II.K.2.20 SYSTEM RESPONSE TO SB LOCA | 7 | 0 | 0 | 0 | 7 |
| F-36 | II.K.3.1 AUTO PORV ISOLATION | 44 | 0 | 0 | 0 | 44 |

TECHNICAL ASSIGNMENT CONTROL SYSTEM
MULTI-PLANT ISSUES SUMMARY REPORT

06/12/81

| CODE | GENERIC SUBJECT | ACTIVE | FUTURE | COMPL. CURRENT FY (81) | COMPL. PRIOR FY | TOTAL |
|----------------------|---|--------|--------|------------------------------|-----------------------|-------|
| F-37 | II.K.3.2 REPORT ON PORV FAILURES | 43 | 0 | 1 | 0 | 44 |
| F-38 | II.K.3.3 REPORTING SV AND RV FAILURES AND CHALLENGES | 69 | 0 | 1 | 0 | 70 |
| F-39 | II.K.3.5 AUTO TRIP OF RCPS | 43 | 0 | 0 | 0 | 43 |
| F-40 | II.K.3.9 PID CONTROLLER | 27 | 0 | 1 | 0 | 28 |
| F-41 | II.K.3.10 ANTICIPATORY TRIP MODIFICATIONS | 28 | 0 | 0 | 0 | 28 |
| F-42 | II.K.3.12 ANTICIPATORY TRIP ON TURBINE TRIP | 27 | 0 | 1 | 0 | 28 |
| F-43 | II.K.3.13 HPCI AND RCIC INITIATION LEVELS | 23 | 0 | 0 | 0 | 23 |
| F-44 | II.K.3.14 ISOLATION CONDENSER ISOLATION MODIFICATION | 4 | 0 | 0 | 0 | 4 |
| F-45 | II.K.3.15 ISOLATION OF HPCI AND RCIC MODIFICATIONS | 23 | 0 | 0 | 0 | 23 |
| F-46 | II.K.3.16 CHALLENGES AND FAILURES OF RELIEF VALVES | 23 | 0 | 0 | 0 | 23 |
| F-47 | II.K.3.17 ECCS OUTAGES | 71 | 0 | 0 | 0 | 71 |
| F-48 | II.K.3.18 ADS ACTUATION | 23 | 0 | 0 | 0 | 23 |
| F-49 | II.K.3.19 INTERLOCK RECIRCULATION PUMP MODIFICATION | 23 | 0 | 0 | 0 | 23 |
| F-50 | II.K.3.21 RESTART OF CSS AND LPCI | 23 | 0 | 0 | 0 | 23 |
| F-51 | II.K.3.22 RCIC SUCTION | 0 | 0 | 0 | 0 | 0 |
| F-52 | II.K.3.24 SPACE COOLING FOR HPCI/RCIC MODIFICATION | 0 | 0 | 0 | 0 | 0 |
| F-53 | II.K.3.25 POWER ON PUMP SEALS | 23 | 0 | 0 | 0 | 23 |
| F-54 | II.K.3.27 COMMON REFERENCE LEVEL | 23 | 0 | 0 | 0 | 23 |
| F-55 | II.K.3.28 QUALIFICATION OF ADS ACCUMULATORS | 0 | 0 | 0 | 0 | 0 |
| F-56 | II.K.3.29 PERFORMANCE OF ISOLATION CONDENSERS | 4 | 0 | 0 | 0 | 4 |
| F-57 | II.K.3.30 SB LOCA METHODS | 70 | 0 | 0 | 0 | 70 |
| F-58 | II.K.3.31 COMPLIANCE WITH 10 CFR 50.46 | 0 | 0 | 0 | 0 | 0 |
| F-59 | II.K.3.44 ANTICIPATED TRANSIENTS WITH SINGLE FAILURES | 23 | 0 | 0 | 0 | 23 |
| F-60 | II.K.3.45 MANUAL DEPRESSURIZATION | 23 | 0 | 0 | 0 | 23 |
| F-61 | II.K.3.46 MICHELSON CONCERNS | 0 | 0 | 0 | 0 | 0 |
| F-62 | II.K.3.57 MANUAL ACTUATION OF ADS | 0 | 0 | 0 | 0 | 0 |
| F-63 | III.A.1.2 TECHNICAL SUPPORT CENTER | 71 | 0 | 0 | 0 | 71 |
| F-64 | III.A.1.2 OPERATIONAL SUPPORT CENTER | 71 | 0 | 0 | 0 | 71 |
| F-65 | III.A.1.2 EMERGENCY OPERATIONS FACILITY | 71 | 0 | 0 | 0 | 71 |
| F-66 | III.A.1.2 NUCLEAR DATA LINK | 71 | 0 | 0 | 0 | 71 |
| F-67 | III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE | 71 | 0 | 0 | 0 | 71 |
| F-68 | III.A.2.2 METEOROLOGICAL DATA UPGRADE | 70 | 0 | 0 | 0 | 70 |
| F-69 | III.D.3.3 IMPLANT RADIATION MONITORING | 70 | 0 | 0 | 0 | 70 |
| F-70 | III.D.3.4 CONTROL ROOM HABITABILITY | 70 | 0 | 0 | 0 | 70 |
| TOTAL GENERIC ISSUES | | 4,087 | 0 | 666 | 1,956 | 6,709 |

MULTI-PLANT ISSUES RECEIVING
DIVISION OF LICENSING
CONTRACT TECHNICAL ASSISTANCE

I-1

| <u>Code</u> | <u>Generic Subject</u> | <u>Contractor</u> | <u>DL Contract Manager</u> | <u>Tech. Monitor & Div.</u> |
|-------------|---|-----------------------|----------------------------|---------------------------------|
| A-01 | 10 CFR 50.55 A(G) - ISI | EG&G,PNW,BH FRC II | Donohew | C. Willis, DSI |
| A-02 | Appendix I - ALARA | | | |
| A-03 | Security Reviews - Modified Amendment Plans | FRC I | Butcher | Y. Huang, DSI |
| A-04 | Appendix J - Containment Leak Testing | | | |
| A-05 | GE Mark I Containment Tech Specs - Short Term | FRC I | Butcher | W. Koo, DL |
| A-06 | Respiratory Protection System | | | |
| A-07 | Appendix G - Fracture Toughness | FRC I | Butcher | W. Koo, DL |
| A-08 | ECCS Evaluation - Generic Per 50.46 Compliance | | | |
| A-09 | Pressure Vessel Beltline Material Surveillance | FRC I | Butcher | W. Koo, DL |
| A-10 | Contingency Planning | | | |
| A-11 | Guard Training Plans | FRC I | Butcher | W. Koo, DL |
| A-12 | Vital Area Analysis | | | |
| A-13 | Non Power Reactor Safeguards Plans | FRC I | Butcher | W. Koo, DL |
| A-14 | 10 CFR 50.55 A(G) - Inservice Testing | | | |
| A-15 | Quality Assurance Request Regarding Diesel Generator Fuel Oil | LLNL | Shemanski | Kendall, DSI |
| B-01 | Diesel Generator Lockout | | | |
| B-02 | Fire Protection | FRC II | Donohew | C. Nelson, DL |
| B-03 | PWR Moderator Dilution | | | |
| B-04 | Reactor Vessel Overpressure Protection | LLNL | Shemanski | Loomis, DSI |
| B-05 | Stress Corrosion Cracking - BWR RCSPB | LLNL | Shemanski | Loomis, DSI |
| B-06 | BWR Relief Valve | | | |
| B-07 | Steam Generator Feedwater Flow Instability | LLNL | Shemanski | Shemanski, DL |
| B-08 | PWR HPSI-LPSI Flow Resistance | | | |
| B-09 | Charging Systems Pipe Vibrations | LLNL | Shemanski | Shemanski, DL |
| B-10 | Burnable Poison Rod Failure - B&W | | | |
| B-11 | Flood of Equipment Important to Safety | LLNL | Shemanski | Shemanski, DL |
| B-12 | Steam Generator Tube Inspection | | | |
| B-13 | Fuel Rod Bow | LLNL | Shemanski | Shemanski, DL |
| B-14 | CEA Guide Tube Wear | | | |
| B-15 | C-E Poison Rod Growth | LLNL | Shemanski | Shemanski, DL |
| B-16 | Emergency Planning and Revisions | | | |
| B-17 | Tech Specs Surveillance for Hydraulic Snubbers | LLNL | Shemanski | Shemanski, DL |
| B-18 | Worthington RHR Pump Shaft Integrity | | | |
| B-19 | Neutron Shielding - CE Reactors | LLNL | Shemanski | Shemanski, DL |
| B-20 | Containment Leakage Due to Seal Deterioration | | | |
| B-21 | Loss of 125-V Bus Voltage with Loss of Annunciator System | LLNL | Shemanski | Shemanski, DL |
| B-22 | Tech Spec Surveillance Requirements for Mechanical Snubber | | | |

| <u>Code</u> | <u>Generic Subject</u> | <u>Contractor</u> | <u>DL Contract Manager</u> | <u>Tech. Monitor & Div.</u> |
|-------------|---|-------------------|----------------------------|---------------------------------|
| B-23 | Degraded Grid Voltage | LLNL/INEL | Shemanski | Saeed, DSI |
| B-24 | Venting and Purging Containments While at Full Power and Effect on LOCA | LLNL/INEL/FRC I | Shemanski/Butcher | Beard, DL |
| B-25 | BWR Feedwater Nozzle Cracking | FRC I | Butcher | J. Fair, DL |
| B-26 | Inadvertent Safety Injection During Cooldown | INEL | Shemanski | ICSB, DSI |
| B-27 | Review Responses to IE Bulletin 78-03 (Offgas Explosions) | | | |
| B-28 | BWR Jet Pump Flow Indication Elimination | | | |
| B-29 | BWR Feedwater Pump Trip | | | |
| B-30 | Steam Generator Replacement Program | | | |
| B-31 | Long Shaft LHSI & Outside Recirc. Pump Degradation | | | |
| B-32 | Blocked SI Signal During Cooldown | INEL | Shemanski | ICSB, DSI |
| B-33 | Iodine Spiking | | | |
| B-34 | BWR-Weld Failure of Jet Pump Retainer Bolt | | | |
| B-35 | Orifice Rod Assembly Integrity - B&W | | | |
| B-36 | Resistance Temperature Detector (RTD) Response - CE | | | |
| B-37 | Steam Generator Tube Denting and Support Plate Modifications - CE | | | |
| B-38 | Tendon Surveillance - Bechtel Containments | | | |
| B-39 | PWR Pressure - Temperature Limit Tech Soecs | | | |
| B-40 | Pipe Support Base Plates | | | |
| B-41 | Fire Protection - Final Tech Soecs (Includes SER Supplements) | | | |
| B-42 | TMI Follow Up - All Plants | | | |
| B-43 | PWR Feedwater Line Cracks | | | |
| B-44 | Lessons Learned Implementation | | | |
| B-45 | WASH 1400 Event V, "Primary Coolant System Pressure Isolation Valves" | FRC I | Butcher | P. Polk, DL |
| B-46 | Analysis of Turbine Disc Cracks | | | |
| B-47 | ECCS; Clad Swelling and Rupture | | | |
| B-48 | Adequacy of Station Electric Distribution Voltage | INEL | Shemanski | Saeed, DSI |
| B-49 | PWR Control Rod Misalignment | | | |
| B-50 | Auxiliary Feedwater System Evaluation | | | |
| B-51 | Evaluation of Bulletin 79-06 and 79-08 | | | |
| B-52 | Review of Safety Aspect of Inadvertent Safety Actions During Sur. Test | | | |
| B-53 | Lessons Learned Category B (Items) | | | |
| B-54 | Lessons Category A Tech Spec | | | |
| B-55 | B&O Report on BWRs | | | |
| B-56 | Control Rods Failure to Insert. BWR | FRC II | Donohew | J. Hannon, DL |
| B-57 | DHR Capability | INEL | Shemanski | RSB, DSI |
| B-58 | SDR Capability | | | |

| <u>Code</u> | <u>Generic Subject</u> | <u>Contractor</u> | <u>DL Contract Manager</u> | <u>Tech. Monitor & Div.</u> |
|-------------|--|-------------------|--------------------------------|-------------------------------------|
| B-59 | Masonry Wall Design | | | |
| B-60 | Environmental Qualification | FRC I | Butcher | T. Lee, DE |
| B-61 | Loss of Non Class IE I&C Power | LLNL | Shemanski | Wilson, DSI |
| B-62 | Class IE 120 VAC Vital Inst. | | | |
| B-63 | Interim Procedures for Short Term Blackout | | | |
| B-64 | B&C Induced Flux Errors | | | |
| B-65 | Safety Concerns Associated with Pipe Breaks in BWR Scram System | | | |
| C-01 | PWR Secondary Water Chemistry Monitoring Requirements | | | |
| C-02 | BWR-Recirc. Pump Trip (ATWS) | LLNL | Shemanski | Shemanski, DL |
| C-03 | Qualifications of Radiation Protection Manager | | | |
| C-04 | Filter Tech Specs | | | |
| C-05 | Conversion to Standard Tech Specs | | | |
| C-06 | Pump Support - Lamellar Tearing | | | |
| C-07 | Fuel Handling Accident Inside Containment | | | |
| C-08 | BWR Post-LOCA H2 Control | | | |
| C-09 | PWR Aux FW Pumps | | | |
| C-10 | Control of Heavy Loads Over Spent Fuel Pool | FRC I | Butcher | F. Clemenson, DSI |
| C-11 | RPS Power Supply | LLNL | Shemanski | Ahmed, DL |
| C-12 | Boron Solubility During Long Term Cooling Following LOCA | | | |
| C-13 | Loss of Offsite Power | | | |
| C-14 | Auxiliary Feedwater Seismic Qualification | | | |
| D-01 | GE Mark I Containment Evaluation - Long Term | | | |
| D-02 | ECCS Zirc Clad Model Error - Compliance with 10 CFR-46 | | | |
| D-03 | Pressurizer Heatup Rate Error | | | |
| D-04 | PWR Reactor Vessel Cavity Seal Ring Missile Potential | | | |
| D-05 | Plant UPI Model Problem | | | |
| D-06 | Peaking Model Change for CE Reactor Core | | | |
| D-07 | BWR Power Level for RWM | | | |
| D-08 | Deficiency in Chem Addition to Containment Sprays | | | |
| D-09 | GE ECCS Input Errors | | | |
| D-10 | Asymmetric LOCA Loads | | | |
| D-11 | Fission Gas Release | | | |
| D-12 | Non-Jet Pump BWR Core Spray Performance | | | |
| D-13 | B&W Small Break Error | | | |
| D-14 | Reactor Vessel Weld - Wire Deficiency | | | |
| D-15 | High Energy Line Break & Consequential System Failure | | | |
| D-16 | Review of Corporate Management Capability | | | |

| <u>Code</u> | <u>Generic Subject</u> | <u>Contractor</u> | <u>DL Contract Manager</u> | <u>Tech. Monitor & Div.</u> |
|-------------|---|-------------------|----------------------------|---------------------------------|
| D-17 | Definition of Operable | | | |
| E-01 | Spent Fuel Pool Expansions | | | |
| E-02 | Fuel Cask Drop | | | |
| E-03 | Core Reloads Requiring Prior NRC Approval | | | |
| E-04 | B:R Single Loop Operation | LLNL | Shemanski | RSB, DSI |
| E-05 | W N-1 Loop Operation | LLNL | Shemanski | RSB, DSI |
| E-06 | CEA Position Indication Failures - CE | | | |
| E-07 | Reactor Protection System Logic - CE | LLNL | Shemanski | Kendall, DSI |
| F-01 | I.A.1.1 Shift Technical Advisor | | | |
| F-02 | I.A.1.3 Shift Manning | | | |
| F-03 | I.A.2.1 Upgrading of RO and SRO Training | | | |
| F-04 | I.C.1.2 Inadequate Core Cooling Guidelines and Procedures | | | |
| F-05 | I.C.1.3 Abnormal Transient Operator Guidelines & Procedures | | | |
| F-06 | I.C.5 Feedback of Operating Experience | | | |
| F-07 | I.C.6 Correct Performance of Operating Activities | | | |
| F-08 | I.D.1 Control Room Design Review | | | |
| F-09 | I.D.2 Safety Parameter Display System | | | |
| F-10 | II.B.1 RCS High Point Vents | LLNL | Donohew | Aberthal, DSI |
| F-11 | II.B.2 Plant Shielding | EG&G | Donohew | F. Skopec, DSI |
| F-12 | II.B.3 Post Accident Sampling | Exxon | Donohew | P. Matthews, DE |
| F-13 | II.B.4 Training for Mitigating Core Damage | | | |
| F-14 | II.D.1 RV and SV Training | | | |
| F-15 | II.E.1.1 AFW System Evaluation | | | |
| F-16 | II.E.1.2.1 AFW System Initiation | FRC I | Butcher | R. Kendall, DSI |
| F-17 | II.E.1.2.2 AFW System Flow Indication | FRC I | Butcher | R. Kendall, DSI |
| F-18 | II.E.4.1 Dedicated Hydrogen Penetration | | | |
| F-19 | II.E.4.2 Containment Isolation Dependability | LLNL | Donohew | M. Fields, DSI |
| F-20 | II.F.1.1 Noble Gas Monitor | Exxon | Donohew | R. Bangert, DSI |
| F-21 | II.F.1.2 Iodine/Particulate Sampling | Exxon | Donohew | R. Bangert, DSI |
| F-22 | II.F.1.3 Containment High Range Monitor | Exxon | Donohew | F. Skopec, DSI |
| F-23 | II.F.1.4 Containment Pressure Instrument | LLNL* | Donohew | CSB, DSI |
| F-24 | II.F.1.5 Containment Water Level Instrument | LLNL* | Donohew | CSB, DSI |
| F-25 | II.F.1.6 Containment Hydrogen Monitor | LLNL* | Donohew | CSB, DSI |
| F-26 | II.F.2 Instruments for Detection on Inadequate Core Cooling | | | |
| F-27 | II.K.2.9 FMEA on ICS | | | |

| Code | Generic Subject | Contractor | DL Contract Manager | Tech Monitor & Div. |
|------|---|------------|---------------------|-----------------------|
| F-28 | II.K.2.1 Safety Grade Arts | EG&G | Donohew | J. Guttman, DSI |
| F-29 | II.K.2.1.1 Continued Operator Training and Drilling | | | |
| F-30 | II.K.2.1.3 Thermal-Mechanical Report | | | |
| F-31 | II.K.2.1.4 Lift Frequency of PORVs and SVS | | | |
| F-32 | II.K.2.1.6 RCP Seal Damage | | | |
| F-33 | II.K.2.1.7 Potential for Voiding in RCS | | | |
| F-35 | II.K.2.2.0 System Response to SB LOCA | | | |
| F-36 | II.K.3.1 Auto PORV Isolation | | | |
| F-37 | II.K.3.2 Report on PORV Failures | | | |
| F-38 | II.K.3.3 Reporting SV and RV Failures and Challenges | | | |
| F-39 | II.K.3.5 Auto Trip of RCPS | | | |
| F-40 | II.K.3.9 Pid Controller | | | |
| F-41 | II.K.3.10 Anticipatory Trip Modifications | | | |
| F-42 | II.K.3.12 Anticipatory Trip on Turbine Trip | | | |
| F-43 | II.K.3.13 HPCI and RCIC Initiation Levels | | | |
| F-44 | II.K.3.14 Isolation Condenser Isolation Modification | | | |
| F-45 | II.K.3.15 Isolation of HCPI and RCIC Modifications | | | |
| F-46 | II.K.3.16 Challenges and Failures of Relief Valves | | | |
| F-47 | II.K.3.17 ECCS Outages | | | |
| F-48 | II.K.3.18 ADS Actuation | | | |
| F-49 | II.K.3.19 Interlock Recirculation Pump Modification | EG&G | Donohew | D. Thatcher, DSI |
| F-50 | II.K.3.21 Restart of CSS and LPCI | | | |
| F-51 | II.K.3.22 RCIC Suction | | | |
| F-52 | II.K.3.24 Space Cooling for HPCI/RCIC Modification | | | |
| F-53 | II.K.3.25 Power on Pump Seals | | | |
| F-54 | II.K.3.27 Common Reference Level | EG&G | Donohew | A. Rooney-Smith, DHFS |
| F-55 | II.K.3.28 Qualification of ADS Accumulators | | | |
| F-56 | II.K.3.29 Performance of Isolation Condensers | | | |
| F-57 | II.K.3.30 SB LOCA Methods | EG&G | Donohew | B. Sherron, DSI |
| F-58 | II.K.3.31 Compliance with 10 CFR 50.46 | | | |
| F-59 | II.K.3.44 Anticipated Transients with Single Failures | | | |
| F-60 | II.K.3.45 Manual Depressurization | | | |
| F-61 | II.K.3.46 Michelson Concerns | EG&G | Donohew | W. Hodges, DSI |
| F-62 | II.K.3.57 Manual Actuation of ADS | | | |

| <u>Code</u> | <u>Generic Subject</u> | <u>Contractor</u> | <u>DL Contract Manager</u> | <u>Tech Monitor & Div.</u> |
|-------------|---|-------------------|----------------------------|--------------------------------|
| F-63 | III.A.1.2 Technical Support Center | PNL | Donohew | S. Ramos, OIE |
| F-64 | III.A.1.2 Operational Support Center | PNL | Donohew | S. Ramos, OIE |
| F-65 | III.A.1.2 Emergency Operations Facility | PNL | Donohew | S. Ramos, OIE |
| F-66 | III.A.1.2 Nuclear Data Link | | | |
| F-67 | III.A.2.1 Emergency Plan Upgrade to Meet Rule | | | |
| F-68 | III.A.2.2 Meteorological Data Upgrade | | | |
| F-69 | III.D.3.3 Implant Radiation Monitoring | | | |
| F-70 | III.D.3.4 Control Room Habitability | Exxon | Donohew | F. Skopec, DSI |

REPORT DATE: 06/12/81

HEARING SUMMARY

FOR 05/01/81 - 05/31/81

| FACILITY | ATTORNEY | TITLE | INITIATION DATE | LIC. ACT. COMPLETE | PROJECT MANAGER |
|------------------|-------------|---|-----------------|--------------------|-----------------|
| BIG ROCK POINT 1 | M. MULKEY | BIG ROCK POINT - SPENT FUEL POOL HEARING | 10/02/79 | | W. PAULSON |
| FARLEY 2 | D. SWANSON | DESIGN OF DNSITE AUXILIARY POWER SYSTEM FOR FARLEY UNIT 2 | | | J. THOMA |
| FORT ST VRAIN | R. BLACK | FORT ST VRAIN - DECALIBRATION OF EX-CORE NEUTRON DETECTORS | | | G. KUZMYCZ |
| HATCH 1 | B. SMITH | HATCH 1 - EVALUATION OF IMPACT ON SHORT NOSE STURGEON STURGEON | 11/08/79 | | M. FAIRTILE |
| HATCH 2 | B. SMITH | HATCH 2 - EVALUATION OF IMPACT ON SHORT NOSE STURGEON STURGEON | 11/08/79 | | M. FAIRTILE |
| LA CROSSE | C. WOODHEAD | LACBWR-SUMMARY DISPOSITION AFFADAVITS NO. 8 | 04/30/80 | | R. CARUSO |
| LA CROSSE | C. WOODHEAD | LACBWR-SUMMARY DISPOSITION AFFADAVITS NO 9 | 04/30/80 | | R. CARUSO |
| NORTH ANNA 1 | D. SWANSON | NORTH ANNA 1&2 - RECONVENING OF APPEALS BOARD HEARINGS | | | L. ENGLE |
| NORTH ANNA 2 | D. SWANSON | NORTH ANNA 2 - REV CHANGES RE NORTH ANNA UNIT 2 STARTUP PHYSICS TESTING PROGRAM | | 02/02/81 | L. ENGLE |
| OYSTER CREEK 1 | C. WOODHEAD | OYSTER CREEK - OPERATIONAL QUALITY ASSURANCE PLAN | 06/30/78 | | W. PAULSON |
| PALISADES | C. BARTH | PALISADES - STEAM GENERATOR REPLACEMENT HEARING | 07/31/79 | | T. WAMBACH |
| PALISADES | C. BARTH | PALISADES - INSTALLATION SCHEDULE FOR ALTERNATE SHUTDOWN PANAL | 09/26/79 | 02/10/81 | T. WAMBACH |
| RANCHO SECO 1 | M. MULKEY | RANCHO SECO - PUBLIC HEARINGS ON COMMISSION ORDER OF MAY 7, 1979 | 05/15/79 | | M. PADOVAN |
| SALEM 1 | B. SMITH | SALEM 1 - SPENT FUEL POOL MOD HEARING EFFORT | 03/07/78 | 02/02/81 | W. ROSS |
| SAN ONDFRE 1 | L. CHANDLER | SAN ONDFRE - EVACUATION ISSUE IN 10CFR 2.206 REQUEST REQUEST | 11/14/79 | | S. NOWICKI |
| SAN ONDFRE 1 | L. CHANDLER | SAN ONDFRE 1 - SEISMIC ISSUE IN 10CFR 2.206 REQUEST REQUEST | 11/14/79 | | S. NOWICKI |
| TROJAN | J. GRAY | TROJAN - CONTROL BUILDING HEARING FOLLOW-UP WORK | 04/18/80 | 02/13/81 | C. TRAMMELL |
| TURKEY POINT 3 | J. GOLDBERG | TURKEY POINT 3 - HEARING RE TURKEY POINT 3 STEAM GENERATOR REPAIR | 08/07/79 | | M. GROTENHAUS |

REPORT DATE: 06/12/81

HEARING SUMMARY

FOR 05/01/81 - 05/31/81

| FACILITY | ATTORNEY | TITLE | INITIATION DATE | LIC. ACT. COMPLETE | PROJECT MANAGER |
|----------------|-------------|--|--------------------|-----------------------|--------------------|
| TURKEY POINT 4 | J. GOLDBERG | TURKEY POINT 4 - HEARING RE TURKEY POINT 4 STEAM GENERATOR REPAIR | 08/07/79 | | M. GROTENHUIS |

FISCAL YEAR: 1981

COMPLETED LICENSING ACTIONS BY FEE CLASS

REPORT DATE: 06/12/81

| FEE CLASS | COMPLETED ACTIONS | | MANHOURS | | AVG HOURS/ACTION | | AVG COST/ACTION | |
|-----------|-------------------|---------------|----------------|------------------|------------------|------------|-----------------|---------------------|
| | MAR. 1981 | FY TO DATE | TOTAL FY 81 | TOTAL TO DATE | FOR FY 81 | TO DATE | FIXED BY REG | ACTUAL @ \$70/HR |
| NONE | 386 | 999 | 23,634.1 | 66,327.1 | 23.6 | 66.3 | N/A | \$4,641 |
| 1 | 29 | 68 | 725.3 | 1,689.3 | 10.6 | 24.8 | 400 | \$1,736 |
| 2 | 17 | 39 | 800.8 | 1,213.3 | 20.5 | 31.1 | 1,200 | \$2,177 |
| 3 | 62 | 135 | 4,786.3 | 7,236.2 | 35.4 | 53.6 | 4,000 | \$3,752 |
| 4 | 10 | 20 | 1,714.5 | 3,869.0 | 85.7 | 193.4 | 12,300 | \$13,538 |
| 5 | 0 | 0 | 0.0 | 0.0 | 0.0 | 0.0 | 25,800 | \$0 |
| 6 | 0 | 0 | 0.0 | 0.0 | 0.0 | 0.0 | 45,900 | \$0 |
| TOTAL | 504 | 1,261 | 31,661.0 | 80,334.9 | 25.1 | 63.7 | | \$25,844 |

REPORT DATE: 06/12/81

FY TO DATE MULTI-PLANT & PLANT SPECIFIC ACTION SUMMARY

10/01/80 - 05/31/81

| | <u>NEW LICENSING ACTIONS</u> | <u>COMPLETED LICENSING ACTIONS</u> | <u>ACTIVE LICENSING ACTIONS</u> |
|-------------------------------|--------------------------------------|--|---|
| <u>MULTI-PLANT ACTIONS</u> | | | |
| REGULATIONS | 0 | 121 | 324 |
| OPERATING EXPERIENCE | 137 | 396 | 943 |
| INTERNAL REVIEWS | 47 | 40 | 187 |
| NEW INFORMATION | 43 | 57 | 153 |
| LICENSE REQUIREMENTS | 2 | 12 | 33 |
| TMI ACTIONS | 76 | 6 | 2,480 |
| I&E TRANSFERS | 0 | 0 | 0 |
| SUBTOTALS | 305 | 632 | 4,120 |
| <u>PLANT SPECIFIC ACTIONS</u> | 427 | 627 | 773 |
| TOTAL | 732 | 1,259 | 4,893 |

OPERATING REACTORS REQUEST UNDER REVIEW AS 2.206 PETITIONS

| <u>CONTROL NO.</u> | <u>FACILITY/REQUESTOR</u> | <u>STATUS</u> | <u>ASSIGNED TO</u> |
|--------------------|--|--|--------------------|
| 7823 | San Onofre - 1/Mis. Petitioners | Mtg. held June 9 w/licensee to discuss the Seismic Reevaluation program. DOL is unable to respond to petition until this issue is resolved | S. Nowicki |
| 220600 | Trojan/Coalition for Safe Power | Director's decision is being typed in final | C. Trammell |
| 10154 | Financial Qualifications /JAbbott | Comm. considering significant policy change on financial qualification, will await Comm. decision. Estimated completion date of September 9. | |
| 10218 | Browns Ferry/Safe Energy Alliance of Alabama | Director's decision to OELD for conc. | R. Clark |

OPERATING REACTOR EVENTS BRIEFING TASKS

| <u>NUMBER</u> | <u>MEETING DATE</u> | <u>LEAD RESPONSIBILITY</u> | <u>TITLE</u> |
|---------------|-------------------------|--------------------------------|--|
| OREB 80-1 | 12/10/80 | RSB | Quad Cities 2 drywell pressurization. |
| OREB 80-2 | 12/17/80 | RSB | Review of drywell pressure setpoint. |
| OREB 80-3 | 12/31/80 05/06/81 | ORAB/IE | Review of Tech Specs to assure decay heat removal capability. |
| OREB 80-4 | 12/31/80 01/07/81 | ORAB | Monitor experience with SDV Continuous Monitoring Systems. |
| OREB 80-5 | 12/31/80 | DE(MTEB) | Review VEPCO claim on turbine disc failure. |
| OREB 81-1 | 01/07/81 | RRAB | Review Hatch 1 and 2 history of problems. |
| OREB 81-2 | 01/28/81 | ORAB | Followup of Palisades Loss of DC Battery Power. |
| OREB 81-3 | 01/28/81 | DST | Dresden 3 failure of 2/4 reactor trip on turbine trip. |
| OREB 81-4 | 01/28/81 | DE | Review Farley inability to complete construction. |
| OREB 81-5 | | IE | include description of Millstone 1 event in IC Notice. |

| <u>NUMBER</u> | <u>MEETING DATE</u> | <u>LEAD RESPONSIBILITY</u> | <u>TITLE</u> |
|---------------|-------------------------|--------------------------------|--|
| OREB 81-6 | 04/08/81 | IE | Follow up on Brunswick 2 MSIV failure to determine cause. |
| OREB 81-7 | 05/06/81 | IE | Investigate cracking in FW regulatory valves at Beaver Valley. |
| OREB 81-8 | 05/06/81 | ORAB/AEOD/ DSI | Continue review of loss of DC. |
| OREB 81-9 | 05/06/81 | ORAB | Determine time RCIC can remove decay heat. |
| OREB 81-10 | 05/06/81 | DST | Review pump seal failures. |
| OREB 81-11 | 05/06/81 | ORAB | Compile plant by plant list of briefing events since 12/80. |
| OREB 81-12 | 05/13/81 | DST | Review TS action statements requiring rapid shutdown. |
| OREB 81-13 | 05/13/81 | DSI | Plan to assure adequate review of Bulletin 79-27. |
| OREB 81-14 | 06/03/81 | IE | San Onofre contaminated sand-source. |
| OREB 81-15 | 06/10/81 | DST | Review C. Michelson's concern relating to UHI line water hammer. |

IE/NRR TASK INTERFACE AGREEMENTS

| TASK NO. | TITLE | LEAD PM | TECHNICAL REVIEW ASSIGNMENT | TAC NUMBER |
|----------|---|----------|-----------------------------|-------------------------|
| ROI 80-1 | Oyster Creek Core Spray Sparger Cracking | Paulson | DE lead, Input from DSI | 12661 |
| ROI 80-2 | Oconee FW Nozzle Thermal Sleeve Cracking | Fairtile | DE | 11798 11799 11800 |
| ROI 80-3 | FitzPatrick Isolation Valve Failure | Polk | DE | 12014 |
| ROI 80-4 | Turbine Disc Cracking | Ross | DE | — |
| ROI 80-5 | Main Steam Line Break Analysis | Engle | DSI | — |
| ROI 80-6 | Davis Besse Turbine EHC Malfunction and ICS Interaction | Garner | DSI | CANCELLED |
| ROI 80-7 | Response to Sen. Hart letter | | N/A | COMPLETE |
| ROI 80-8 | Millstone 1 - Pipe Support Iso. Condenser | Shea | DL, ORAB | 12784 |

| TASK NO. | TITLE | LEAD PM | TECHNICAL REVIEW ASSIGNMENT | TAC NUMBER |
|-----------|--|----------|-----------------------------|-----------------|
| ROI 80-9 | IE Bulletin 79-01B Regional Meetings | Williams | DE, EQB | 12871 |
| ROI 80-10 | Maine Yankee Refueling Anomalies | Requa | DSI | 12872 |
| ROI 80-11 | IE Bulletin 79-27, Loss of Non-Class IE Instru. Power Bus | Fairtile | DSI | 42649 |
| ROI 80-12 | Crystal River 3 - Reactor Trip & Loss of Indication Instrumenta. | Fairtile | DL, ORAB (J.T. Beard) | 12961 |
| ROI 80-13 | North Anna 1 Loose Parts Monitoring System Indication | Engle | DSI | |
| ROI 80-14 | Quad Cities Core Spray Piping Cracks | Bevan | DE | 08228 |
| ROI 80-15 | ESF Reset Control Design Deficiency IE Bulletin 80-06 | Miner | DSI | 42720 --- 42789 |
| ROI 80-16 | BWR Jet Pump Hold Down Failures | Bevan | DE | 10*98 |
| ROI 80-17 | B&W Steam Gen. Overfill | Erickson | DSI | IN PROCESS |

| TASK NO. | TITLE | LEAD PM | TECHNICAL REVIEW ASSIGNMENT | TAC NUMBER |
|-----------|--|---------------|--------------------------------|-----------------|
| ROI 80-18 | Davis Besse 1 Loss of Decay Heat Removal During Refueling | D. Garner | DSI, DST Std. Tech. Spec. Sec. | 42122 |
| ROI 80-19 | E/Q of Post LOCA H ₂ Analyses at Browns Ferry 1, 2 & 3 | D. Clark | DL, ORAB | 42335 |
| ROI 80-20 | Arkansas 1 Pump Seal Failure May 10, 1980 | G. Vissing | DL, ORAB | 42616 |
| ROI 80-21 | Reactor Vessel Thermal Shock Due to Safety Injection | M. Fairtile | DE, DSI | 43425 --- 43432 |
| ROI 80-22 | Davis Besse Broken Hold Down Springs in B&W Fuel Assembly | D. Garner | DE, DSI | 42047 |
| ROI 80-23 | BWR Operation With Scram Discharge Volume in Degraded Cond. | J. Hannon | DSI | 42332 |
| ROI 80-24 | Boron Dilution in Mode 4 & 5 | E. Conner | DSI | COMPLETED BY PM |
| ROI 80-25 | ANO-2 Loss of Suction to Aux. FW Pumps After Loss of Offsite Power | R. Marlin | DSI | |
| ROI 80-26 | Turkey Pt. 4 Cracks in FW Nozzle to Pipe Reducer Weld | M. Grotenhuis | DE | |
| ROI 80-27 | Ft. Calhoun Corrosion of Pri. Coolant Pump Studs | P. Wagner | DE | 41002 |

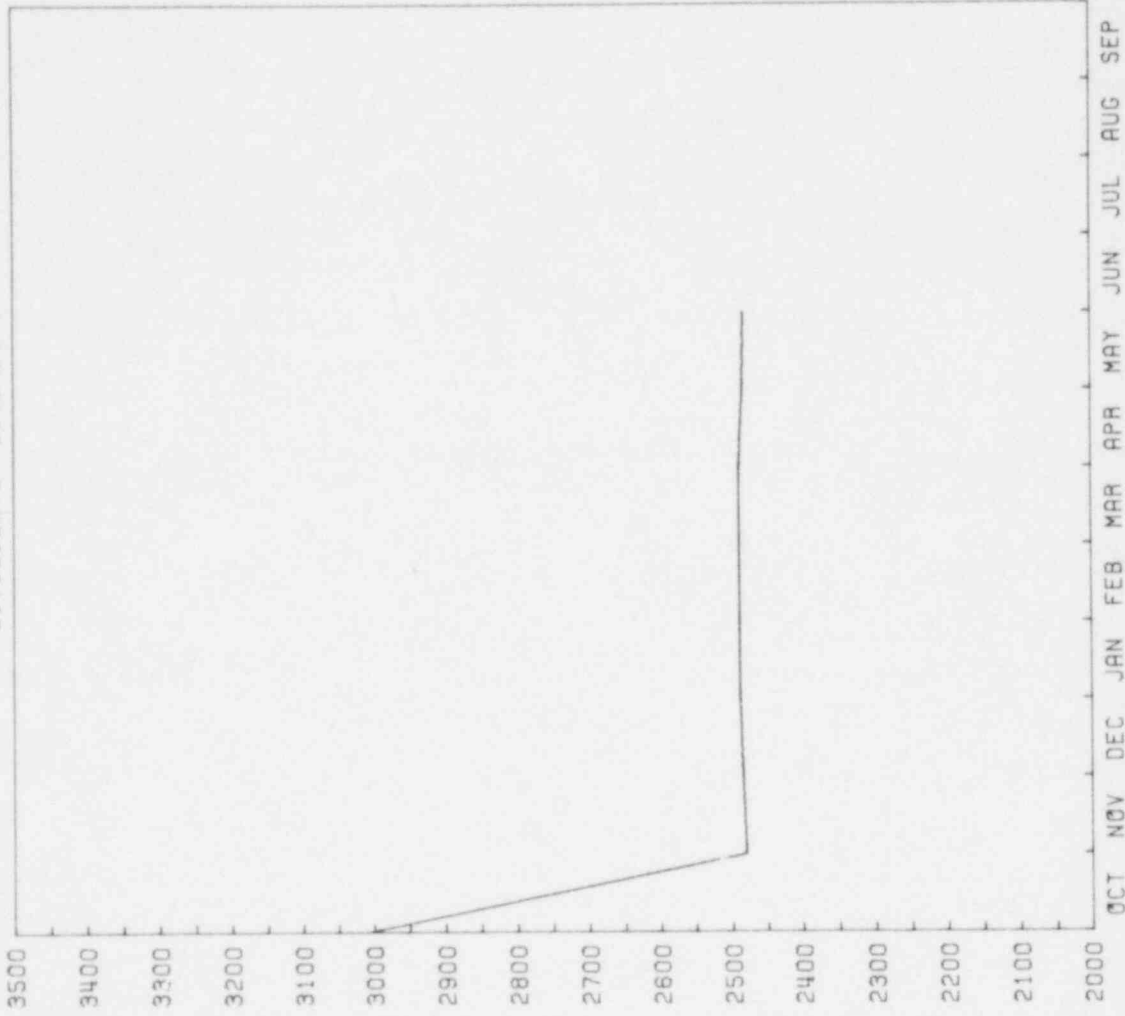
| TASK NO. | TITLE | LEAD PM | TECHNICAL REVIEW ASSIGNMENT | TAC NUMBER |
|-----------|---|-----------|-----------------------------|------------------------|
| ROI 80-28 | Deleted | — | — | — |
| ROI 80-29 | Deleted | — | — | — |
| ROI 80-30 | Masonry Wall Design (IE Bulletin 80-11) | Trammell | SEB | 42858 --- 42925 |
| ROI 80-31 | Kewaunee Steam Generator Bolt Deficiencies | Licciardo | ORAB (complete) | 42033 |
| ROI 80-32 | Dimensional Design Error in W Fabricated Fuel Assemblies For CE Reactors | — | cancelled 7/15/80 | — |
| ROI 80-33 | BWR Failure to Scram | Hannon | DSI | |
| ROI 80-34 | Severity of Open Valve in Sump Suction Line to ESF Pumps (Palisades) | Wambach | ORAB | NO TACS (short effort) |
| ROI 80-35 | Primary Containment Temperatures Exceed Design in Case of a Small Steam Line Break at 3RP | Paulson | ORAB | 42812 |
| ROI 80-36 | Palisades Potential for Water Hammer at High Auxiliary Feed-water Flowrates | Wambach | ORAB, ASB | NO TACS (short effort) |
| ROI 80-37 | B&W Operating Guidelines CR-3 | Erickson | ORAB, RSB | 42951 |

| TASK NO. | TITLE | LEAD PM | TECHNICAL REVIEW ASSIGNMENT | TAC NUMBER |
|-----------|--|-------------------|-----------------------------|-----------------|
| ROI 80-38 | ANO 1&2 Service Water | Vissing Martin | ORAB | 42831 |
| ROI 80-39 | Observed 40" level difference in Downcomers of "A" and "B" OTSGs at ANO-1 | Vissing | ORAB | 42930 |
| ROI 80-40 | Bus Fault and Loss of Instrument Bus Inverters at Kewaunee | Licciardo | ORAB | 42993 |
| ROI 80-41 | — | — | — | — |
| ROI 80-42 | IP-2 Flooding in Containment | Olshan | ORAB, MTEB | 43095 |
| ROI 80-43 | Accident-Induced Neutron Flux errors | Fairtile | | 43259 --- 43266 |
| ROI 80-44 | Spurious Actuation of ECCS with Spurious automatic transfer of ECCS to recir. mode due to elec. interconnection between channels | Garner | | 43253 |
| ROI 80-45 | Ice Condenser Flammability | Stahle | CEB | |
| ROI 81-1 | Millstone 2 - Loss of 125V Direct Current (DC) Emergency Bus | Conner | ORAB | 43543 |
| REB 81-1 | Supplement 4 to IE Bulletin 80-17 "Failure of Control Rods to Insert During a Scram at a BWR | Hannon | | 42208 |

| TASK NO. | TITLE | LEAD PM | TECHNICAL REVIEW ASSIGNMENT | TAC NUMBER |
|----------|--|----------------|-----------------------------|--|
| ROI 81-2 | Sequoyah LOCA | Stahle | ORAB | To Be Issued |
| ROI 81-4 | Hatch 1 Undervoltage Relays | Fairtile | ORAB | 43083 |
| ROI 81-5 | Brunswick Snubber Lockup Velocity | Hannon | ORAB | To Be Issued |
| ROI 81-6 | Palisades Disconnect of DC Power | Wambach | ORAB | To Be Issued (prior work charged to 50000255) |
| ROI 81-7 | South Texas, Liquefaction Problem | Sells | H&GEB | 50000498 R24 50000499 R24 |
| ROI 81-9 | Brunswick RHR Heat Exchangers | Hannon | ORAB | To Be Issued |
| ROI 81-9 | Westinghouse Part 21 Notification on CVCS design | To Be Assigned | RSB | To Be Issued |

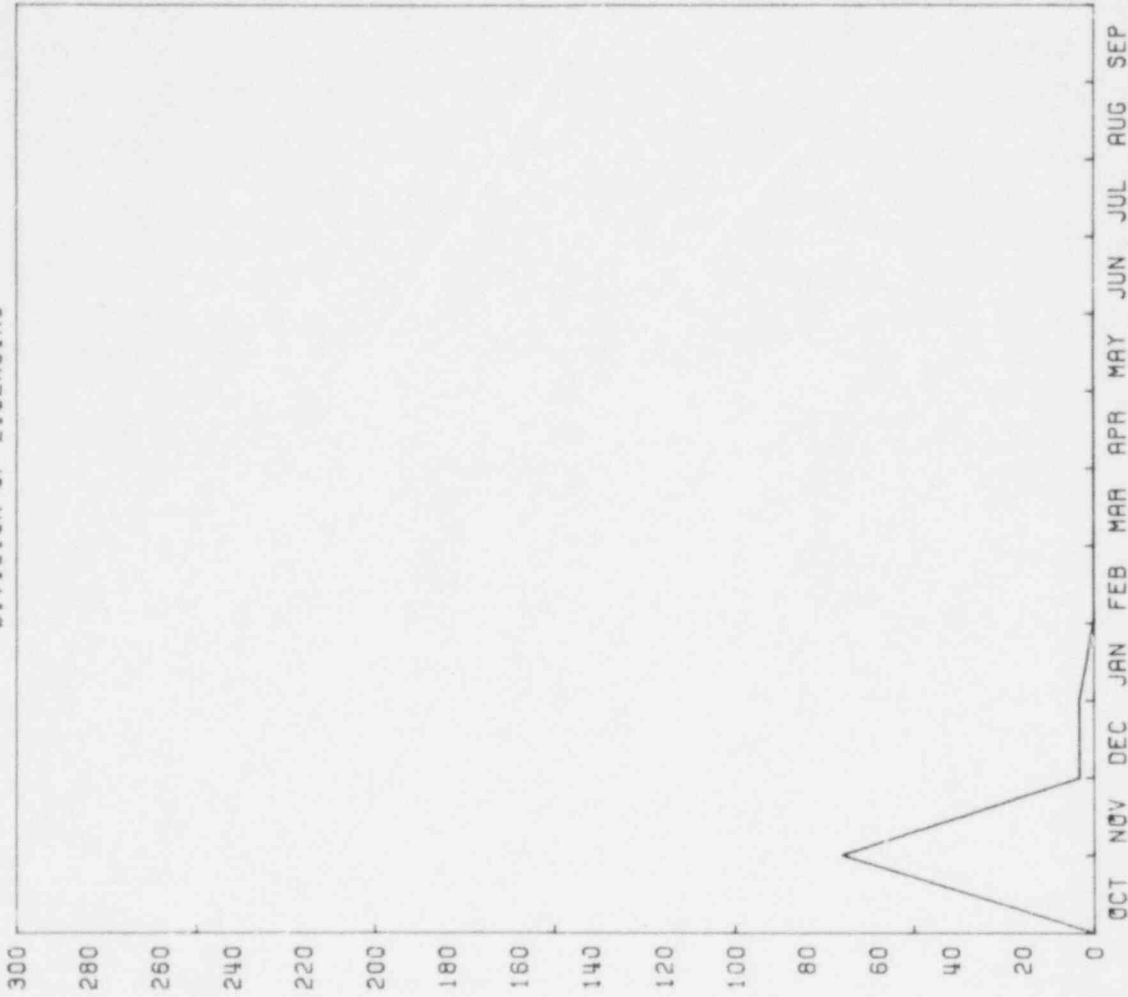
SECTION 2
T M I L I C E N S I N G A C T I O N S

TOTAL ACTIVE TMI ACTIONS
DIVISION OF LICENSING



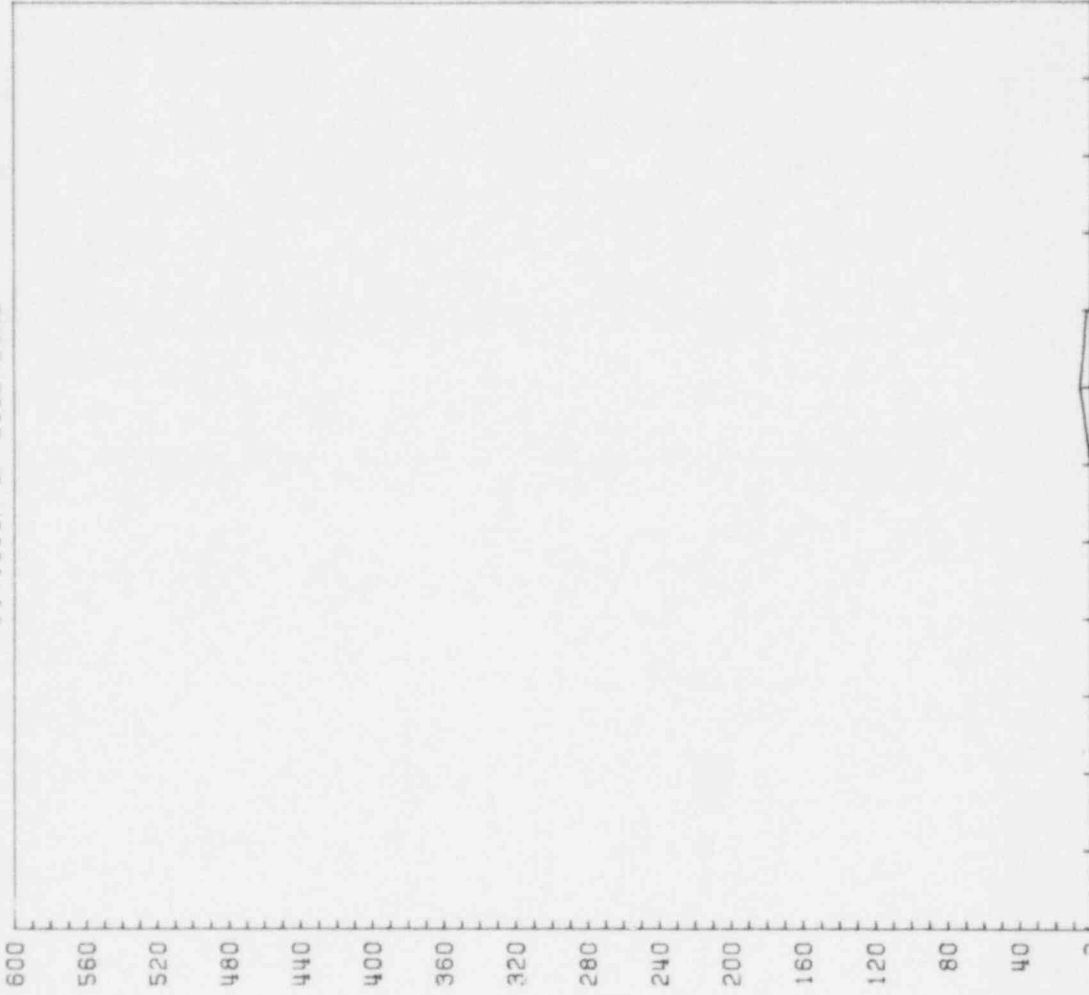
| | OCT | NOV | DEC | JAN | FEB | MAR | APR | MAY | JUN | JUL | AUG | SEP |
|----------|------|------|------|------|------|------|------|------|-----|-----|-----|-----|
| TOTAL | 2480 | 2484 | 2488 | 2488 | 2488 | 2488 | 2482 | 2480 | | | | |
| ACTIONS | 2480 | 2484 | 2488 | 2488 | 2488 | 2488 | 2482 | 2480 | | | | |
| HEARINGS | 00 | 00 | 00 | 00 | 00 | 00 | 00 | 00 | | | | |

NEW TMI ACTIONS BY MONTH
DIVISION OF LICENSING



| | OCT | NOV | DEC | JAN | FEB | MAR | APR | MAY | JUN | JUL | AUG | SEP |
|-------------|-----|-----|-----|-----|-----|-----|-----|-----|-----|-----|-----|-----|
| NEW ACTIONS | 70 | 04 | 04 | 00 | 00 | 00 | 00 | 00 | 00 | | | |
| CUMULATIVE | 70 | 74 | 78 | 78 | 78 | 78 | 78 | 78 | 78 | | | |

COMPLETED TMI ACTIONS BY MONTH
DIVISION OF LICENSING



| | OCT | NOV | DEC | JAN | FEB | MAR | APR | MAY | JUN | JUL | AUG | SEP |
|------------|-----|-----|-----|-----|-----|-----|-----|-----|-----|-----|-----|-----|
| COMPLETED | 00 | 00 | 00 | 00 | 00 | 00 | 06 | 02 | | | | |
| CUMULATIVE | 0 | 0 | 0 | 0 | 0 | 0 | 6 | 8 | | | | |

FISCAL YEAR: 1981

TMI LICENSING ACTIONS BY MONTH

REPORT DATE: 06/15/81

| <u>FACILITY NAME</u> | <u>OCT.</u> | <u>NOV.</u> | <u>DEC.</u> | <u>JAN.</u> | <u>FEB.</u> | <u>MAR.</u> | <u>APR.</u> | <u>MAY</u> | <u>JUNE</u> | <u>JULY</u> | <u>AUG.</u> | <u>SEPT.</u> |
|----------------------|-------------|-------------|-------------|-------------|-------------|-------------|-------------|------------|-------------|-------------|-------------|--------------|
| ARKANSAS 1 | 39 | 39 | 39 | 39 | 39 | 39 | 39 | 39 | | | | |
| ARKANSAS 2 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | | | | |
| BEAVER VALLEY 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| BIG ROCK POINT 1 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| BROWNS FERRY 1 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| ----- | | | | | | | | | | | | |
| BROWNS FERRY 2 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| BROWNS FERRY 3 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| BRUNSWICK 1 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| BRUNSWICK 2 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| CALVERT CLIFFS 1 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | | | | |
| ----- | | | | | | | | | | | | |
| CALVERT CLIFFS 2 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | | | | |
| COOK 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| COOK 2 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| COOPER STATION | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| CRYSTAL RIVER 3 | 39 | 39 | 39 | 39 | 39 | 39 | 39 | 39 | | | | |
| ----- | | | | | | | | | | | | |
| DAVIS-BESSE 1 | 39 | 39 | 39 | 39 | 39 | 39 | 39 | 39 | | | | |
| DRESDEN 2 | 71 | 72 | 72 | 72 | 72 | 72 | 72 | 72 | | | | |
| DRESDEN 3 | 37 | 38 | 38 | 38 | 38 | 38 | 38 | 38 | | | | |
| DUANE ARNOLD | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| FARLEY 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| ----- | | | | | | | | | | | | |
| FARLEY 2 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| FITZPATRICK | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| FORT CALHOUN 1 | 30 | 30 | 31 | 31 | 31 | 31 | 31 | 31 | | | | |
| FORT ST VRAIN | 26 | 26 | 26 | 26 | 26 | 26 | 26 | 26 | | | | |
| GINNA | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| ----- | | | | | | | | | | | | |
| HADDAM NECK | 35 | 35 | 35 | 35 | 35 | 35 | 34 | 34 | | | | |
| HATCH 1 | 36 | 36 | 36 | 36 | 36 | 36 | 35 | 35 | | | | |
| HATCH 2 | 36 | 36 | 36 | 36 | 36 | 36 | 35 | 35 | | | | |
| INDIAN POINT 2 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | 1 | | | | |
| INDIAN POINT 3 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| ----- | | | | | | | | | | | | |
| KEWAUNEE | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| MAINE YANKEE | 32 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | | | | |
| MCGUIRE 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| MILLSTONE 1 | 26 | 26 | 26 | 26 | 26 | 26 | 26 | 26 | | | | |
| MILLSTONE 2 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | | | | |
| ----- | | | | | | | | | | | | |
| MONTICELLO | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| NINE MILE POINT 1 | 38 | 38 | 38 | 38 | 38 | 38 | 37 | 37 | | | | |
| NORTH ANNA 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| NORTH ANNA 2 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| OCONEE 1 | 40 | 40 | 41 | 41 | 41 | 41 | 41 | 41 | | | | |

R-1208612-001

TECHNICAL ASSIGNMENT CONTROL SYSTEM

PAGE: D-2

FISCAL YEAR: 1981

T M I LICENSING ACTIONS BY MONTH

REPORT DATE: 06/15/81

| FACILITY NAME | OCT. | NOV. | DEC. | JAN. | FEB. | MAR. | APR. | MAY | JUNE | JULY | AUG. | SEPT. |
|---------------------|-------|-------|-------|-------|-------|-------|-------|-------|------|------|------|-------|
| OCONEE 2 | 41 | 41 | 42 | 42 | 42 | 42 | 42 | 41 | | | | |
| OCONEE 3 | 41 | 41 | 42 | 42 | 42 | 42 | 42 | 41 | | | | |
| OYSTER CREEK 1 | 38 | 38 | 38 | 38 | 38 | 38 | 38 | 38 | | | | |
| PALISADES | 32 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | | | | |
| PEACH BOTTOM 2 | 37 | 37 | 37 | 37 | 37 | 37 | 36 | 36 | | | | |
| PEACH BOTTOM 3 | 36 | 36 | 36 | 36 | 36 | 36 | 35 | 35 | | | | |
| PILGRIM 1 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| POINT BEACH 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| POINT BEACH 2 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| PRAIRIE ISLAND 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| PRAIRIE ISLAND 2 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| QUAD CITIES 1 | 35 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| QUAD CITIES 2 | 35 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| RANCHO SECO 1 | 39 | 39 | 39 | 39 | 39 | 39 | 39 | 39 | | | | |
| ROBINSON 2 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| SALEM 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| SALEM 2 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| SAN ONOFRE 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| SEQUOYAH 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| ST LUCIE 1 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | 32 | | | | |
| SURRY 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| SURRY 2 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| THREE MILE ISLAND 1 | 13 | 13 | 13 | 13 | 13 | 13 | 13 | 13 | | | | |
| TROJAN | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| TURKEY POINT 3 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| TURKEY POINT 4 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| VERMONT YANKEE 1 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | 36 | | | | |
| YANKEE-ROWE 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| ZION 1 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| ZION 2 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | 35 | | | | |
| GRAND TOTAL | 2,480 | 2,484 | 2,488 | 2,488 | 2,488 | 2,488 | 2,482 | 2,480 | | | | |

REPORT DATE: 06/12/81

TMI LICENSING ACTIONS

POWER REACTORS

FOR 05/01/81 - 05/31/81

| FACILITY | NEW ACTIONS FOR THIS REPORTING PERIOD | NEW ACTIONS TO DATE IN FY 81 | COMPLETED ACTIONS FOR THIS REPORTING PERIOD | COMPLETED ACTIONS TO DATE IN FY 81 | NUMBER OF ROUTINE ACTIONS | NUMBERS OF HEARINGS | TOTAL ACTIONS |
|------------------|---------------------------------------|------------------------------|---|------------------------------------|---------------------------|---------------------|---------------|
| ARKANSAS 1 | 0 | 1 | 0 | 0 | 39 | 0 | 39 |
| ARKANSAS 2 | 0 | 1 | 0 | 0 | 32 | 0 | 32 |
| BEAVER VALLEY 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| BIG ROCK POINT 1 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| BROWNS FERRY 1 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| BROWNS FERRY 2 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| BROWNS FERRY 3 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| BRUNSWICK 1 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| BRUNSWICK 2 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| CALVERT CLIFFS 1 | 0 | 1 | 0 | 0 | 32 | 0 | 32 |
| CALVERT CLIFFS 2 | 0 | 1 | 0 | 0 | 32 | 0 | 32 |
| COOK 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| COOK 2 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| COOPER STATION | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| CRYSTAL RIVER 3 | 0 | 1 | 0 | 0 | 39 | 0 | 39 |
| DAVIS-BESSE 1 | 0 | 1 | 0 | 0 | 39 | 0 | 39 |
| DRESDEN 2 | 0 | 2 | 0 | 0 | 72 | 0 | 72 |
| DRESDEN 3 | 0 | 1 | 0 | 0 | 38 | 0 | 38 |
| DUANE ARNOLD | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| FARLEY 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| FARLEY 2 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| FITZPATRICK | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| FORT CALHOUN 1 | 0 | 2 | 0 | 0 | 31 | 0 | 31 |
| FORT ST VRAIN | 0 | 1 | 0 | 0 | 26 | 0 | 26 |
| GINNA | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| HADDAM NECK | 0 | 1 | 0 | 0 | 34 | 0 | 34 |
| HATCH 1 | 0 | 1 | 0 | 1 | 35 | 0 | 35 |
| HATCH 2 | 0 | 1 | 0 | 1 | 35 | 0 | 35 |
| INDIAN POINT 2 | 0 | 0 | 0 | 0 | 1 | 0 | 1 |
| INDIAN POINT 3 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| KFWAUNEE | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| LA CROSSE | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| MAINE YANKEE | 0 | 2 | 0 | 0 | 32 | 0 | 32 |
| MCGUIRE 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| MILLSTONE 1 | 0 | 1 | 0 | 0 | 26 | 0 | 26 |

REPORT DATE: 06/12/81

TMI LICENSING ACTIONS

POWER REACTORS

FOR 05/01/81 - 05/31/81

| FACILITY | NEW ACTIONS FOR THIS REPORTING PERIOD | NEW ACTIONS TO DATE IN FY 81 | COMPLETED ACTIONS FOR THIS REPORTING PERIOD | COMPLETED ACTIONS TO DATE IN FY 81 | NUMBER OF ROUTINE ACTIONS | NUMBERS OF HEARINGS | TOTAL ACTIONS |
|---------------------|---|---------------------------------------|---|---|------------------------------------|---------------------------|------------------|
| MILLSTONE 2 | 0 | 1 | 0 | 0 | 32 | 0 | 32 |
| MONTICELLO | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| NINE MILE POINT 1 | 0 | 1 | 0 | 1 | 37 | 0 | 37 |
| NORTH ANNA 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| NORTH ANNA 2 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| OCONEE 1 | 0 | 2 | 0 | 0 | 41 | 0 | 41 |
| OCONEE 2 | 0 | 3 | 1 | 1 | 41 | 0 | 41 |
| OCONEE 3 | 0 | 3 | 1 | 1 | 41 | 0 | 41 |
| OYSTER CREEK 1 | 0 | 1 | 0 | 0 | 38 | 0 | 38 |
| PALISADES | 0 | 1 | 0 | 0 | 32 | 0 | 32 |
| PEACH BOTTOM 2 | 0 | 1 | 0 | 1 | 36 | 0 | 36 |
| PEACH BOTTOM 3 | 0 | 1 | 0 | 1 | 35 | 0 | 35 |
| PILGRIM 1 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| POINT BEACH 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| POINT BEACH 2 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| PRAIRIE ISLAND 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| PRAIRIE ISLAND 2 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| QUAD CITIES 1 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| QUAD CITIES 2 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| RANCHO SECO 1 | 0 | 1 | 0 | 0 | 39 | 0 | 39 |
| ROBINSON 2 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| SALEM 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| SALEM 2 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| SAN ONOFRE 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| SEQUOYAH 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| ST LUCIE 1 | 0 | 1 | 0 | 0 | 32 | 0 | 32 |
| SURRY 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| SURRY 2 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| THREE MILE ISLAND 1 | 0 | 0 | 0 | 0 | 13 | 0 | 13 |
| TROJAN | 0 | 2 | 0 | 0 | 35 | 0 | 35 |
| TURKEY POINT 3 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| TURKEY POINT 4 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| VERMONT YANKEE 1 | 0 | 1 | 0 | 0 | 36 | 0 | 36 |
| YANKEE-ROWE 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| ZION 1 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |

REPORT DATE: 06/12/81

TMI LICENSING ACTIONS

POWER REACTORS

FOR 05/01/81 - 05/31/81

| FACILITY | NEW ACTIONS FOR THIS REPORTING PERIOD | NEW ACTIONS TO DATE IN FY 81 | COMPLETED ACTIONS FOR THIS REPORTING PERIOD | COMPLETED ACTIONS TO DATE IN FY 81 | NUMBER OF ROUTINE ACTIONS | NUMBERS OF HEARINGS | TOTAL ACTIONS |
|-------------|---|---------------------------------------|---|---|------------------------------------|---------------------------|------------------|
| ZION 2 | 0 | 1 | 0 | 0 | 35 | 0 | 35 |
| GRAND TOTAL | 0 | 78 | 2 | 8 | 2,480 | 0 | 2,480 |

| Item # | Title | Description | Applicability | Pre Imp | Post Imp | Tech Spec | Lic. Action | Lead Tech Rev. | Tech Assist. | |
|---------------|----------------------------|----------------------------|---------------|---------|----------|-----------|-------------|----------------|--------------|--|
| I.A.1.1 | STA | 1. on duty | All | No | Yes | Yes | ER | DOR | No | |
| | | 2. T.S. | All | Yes | No | Yes | SER | PM/OMB | No | |
| | | 3. training | All | No | Yes | Yes | SER | LQB | LLL | |
| | | 4. long term | All | No | No | No | None | LQB | None | |
| I.A.1.2 | Shift Super. Resp. | New procedure | All | No | Yes | No | ER | DOR | No | |
| I.A.1.3 | Shift Manning | 1. limit OT | All | No | Yes | No | None | OIE | No | |
| | | 2. minimum crew | All | No | Yes | Yes | Model | LQB | No | |
| | | 3. T.S. | All | Yes | No | Yes | SER | PM/ORAB | No | |
| I.A.2.1 | Upgrade RO/SRO Training | 1. SRO exp. | All | No | Yes | No | None | OLB | No | |
| | | 2. SRO/RO exp | All | No | Yes | No | None | OLB | No | |
| | | 3. 3 mth trng. | All | No | Yes | No | None | OLB | No | |
| | | 4. modify trng | All | No | Yes | No | Ltr. | OLB | No | |
| | | 5. facility cert | All | No | Yes | No | None | OLB | No | |
| I.A.2.3 | Admin. Trng. Program | Instructing SRO Superv. | All | No | Yes | No | None | OLB | No | |
| I.A.3.1 | Scope C.it. for Lic. Exams | 1. Inc. scope | All | No | No | No | None | OLB | No | |
| | | 2. Inc. grade | All | No | No | No | None | OLB | No | |
| | | 3. Sim. exam | All | No | No | No | None | OLB | No | |
| I.C.1 | | 1. SB LOCA | All | No | Yes | No | ER | DOR | No | |
| | | 2. Inadequate Core Cooling | | | | | | | | |
| | | a. guideliner | All | Yes | No | No | Ltr | RSB | ORM | |
| | | b. procedures | All | No | Yes | No | SER | PTB/OIE | FRC | |
| | | 3. Transients & Accidents | | | | | | | | |
| | | a. guidelines | All | Yes | No | No | Ltr | RSB | EGG | |
| b. procedures | All | No | Yes | No | SER | PTB/OIE | FRC | | | |

| Item # | Title | Description | Applicability | Pre Imp | Post Imp | Tech Spec | Lic. Action | Lead Tech Rev. | Tech Assist. |
|--------|------------------------|-------------------------|---------------|---------|----------|-----------|----------------|----------------|--------------|
| I.C.2 | Shift Relief Turnover | Proceadure Change | All | No | Yes | No | ER | DOR | No |
| I.C.3 | Shift Super Resp | Define Responsibilities | All | No | Yes | No | ER | DOR | No |
| I.C.4 | C.R. Access | Procedures | All | No | Yes | No | ER | DOR | No |
| I.C.5 | Feedback Op. Exp | Procedures | All | No | Yes | No | None | OIE | No |
| I.C.6 | Correct perf. op. act. | Procedures | All | No | Yes | No | None | OIE | No |
| I.D.1 | Control Room | a. prel. assess. | All | No | No | No | NUREG criteria | HFE | Yes |
| | | b. correct deficiencies | All | Yes | No | TID | TID | HFE | Yes |
| I.D.2 | Plant Safety | 1. Criteria | All | NA | NA | No | NUREG | HFE | No |
| | | 2. Installed | All | Yes | No | Yes | SER | HFE | No |
| | | 3. Implement | All | No | Yes | No | None | OIE | No |
| II.B.1 | RCS Vents | 1. Conceptual Des. | All | No | Yes | No | ER | DOR | No |
| | | 2. Final Design | All | No | Yes | Yes | Model T.S. | RSB | LLL |
| | | 3. Install | All | Yes | No | Yes | SER | RSB | FRC |
| | | 4. Procedures | All | Yes | No | No | SER | RSB | LLL |

| Item # | Title | Description | Applicability | Pre Imp | Post Imp | Tech Spec | Lic. Action | Lead Tech Rev. | Tech Assist. |
|----------|---------------------------|-----------------------|---------------|---------|----------|-----------|-------------|----------------|--------------|
| II.B.2 | Shielding | 1. Prel. Des. | All | No | Yes | No | ER | DOR | Yes |
| | | 2. Final Des. & Mods. | All | No | Yes | No | SER | RAB | Exxon |
| | | 3. Equip. Qual. | All | No | Yes | No | SER | EQB | No |
| II.B.3 | Sampling | 1. Interim Syst. | All | No | Yes | No | ER | DOR | Yes |
| | | 2. Plant Mods. | All | No | Yes | Yes | SER | ETS | Exxon |
| II.B.4 | Trng for Mit. Core Damage | 1. Dev. program | All | No | Yes | No | None | OLB | None |
| | | 2. Initiation | All | No | Yes | No | None | OLB | None |
| | | 3. Completed | All | No | Yes | No | SER | OLB | None |
| II.D.1 | Valve Test Req. | 1. Subm program | All | No | Yes | No | ER | DOR | Yes |
| | | 2. Testing | | | | | | | |
| | | a. Complete | All | No | Yes | No | None | MEB | LLL |
| | | b. Plant Subm | All | Yes | Yes | TBD | SER | MEB | LLL |
| | | 3. Block Valve Tests | PWR | Yes | Yes | TBD | SER | MEB | LLL |
| II.D.3 | Valve Position Indicate | 1. Install | All | No | Yes | No | ER | DOR | Yes |
| | | 2. T.S. | All | Yes | No | Yes | SER | PM/ORAB | No |
| II.E.1.1 | AFW Eval. | 1. Short term | PWR | Yes | No | TBD | SER | ASB | FRC |
| | | 2. Long term | PWR | Yes | No | TBD | SER | ASB | FRC |
| II.E.1.2 | AFW Inst. & Flow | 1. Initiation | | | | | | | |
| | | a. Control grade | PWR | No | Yes | Yes | ER | DOR | Yes |
| | | b. T.S. | PWR | Yes | No | Yes | SER | PM/ORAB | FRC |
| | | c. Safety grade | PWR | Yes | No | Yes | SER | ICS | FRC |
| | | d. T.S. | PWR | Yes | No | Yes | SER | ICS | FRC |

| Item # | Title | Description | Applicability | Pre Imp | Post Imp | Tech Spec | Lic. Action | Lead Tech Rev. | Tech Assist. |
|----------|--------------------|---------------------------------|---------------|---------|----------|-----------|-------------|----------------|--------------|
| II.E.1.2 | AFW Flow | 2. Flow Ino. | | | | | | | |
| | | a. Control grade | PWR | No | Yes | Yes | ER | DOR | Yes |
| | | b. T.S. | PWR | Yes | No | Yes | SER | PM/ORAB | No |
| | | c. Safety grade | PWR | Yes | No | Yes | SER | ICS | FRC |
| II.E.1.2 | AFW Flow | d. T.S. | PWR | Yes | No | Yes | SER | ICS | FRC |
| | | 1. Upgrade | PWR | No | Yes | Yes | ER | DOR | Yes |
| | | 2. T.S. | PWR | Yes | No | Yes | SER | PM/ORAB | No |
| | | 1. Design | All | No | Yes | No | ER | DOR | Yes |
| II.E.4.1 | Ded. Penetra. | 2. Installed | All | Yes | No | Yes | SER | PM/ORAB | No |
| | | 1. Mods | All | No | Yes | Yes | ER | DOR | Yes |
| II.E.4.2 | Cont. Isol. | 2. T.S. | All | Yes | No | Yes | SER | PM/ORAB | No |
| | | 3. Pressure Setpoint | | | | | | | |
| | | a. Specify | All | No | Yes | No | Model T.S. | CSB | LLL |
| | | b. Mods | All | Yes | No | Yes | SER | CSB | LLL |
| | | 4. Cont. Purge | All | No | Yes | Yes | SER | CSB | LLL |
| | | 5. Rad signal | All | No | Yes | Yes | SER | CSB | LLL |
| | | 6. Classify Sys. | All | No | Yes | Yes | SER | CSB | LLL |
| II.F.1 | Acc. Mon. | 1. NG monitor | All | No | Yes | Yes | SER | ETSB | EGG |
| | | 2. Iodine | All | No | Yes | Yes | SER | ETSB | Exxon |
| | | 3. Cont. Hi Rad | All | No | Yes | Yes | SER | ETSB | Exxon |
| | | 4. Cont. Pressure | All | No | Yes | Yes | SER | CSB | LLL |
| | | 5. Cont. H ₂ O Level | All | No | Yes | Yes | SER | CSB | LLL |
| | | 6. Cont. Hydrogen | All | No | Yes | Yes | SER | CSB | LLL |
| II.F.2 | Inad. Core Cooling | 1. Sub cool meter | | | | | | | |
| | | a. Install | PWR | No | Yes | Yes | SER | DOR | Yes |
| | | b. T.S. | PWR | Yes | No | Yes | SER | PM/ORAB | No |
| II.F.2 | Inad. Core Cooling | 2. Level Instr | All | No | Yes | Yes | SER | CPB | ORAL |
| | | 1. Upgrade Sys. | PWR | No | Yes | Yes | SER | DOR | Yes |
| II.G.1 | Power Supply PORV | 2. T.S. | PWR | Yes | No | Yes | SER | PM/ORAB | No |
| | | 79-08, 06, 08 | All | No | Yes | No | SER | PM | No |

| Item # | Title | Description | Applicability | Pre Imp | Post Imp | Tech Spec | Lic. Action | Lead Tech Rev | Tech Assist. | | | | | | | | | | | | |
|---------------------------------|---------------|---------------------------|---------------|---------------------------|----------|-----------|----------------|---------------|--------------|------|------|------|----|--|------|-----|-----|-----|-----|------|----|
| II.K.2 | Orders on B&W | 8. Upgrade AFW | B&W | Yes | No | As req. | (SEE II.E.1.1) | ICSB | LLL | | | | | | | | | | | | |
| | | 9. FEMA on ICS | D&W | No | Yes | TBD | | | | | | | | | | | | | | | |
| | | 10. Safety Grade Trip | B&W | Yes | No | Yes | | | | SER | ICSB | LLL | | | | | | | | | |
| | | 11. Op. Tng | B&W | No | Yes | No | | | | None | OIE | No | | | | | | | | | |
| | | 13. Thermal Acc. Rep | B&W | No | Yes | TBD | | | | SER | RSB | LLL | | | | | | | | | |
| | | 14. Lift freq PORV | B&W | (SEE II.K.3.7) | No | Yes | | | | No | SER | RRAB | No | | | | | | | | |
| | | 15. Effects of Slug Flow | B&W | | | | | | | | | | | | | | | | | | |
| | | 16. RCP Seal Damage | B&W | No | Yes | No | | | | SER | RSB | No | | | | | | | | | |
| | | 17. Voiding in RCS | B&W | No | Yes | No | | | | SER | RSB | No | | | | | | | | | |
| | | 19. Benchmark Anal. | B&W | No | Yes | No | | | | SER | RSB | No | | | | | | | | | |
| | | | CE & W | No | Yes | No | | | | SER | RSB | No | | | | | | | | | |
| | | 20. Sys. Resp. to SB LOCA | B&W | No | Yes | No | | | | SER | RSB | No | | | | | | | | | |
| | | II.K.3 | | 1. Auto PORV Iso. | | | | | | | | | | | | | | | | | |
| | | | | a. design | | | | | | | | | | | PWR | Yes | No | No | SER | ICSB | No |
| | | | | b. Inst. & Test | | | | | | | | | | | PWR | Yes | No | Yes | SER | ICSB | No |
| | | | | 2. Report PORV Failures | | | | | | | | | | | PWR | No | Yes | No | SER | RRAB | No |
| | | | | 3. Report RV, SV Failures | | | | | | | | | | | All | No | Yes | Yes | SER | OEEB | No |
| | | | | 5. Auto Trip of RCB | | | | | | | | | | | PWR | No | Yes | No | SER | RSB | No |
| | | | | a. design | | | | | | | | | | | | Yes | No | Yes | SER | RSB | No |
| | | | | b. modify | | | | | | | | | | | | | | | | | |
| 7. Eval PORV Opening | B&W | | | No | | | Yes | No | SER | | | | | | RRAB | No | | | | | |
| 9. PID Controller | W | | | No | | | Yes | No | None | | | | | | ICSB | No | | | | | |
| 10. Antic Trip Mod | W | | | Yes | | | No | Yes | SER | | | | | | ICSB | LLL | | | | | |
| 11. Justify cert PORVs | PWR | | | No | | | Yes | No | SER | | | | | | RSB | No | | | | | |
| 12. Antic trip on T.T. | W | | | No | | | Yes | No | SER | | | | | | ICS | LLL | | | | | |
| a. design | | | | Yes | | | No | Yes | SER | | | | | | ICS | LLL | | | | | |
| b. modify | | | | | | | | | | | | | | | | | | | | | |
| 13. HPCI/RCIC ini level | BWR | No | Yes | No | SER | RSB | No | | | | | | | | | | | | | | |
| a. analysis | | Yes | No | Yes | SER | RSB | No | | | | | | | | | | | | | | |
| b. modify | | | | | | | | | | | | | | | | | | | | | |
| 14. Iso. condenser on radiation | BWR | No | Yes | Yes | SER | CSB | No | | | | | | | | | | | | | | |

| tem # | Title | Description | Applicability | Pre Imp | Post Imp | Tech Spec | Lic. Action | Lead Tech Rev | Tech Assist. |
|-------|-------|-------------------------------|---------------|------------------|----------|-----------|-------------|---------------|--------------|
| I.K.3 | | 15. Iso of HPCI/RCIC | BWR | No | Yes | Yes | SER | RSB | No |
| | | 16. | | | | | | | |
| | | a. Study | BWR | No | Yes | No | SER | RRAB | No |
| | | b. Modify | BWR | Yes | No | Yes | SER | RRAB | No |
| | | 17. ECCS outages | All | No | Yes | TBD | Ltr | RRAB | No |
| | | 18. ADS actuation | | | | | | | |
| | | a. Study | BWR | No | Yes | No | Ltr | ICS | No |
| | | b. design | BWR | No | Yes | No | SER | ICS | No |
| | | c. modify | BWR | Yes | No | Yes | SER | ICS | No |
| | | 19. Interlock Recir Pump | BWR | No | Yes | Yes | Yes | RSB | No |
| | | 20. Loss SV water | Big Rock | No | Yes | TBD | SER | ASB | No |
| | | 21. Restart LPCI/CSB | | | | | | | |
| | | a. design | BWR | No | Yes | No | SER | ICSB | No |
| | | b. modify | BWR | Yes | No | Yes | SER | ICSB | No |
| | | 22. RCIC suction | | | | | | | |
| | | a. verify press. | BWR | No | Yes | No | None | GIE | No |
| | | b. modify | BWR | No | Yes | Yes | SER | ICSB | No |
| | | 24. Space cooling - HPCI/RCIC | BWR | No | Yes | Yes | SER | RSB | No |
| | | 25. Power on pump seals | | | | | | | |
| | | a. design | BWR | No | Yes | No | SER | RSB | No |
| | | | CE, W | No | Yes | No | SER | RSB | No |
| | | b. modify | BWR | Yes | No | No | None | RSB | No |
| | | | CE, W | Yes | No | No | None | RSB | No |
| | | 27. Common ref. level | BWR | No | Yes | Yes | SER | ICS | No |
| | | 28. Qual of ADS | BWR | No | Yes | TBD | Position | EQB | No |
| | | 29. Perf. of IC | BWR | No | Yes | No | Position | RSB | No |
| | | 30. SB LOCA methods | | | | | | | |
| | | a. schedule | All | No | Yes | No | Position | RSB | Yes |
| | | b. model | All | Yes | No | No | SER | RSB | Yes |
| | | c. new analyses | All | Yes | No | No | SER | RSB | Yes |
| | | 31. Comply w/ BO, 46 | All | Yes | No | TBD | SER | RSB | Yes |
| | | 40. RCP Seal Damage | | (SEE II.K.2.16) | | | | | |
| | | 43. Slug Flow | | (SEE II.K.2.15) | | | | | |
| | | 44. | BWR | No | Yes | TBD | Position | RSB | Yes |

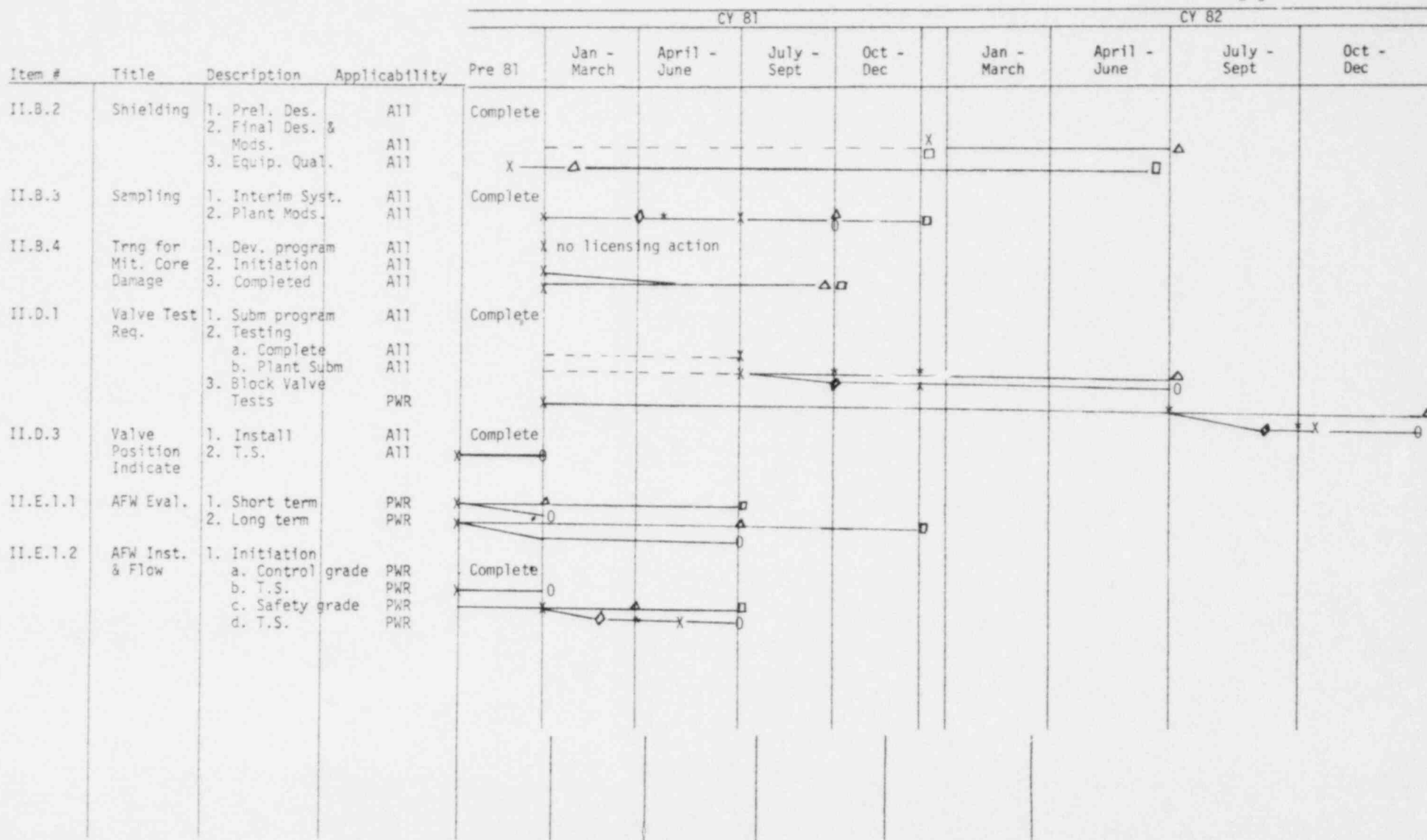
| Item # | Title | Description | Applicability | Pre Imp | Post Imp | Tech Spec | Lic. Action | Lead Tech Rev. | Tech Assist. |
|----------|-------------------|--|-------------------|------------------|-------------------|------------------|------------------------|---------------------|--------------------|
| I.K.3 | | 45. Manual depress. 46. Michelson concern 57. Manual Init of ADS | BWR BWR BWR | No No No | Yes Yes Yes | TBD No No | SER Ltr Position | RSB RSB RSB | Yes Yes Yes |
| II.A.1.1 | EP Short Term | Short Term | A11 | No | Yes | No | None | EPLB | LLL PNL |
| II.A.1.2 | Upgrade Sys Func. | 1. Interim TSC, OSC 2. Design 3. Modify | A11 A11 A11 | No TBD TBD | Yes TBD TBD | No TBD TBD | ER SER SER | DOR EPLB EPLB | None Yes res |
| II.A.2 | Emer. Prep. | 1. App. E upgrade 2. MET dates | A11 A11 | No No | Yes Yes | Yes Yes | SER SER | EPLB EPLB | Yes Yes |
| II.D.1.1 | Sys Integ | 1. Lead Rad 2. T.S. | A11 A11 | No Yes | Yes No | Yes Yes | ER SER | DOR PM/ORAB | Yes No |
| II.D.2.3 | In-plant Rad Mod. | 1. Interim 2. Upgrade Cap. | A11 A11 | No No | Yes Yes | No Yes | ER SER | DOR RAB | Yes PNL |
| II.D.3.4 | Con. Room Habit. | 1. Review 2. Modify | A11 A11 | No No | Yes Yes | No Yes | | | |

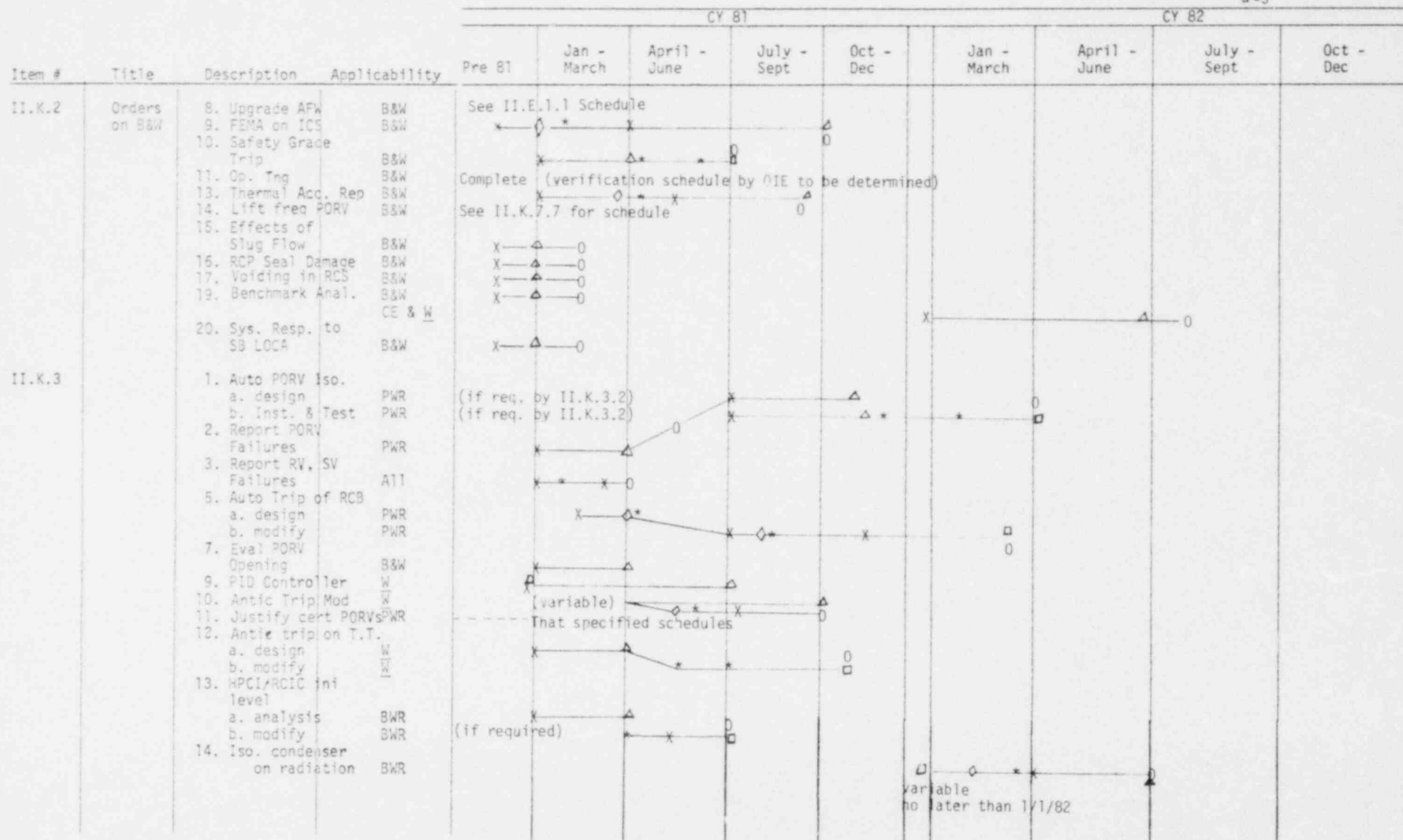
NUREG-0737 MILESTONE

G-1

| Item # | Title | Description | Applicability | CY 81 | | | | CY 82 | | | | | |
|---------|----------------------------|---|--|--|-------------|--------------|-------------|-----------|-------------|--------------|-------------|-----------|---|
| | | | | Pre 81 | Jan - March | April - June | July - Sept | Oct - Dec | Jan - March | April - June | July - Sept | Oct - Dec | |
| I.A.1.1 | STA | 1. on duty 2. T.S. 3. training 4. long term | A11 A11 A11 A11 | Complete | X | 0 | X | 0 | | | | | |
| | | | | <p>———— Submittal for info - No licensing action ————</p> | | | | | | | | | |
| I.A.1.2 | Shift Super. Resp. | New procedure | A11 | Complete | | | | | | | | | |
| I.A.1.3 | Shift Manning | 1. limit OT 2. minimum crew 3. T.S. | A11 A11 A11 | | X | | ◇ | X | | | | | |
| | | | | <p>X — No NRR action - OIE will verify</p> | | | | | | | | | |
| I.A.2.1 | Upgrade RO/SRO Training | 1. SRO exp. 2. SRO/RO exp 3. 3 mth trng. 4. modify trng 5. facility cert | A11 A11 A11 A11 A11 | No further NRC action required No further NRC action required No further NRC action required No further NRC action required No further NRC action required | □ | X | | | | | | | □ |
| I.A.2.3 | Trng. Program | Instructing SRO Superv. | A11 | No further NRC action required | | | | | | | | | |
| I.A.3.1 | Scope Crit. for Lic. Exams | 1. Inc. scope 2. Inc. grade 3. Sim. exam | A11 A11 A11 | No further NRC action required No further NRC action required | | | | | | | | | □ |
| I.C.1 | | 1. SB LOCA 2. Inadequate Core Cooling a. guidelines b. procedures 3. Transients & Accidents a. guidelines b. procedures | A11 A11 A11 A11 A11 A11 | Complete | X | | ◇ | * | X | | | | □ |
| | | | | <p>X ————— X ————— X —————</p> | | | | | | | | | |

Key
 ◇ Position/Criteria Developed
 * Issue Model Tech Specs
 X Licensee Submittal
 0 Tech Spec Comp (SER Issued)
 △ Review Complete (SER Issued)
 □ Installation/Implementation

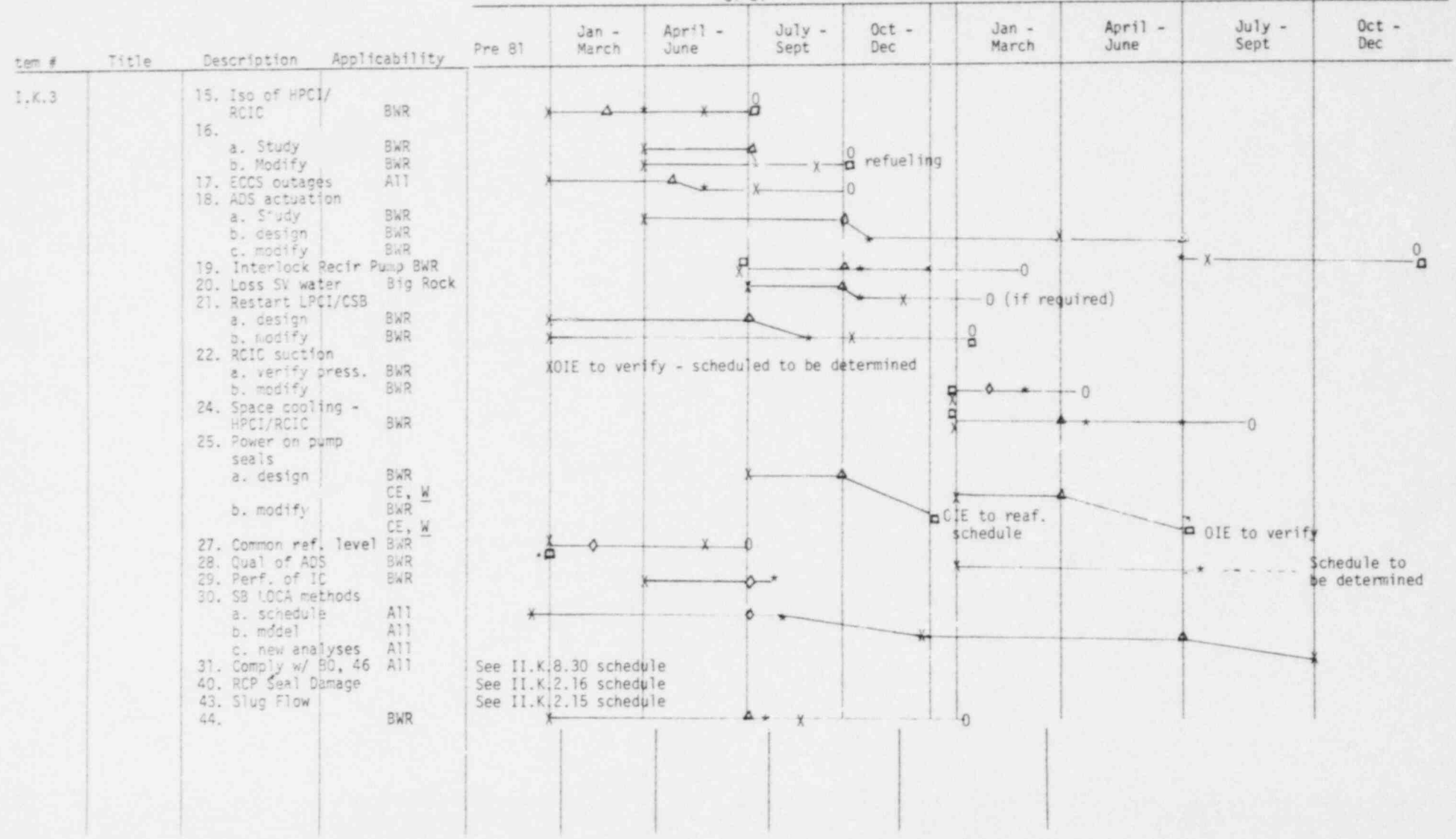




variable no later than 1/1/82

CY 81

CY 82



| Item # | Title | Description | Applicability | CY 81 | | | | CY 82 | | | | | |
|----------|-------------------|--|-------------------|----------|-------------|--------------|-------------|-----------|-------------|--------------|-------------|-----------|--------|
| | | | | Pre 81 | Jan - March | April - June | July - Sept | Oct - Dec | Jan - March | April - June | July - Sept | Oct - Dec | |
| I.K.3 | | 45. Manual depress. 46. Michelson concern 57. Manual Init of ADS | BWR BWR BWR | X | | △ | ● | | | | | | |
| II.A.1.1 | EP Short Term | Short Term | A11 | Complete | | | | | | | | | |
| II.A.1.2 | Upgrade Sys Func. | 1. Interim TSC, OSC 2. Design 3. Modify | A11 A11 A11 | Complete | | | | | | | | | |
| II.A.2 | Emer. Prep. | 1. Anp. E upgrade 2. MET dates | A11 A11 | X | | | | | □ | ● | | □ | □ |
| II.D.1.1 | Sys Integ | 1. Lead Rad 2. T.S. | A11 A11 | Complete | | | | | | | | | 6/1/83 |
| II.D.2.3 | In-plant Rad Mod. | 1. Interim 2. Upgrade Cap. | A11 A11 | Complete | □ | △ | ● | | | | | | |
| II.D.3.4 | Con. Room Habit. | 1. Review 2. Modify | A11 A11 | X | | | ● | ○ | | | | | |

MULTIPLANT ACTION NUMBER ASSIGNMENTS FOR THE
TMI ACTION PLAN REQUIREMENTS AND ASSIGNED
LEAD PROJECT MANAGERS

| Multiplant Action Number (Lead PM) | Title |
|---------------------------------------|---|
| MP F-1 (J. Neighbors) | I.A.1.1 Shift Technical Advisor |
| MP F-2 (W. Ross) | I.A.1.3 Shift Manning |
| MP F-3 (D. Wigginton) | I.A.2.1 Upgrading of RO and SRO Training |
| MP F-4 (D. Chaney) | I.C.1.2 Inadequate Core Cooling Guidelines and Procedures |
| MP F-6 (M. Grotenhuis) | I.C.5 Feedback of Operating Experience |
| MP F-7 (R. Licciardo) | I.C.6 Correct Performance of Operating Activities |
| MP F-8 (R. Caruso) | I.D.1 Control Room Design Review |
| MP F-9 (R. Snaider) | I.D.2 Safety Parameter Display System |
| MP-F-10 (J. Van Vliet) | II.B.1 RCS High Point Vents |
| MP F-11 (L. Engle) | II.B.2 Plant Shielding |

| Multipiant Action Number (Lead PM) | Title |
|---------------------------------------|--|
| MP F-12 (L. Engle) | II.B.3 Post accident Sampling |
| MP F-13 (D. Wigginton) | II.B.4 Training for Mitigating Core Damage |
| MP F-14 (C. Nelson) | II.D.1 RV and SV Testing |
| MP F-15 (P. Wagner) | II.E.1.1 AFW System Evaluation |
| MP F-16 (D. Garner) | II.E.1.2.1 AFW System Initiation |
| MP F-17 (D. Garner) | II.E.1.2.2 AFW System Flow Indication |
| MP F-18 (M. Fairtile) | II.E.4.1 Dedicated Hydrogen Penetrations |
| MP F-19 (E. Reeves) | II.E.4.2 Containment Isolation Dependability |
| MP F-20 (E. Conner) | II.F.1.1 Noble Gas Monitor |
| MP F-21 (E. Conner) | II.F.1.2 Iodine/Particulate Sampling |
| MP F-22 (M. Fairtile) | II.F.1.3 Containment High Range Monitor |

| Multiplant Action Number (Lead PM) | Title |
|---------------------------------------|---|
| MP F-23 (D. Dianni) | II.F.1.4 Containment Pressure Instrument |
| MP F-24 (P. Erickson) | II.F.1.5 Containment Water Level Instrument |
| MP F-25 (P. Erickson) | II.F.1.6 Containment Hydrogen Monitor |
| MP F-26 (J. Shea) | II.F.2 Instruments for Detection of Inadequate Core Cooling |
| MP F-27 (M. Padovan) | II.K.2.9 FMEA on ICS |
| MP F-28 (M. Padovan) | II.K.2.10 Safety Grade ARTS |
| MP F-29 (M. Padovan) | II.K.2.11 Continued Operator Training and Drilling |
| MP F-30 (G. Vissing) | II.K.2.13 Thermal-Mechanical Report |
| MP F-31 (D. DiIanni) | II.K.2.14 Lift Frequency of PORVs and SVs |
| MP F-32 (G. Vissing) | II.K.2.16 RCP Seal Damage |
| MP F-33 (D. DiIanni) | II.K.2.17 Potential for Voiding in RCS |

| Multiplant Action Number (Lead PM) | Title |
|---------------------------------------|--|
| MP F-35 (M. Fairtile) | II.K.2.20 System Response to SB LOCA |
| MP F-36 (T. Colburn) | II.K.3.1 Auto PORV Isolation |
| MP F-37 (T. Colburn) | II.K.3.2 Report on PORV Failures |
| MP F-38 (T. Colburn) | II.K.3.3 Reporting SV and RV Failures and Challenges |
| MP F-39 (R. Martin) | II.K.3.5 Auto Trip of RCPs |
| MP F-40 (C. Trammell) | II.K.3.9 PID Controller |
| MP F-41 (C. Trammell) | II.K.3.10 Anticipatory Trip Modifications |
| MP F-42 (C. Trammell) | II.K.3.12 Anticipatory Trip on Turbine Trip |
| MP F-43 (P. Polk) | II.K.3.13 HPCI and RCIC Initiation Levels |
| MP F-44 (T. Alexion) | II.K.3.14 Isolation Condenser Isolation Modification |
| MP F-45 (M. Williams) | II.K.3.15 Isolation of HPCI and RCIC Modification |

| Multiplant Action Number (Lead PM) | Title |
|---------------------------------------|---|
| MP F-46 (R. Clark) | II.K.3.16 Challenges and Failures of Relief Valves |
| MP F-47 (V. Rooney) | II.K.3.17 ECCS Outages |
| MP F-48 (R. Bevan) | II.K.3.18 ADS Actuation |
| MP F-49 (K. Eccleston) | II.K.3.19 Interlock Recirculation Pump Modification |
| MP F-50 (V. Rooney) | II.K.3.21 Restart of CSS and LPCI |
| MP F-51 (P. Polk) | II.K.3.22 RCIC Suction |
| MP F-52 (M. Williams) | II.K.3.24 Space Cooling for HPCI/RCIC Modifications |
| MP F-53 (P. Polk) | II.K.3.25 Power on Pump Seals |
| MP F-54 (J. Hannon) | II.K.3.27 Common Reference Level |
| MP F-55 (K. Eccleston) | II.K.3.28 Qualification of ADS Accumulators |

| Multiplant Action Number (Lead PM) | Title |
|---------------------------------------|---|
| MP F-56 (T. Alexion) | II.K.3.29 Performance of Isolation Condensers |
| MP F-57 (J. Hannon) | II.K.3.30 SB LOCA Methods |
| MP F-58 (B. Siegel) | II.K.3.31 Compliance with 10 CFR 50.46 |
| MP F-59 (B. Siegel) | II.K.3.44 Anticipated Transients with Single Failures |
| MP F-60 (R. Bevan) | II.K.3.45 Manual Depressurization |
| MP F-62 (R. Clark) | II.K.3.57 Manual Actuation of ADS |
| MP F-63 (R. Caruso) | III.A.1.2 Technical Support Center |
| MP F-64 (S. Nowlicki) | III.A.1.2 Operational Support Center |
| MP F-65 (T. Wambach) | III.A.1.2 Emergency Operations Facility |
| MP F-66 (R. Snaider) | III.A.1.2 Nuclear Data Link |

| Multiplant Action Number (Lead PM) | Title |
|---------------------------------------|---|
| MP F-67 (W. Paulson) | III.A.2.1 Emergency Plan Upgrade to Meet Rule |
| MP F-68 (J. Lombardo) | III.A.2.2 Meteorological Data Upgrade |
| MP F-69 (G. Requa) | III.D.3.3 Implant Radiation Monitoring |
| MP F-70 (P. O'Connor) | III.D.3.4 Control Room Habitability |

SECTION 3

L I C E N S I N G A C T I O N S B Y P O W E R R E A C T O R C O N D E N S E D M A N A G E M E N T R E P O R T

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: ARKANSAS 1

PLANT LOCATION: 6 MI WNW OF RUSSELLVILLE, AR
 DOCKET NUMBER: 050-00313
 ARCH/ENGINEER: BECH
 IE INSPECTOR: W. JOHNSON

LICENSED POWER: 2568 MWT
 DESIGN POWER: 0850 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: G. VISSING
 BRANCH CHIEF: R. REID
 LIC. ASSISTANT: R. INGRAM

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 07863 | ANO 1 - REACTOR CAVITY VESSEL SEAL RING PROBLEM | | 2 | 10/30/80 |
| 10539 | ANO 1 - CONTINGENCY PLAN REVIEW | | 1 | 11/01/80 |
| 11727 | ANO 1 - HI PRESSURE RPS SET POINT 2300, PORV 2450 | | 1 | 12/19/80 |
| 11795 | ANO 1 - FEEDWATER LINE CRACKS | | 2 | 01/12/81 |
| 07246 | ANO-1 STANDARD TECHNICAL SPECIFICATIONS | | 3 | 01/21/81 |
| 10316 | ANO 1 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 01/30/81 |
| 11728 | ANO 1 - LCO FOR LOSS OF EMERG. FEEDWATER | | 1 | 01/30/81 |
| 11241 | ANO 1 - FIRE PROTECTION - SUPPLEMENTAL SER | | 1 | 02/03/81 |
| 11826 | ANO 1 - TECH SPEC FOR ANTICIPATORY REACTOR TRIPS | | 1 | 02/05/81 |
| 11811 | ANO 1 - REFUELING FOR CYCLE 5 OPERATION | | 1 | 03/09/81 |
| 12199 | ANO-1 - UPGRADE EMERGENCY PLAN | | 1 | 04/01/81 |
| 12877 | ANO 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES ISOLATION VALVES | | 1 | 04/20/81 |
| 42120 | ANO 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS CAPABILITY TECH SPECS | | 2 | 05/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 11266 | ANO 1 - MECHANICAL SNUBBERS | | 2 | |
| 11267 | ANO 1 - HYDRAULIC SNUBBERS | | 3 | |
| 11480 | ANO 1 - REQUEST FOR EXEMPTION - APP J TESTING OF VALVES OF VALVES | | 2 | |
| 12320 | ANO 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ARKANSAS 1 | | 3 | |
| 43632 | ANO 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 42650 | ANO 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43036 | ANO 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY SAFETY SYSTEMS | | 2 | |
| 10844 | ANO 1 - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 06806 | ANO 1 - OVERPRESSURIZATION | | 2 | |
| 08371 | ANO 1 - GENERIC - PWR MODERATOR DILUTION | | 3 | |
| 43265 | ANO 1 - B&W ACCIDENT INDUCED NEUTRON FLUX ERRORS (MULTI-PLANT ITEM B-64) | | 3 | |
| 43886 | ARKANSAS UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 08105 | ANO 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 06038 | ANO 1 - PWR SECONDARY WATER CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | |
| 42901 | ANO 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 07087 | ANO-1 TENDON SURVEILLANCE PROBLEMS | | 2 | |
| 42720 | ANO 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43549 | ANO 1 - SEISMIC QUALIFICATION OF AUXILIARY FEEDWATER SYSTEMS | | 1 | |
| 10378 | ANO 1 - LOSS OF 125 DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 07862 | ANO 1 - PRESSURE VESSEL SUPPORT (ASYMMETRICAL LOADS) | | 2 | |
| 10537 | ANO 1 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10542 | ANO 1 - VITAL AREA ANALYSIS | | 1 | |
| 11236 | ANO 1 - REVISED IST - TESTING | | 1 | |
| 12374 | ANO 1 - REVISE ENVIRONMENTAL (T.S. APPENDIX B) | | 3 | |
| 12578 | ANO 1 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 07971 | ANO 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 07521 | ANO 1 - REVISED ISI | | 1 | |
| 08447 | ANO-1 S/G SUPPORTS LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 42514 | ANO 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| 08701 | ANO 1 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--|-----------------|-----------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44000 | AND 1 - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | <u>TARGET</u> |
| 44068 | AND 1 - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44139 | AND 1 - NUREG-0737 | I.A.2.1.4, UPGRADING OF PO AND SRO TRAINING | | |
| 44210 | AND 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44351 | AND 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44419 | AND 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44489 | AND 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44550 | AND 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44707 | AND 1 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44749 | AND 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44757 | AND 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATION | | |
| 44828 | AND 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44899 | AND 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44970 | AND 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45041 | AND 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45112 | AND 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45182 | AND 1 - NUREG-0737 | II.K.2.10, SAFETY GRADE ARTS | | |
| 45189 | AND 1 - NUREG-0737 | II.K.2.11, OPERATOR TRAINING AND DRILLING | | |
| 45196 | AND 1 - NUREG-0737 | II.K.2.13, THERMAL-MECHANICAL REPORT | | |
| 45203 | AND 1 - NUREG-0737 | II.K.2.14, LIFT FREQUENCY OF PORV'S AND SV'S | | |
| 45267 | AND 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45313 | AND 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45321 | AND 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45434 | AND 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45607 | AND 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45803 | AND 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45923 | AND 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 45994 | AND 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46066 | AND 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46138 | AND 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46210 | AND 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46282 | AND 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46353 | AND 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46424 | AND 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| 42617 | AND 1 - B&W PLANTS | - ITEM II K.2.16 - RCP SEAL DAMAGE ON LOSS OF OFFSITE POWER | 1 | |
| 42634 | AND 1 - AFW SYSTEM | RELIABILITY ANALYSIS & UPGRADE | 1 | |
| 42619 | AND 1 - B&W PLANTS | - ITEM II K.2.17 POTENTIAL VOIDING DURING ANTICIPATED TRANSIENTS | 1 | |
| 42621 | AND 1 - B&W PLANTS | ITEM II K.2.20 ANALYSIS OF SBLOCA WHICH DEPRESSURIZES TO PORV SET PT | 1 | |
| 42842 | AND 1 - ATOG | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42988 | AND 1 - REORGANIZATION CHANGE | | 2 | <u>COMPLETE</u> 11/14/80 |
| 42456 | AND 1 - SAFETY GRADE ANTICIPATORY REACTOR TRIP SYSTEM | | 1 | 12/16/80 |
| 12870 | AND 1 - DIVERSE REACTOR BUILDING ISOLATION T.S. | | 1 | 12/19/80 |
| 42238 | AND-1 CAT. 'A' TS FOR CONTAINMENT ISOLATION | | 1 | 12/19/80 |
| 42267 | AND 1 - FA HOLDDOWN SPRINGS | | 1 | 01/29/81 |
| 42236 | AND 1 - AECAT. 'A' TS FOR SHIFT TECH. ADVISOR | | 1 | 01/30/81 |
| 42237 | AND 1 - CAT. 'A' TS FOR AUXILIARY FEEDWATER .CO | | 1 | 01/30/81 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 1

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|---|---|----------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> (CONTINUATION) | | | | |
| 42239 | ANO 1 - CAT. 'A' FOR INSTRUMENTATION SURFACED EQUATE CORE COOLING | | 1 | <u>COMPLETE</u> 01/30/81 |
| 42240 | ANO 1 - CAT. 'A' TS FOR VALVE POSITION INDICATION | | 1 | 01/30/81 |
| 42241 | ANO 1 - CAT. 'A' TS FOR EMERGENCY POWER SUP. | | 1 | 01/30/81 |
| 42957 | ANO 1 - NUREG 1578 CATEGORY A TECHNICAL SPECIFICATIONS | | | 01/30/81 |
| 43189 | ANO 1 - CHANGE IN FREQUENCY IN TAKIN SAMPLES OF FISH | | 2 | 01/30/81 |
| 42618 | ANO 1 - B&W PLANTS ITEM II K.2.15 OTSG SLUG FLOW EFFECTS | | 1 | 02/03/81 |
| 43240 | ANO 1 - RESPONSE TO LETTER (11/28/80) CONCERNING IE 79-05A & 79-05B | | 1 | 02/05/81 |
| 43313 | ANO-1 EXCEPTIONS TO IE BULLETINS (79-05A & 79-05B) | | 1 | 02/05/81 |
| 43384 | ANO 1 - RESPONSE CONCERNING RESPONSES TO IE 78-05 11/13 | | 1 | 02/05/81 |
| 13146 | ANO-1 PLANT RESPONSE TO LOSS OF OFF SITE POWER EVENT OF 4/7/80 | | 1 | 02/15/81 |
| 42150 | ANO 1 - FIVE ADDITIONAL TMI-2 RELATED REQUIREMENTS RELATED REQUIREMENTS | | 1 | 02/25/81 |
| 42620 | ANO 1 - B&W PLANTS - ITEM II K.2.19 BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW | | 1 | 02/28/81 |
| 43094 | ANO 1 - TECH SPEC CHANGE RELATING TO RADIATION PROTECTION | | 1 | 03/10/81 |
| 42308 | ANO 1 - T.S. CHANGE-FIRE DETECTION INSTRUMENTATION | | 1 | 03/20/81 |
| 43689 | ANO 1 - REQUEST FOR EXEMPTION FROM 10 CFR 50 APP R SEC III G&L | | 1 | 04/29/81 |
| 43193 | ANO 1 - CONTAINMENT PURGE RESTRICTED DURING POWER OPERATION | | 1 | 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 11494 | ANO 1 - INSPECTION OF FLY WHEELS OF RCP MOTORS | | 1 | <u>TARGET</u> |
| 41013 | ANO 1 - IREP | | 1 | |
| 42791 | ANO 1 - CONFIRMATORY ORDER - ICS RELIABILITY ANALYSIS | | 1 | |
| 42857 | ANO 1 - T.S. FOR CONTROL GRADE ANTICIPATORY TRIP ON LOF W & T.T | | 1 | |
| 42929 | ANO 1 - CLAM SHELLS IN SW SYSTEM PROBLEM | | 1 | |
| 42930 | ANO 1 - OTSG LEVEL DIFFERENTIAL | | 1 | |
| 42982 | ANO 1 - PHYSICS METHODS FOR MIDDLE SOUTH PWR'S | | 1 | |
| 43376 | ANO 1 - REVISION TO SECURITY PLAN | | 1 | |
| 43623 | ANO 1 - ANALYSIS OF BORON DILUTION EVENT FOR COLD SHUTDOWN | | 3 | |
| 42847 | ANO 1 - REVISED TECH. SPEC. LIMITS FOR T/P DURING EMERGENCY | | 1 | |
| 42148 | ANO 1 - REVIEW OF WASTE GAS SYSTEM | | 1 | |
| 43425 | ANO 1 - ITEM II K.2.13 THERMAL-MECH EFFECTS OF HPI ON VESSEL | | 1 | |
| 43503 | ANO 1 - T.S FOR DEGRADED GRID VOLTAGE MODIFICATIONS | | 1 | |
| 42151 | ANO 1 - T.S DEFINING OPERABILITY OF S.S | | 1 | |
| 43691 | ANO 1 - REQUEST FOR EXEMPTION FROM MEETING SCHEDULE OF 50.48 | | 1 | |
| 43565 | ANO 1 - T.S. CHANGES ON ADMINISTRATIVE TITLES | | 2 | |
| 43595 | ANO 1 - T.S. FOR OPERABILITY OF PORV | | 1 | |
| 43383 | ANO 1 - RESPONSE RELATED TO REACTOR VESSEL SURVEILLANCE | | 3 | |
| 43554 | ANO 1 - REASSESSMENT OF SEC. III.G, III.J & III.O OF APP.R | | 1 | |
| 43548 | ANO 1 - EMERG PROCEDURES & TRAINING FOR STATION BLACKOUT EVENT | | 1 | |
| 43771 | ANO 1 - SITE INVESTIGATION OF CYCLE 4 FUEL FAILURES | | 1 | |
| 43163 | ANO 1 - INSTRUMENTATION FOR DETECTION OF INADEQUATE CORE COOLING | | 1 | |
| 43164 | ANO 1 - WASTE GAS H/O LIMITS T.S. CHANGE | | 1 | |
| 12790 | ANO 1 - ADEQUACY OF ELECT. DIST SYSTEM | | 3 | |
| 42595 | ANO 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 07194 | ANO 1 - CONTROL ROOM ISOLATION | | 3 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: ARKANSAS 2

PLANT LOCATION: 6 MI WNW OF RUSSELLVILLE, AR
 DOCKET NUMBER: 050-00368
 ARCH/ENGINEER: BECH
 IE INSPECTOR: W. JOHNSON

LICENSED POWER: 2815 MWT
 DESIGN POWER: 0912 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: R. MARTIN
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12061 | ANO 2 - LEAKING OF CONTAINMENT ISOLATION VALVES WITH RESILIENT SEATS (B-20) | | 1 | <u>COMPLETE</u> 10/02/80 |
| 12254 | ANO 2 - MULTIPLE EQUIPMENT FAILURES | | 2 | 10/21/80 |
| 12056 | ANO 2 - PRESSURE TEMP LIMIT T.S. | | 1 | 11/13/80 |
| 10540 | ANO 2 - CONTINGENCY PLAN REVIEW | | 1 | 11/19/80 |
| 12298 | ANO 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ANO-2 | | 1 | 12/01/80 |
| 12057 | ANO 2 - REACTOR VESSEL SEAL RING MISSILE (D-04) | | 3 | 01/21/81 |
| 12094 | ANO 2 - SPENT FUEL POOL EXPANSION (E-1) | | 2 | 02/02/81 |
| 11390 | ANO 2 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE SURVEILLANCE | | 1 | 02/27/81 |
| 12360 | ANO-2 LESSONS LEARNED IMPLEMENTATION SHORT TERM | | 3 | 03/03/81 |
| 43024 | ANO 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 1 | 03/03/81 |
| 12878 | ANO 2 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION | | 2 | 03/03/81 |
| | | | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08159 | ANO 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | <u>TARGET</u> |
| 10536 | ANO 2 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10543 | ANO 2 - VITAL AREA ANALYSIS | | 1 | |
| 12062 | ANO 2 - INSERVICE TESTING PROGRAM (A-14) | | 1 | |
| 12067 | ANO 2 - ASYMMETRIC LOCA LOADS - PUR (D-10) CHANNEL FACTOR (B-13) | | 1 | |
| 12068 | ANO 2 - OVER PRESSURE MITIGATING SYSTEM | | 2 | |
| 12069 | ANO 2 - SURVEILLANCE FOR HYDRAULIC SHUBBERS (B-17) | | 3 | |
| 12071 | ANO 2 - SECONDARY WATER CHEMISTRY MONITORING (C-1) | | 3 | |
| 12075 | ANO 2 - FUEL PERFORMANCE-FISSION GAS RELEASE (D-11) | | 3 | |
| 12076 | ANO 2 - FUEL ROD BOW EFFECTS ON NUCLEAR ENTHALPHY HOT CHANNEL FACTOR (B-13) | | 3 | |
| 12078 | ANO 2 - 1ST RELOAD | | 3 | |
| 12083 | ANO 2 - FIRE PROTECTION-LICENSE CONDITION RESOLUTION | | 1 | |
| 12090 | ANO 2 - RESISTANCE TEMP DETECTOR RESPONSE (B-36) | | 1 | |
| 12093 | ANO 2 - LOSS OF 125V DC BUS VOLTAGE (B-21) | | 3 | |
| 12097 | ANO 2 - APPENDIX I - ALARA | | 3 | |
| 12542 | ANO 2 - MISC. CHANGES TO FIRE PROTECTION T.S. | | 3 | |
| 12946 | ANO 2 - HEAVY LOADS NEAR SPENT FUEL STORAGE | | 1 | |
| 42107 | ANO 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS CAPABILITY TECH SPECS | | 2 | |
| 42651 | ANO 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43633 | ANO 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 2 | |
| 12060 | ANO 2 - CEA GUIDE TUBE SURVEILLANCE PROGRAM | | 1 | |
| 12358 | ANO 2 - TMI RELATED ACTIVITIES-BULLETINS & ORDERS | | 3 | |
| 42721 | ANO 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 1 | |
| 42070 | ANO 2 - ANALYSIS OF TURBINE DISCS IN G.E. TURBINES | | 3 | |
| 12087 | ANO 2 - CONTAINMENT FAK TESTING - APPENDIX J | | 3 | |
| 43887 | ARKANSAS UNIT 2 - ION BLACKOUT PROCEDURES & TRAINING | | 3 | |
| 42889 | ANO 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43128 | ANO 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 12070 | ANO 2 - T.S. SURVEILLANCE MECHANICAL SHUBBERS (B-22) (B-22) | | 2 | |
| 42583 | ANO 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 2 | |
| 12066 | ANO 2 - PUMP SUPPORT - FRACTURE TOUGHNESS (C-6) | | 1 | |
| 42496 | ANO 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 2 | |
| | | | 1 | |

DATA AS OF - 05/31/91

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|------------------------|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 44001 | ANO 2 - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44069 | ANO 2 - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44140 | ANO 2 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44211 | ANO 2 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44281 | ANO 2 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44352 | ANO 2 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44420 | ANO 2 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44490 | ANO 2 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44561 | ANO 2 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44660 | ANO 2 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44702 | ANO 2 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44741 | ANO 2 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44758 | ANO 2 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44829 | ANO 2 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44900 | ANO 2 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44971 | ANO 2 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45042 | ANO 2 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45113 | ANO 2 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45259 | ANO 2 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45305 | ANO 2 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45322 | ANO 2 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45426 | ANO 2 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45608 | ANO 2 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45804 | ANO 2 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45924 | ANO 2 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 45995 | ANO 2 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46067 | ANO 2 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46139 | ANO 2 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46211 | ANO 2 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46283 | ANO 2 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46354 | ANO 2 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46425 | ANO 2 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42986 | ANO 2 - REORGANIZATION | | 2 | 11/14/80 |
| 12095 | ANO 2 - CONTAINMENT RADIATION MONITOR | | 1 | 02/27/81 |
| 42312 | ANO 2 - T.S. CHANGE REQUEST POST ACCIDENT RADIATION MONITORS | | | 02/27/81 |
| 42351 | ANO 2 - STD TECH SPEC DEFINITION OF OPERABILITY | | 1 | 03/03/81 |
| 43093 | ANO 2 - TECH. SPEC. CHANGE RELATING TO RADIATION PROTECTION | | 2 | 03/10/81 |
| 42353 | ANO 2 - FIRE DETECTION INSTRUMENTATION | | 2 | 03/23/81 |
| 42831 | ANO 2 - T.S. CHANGE REQUEST - CONTAINMENT COOLERS | | 1 | 03/30/81 |
| 42350 | ANO 2 - FIVE ADDITIONAL TMI-2 ITEMS | | 1 | 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12064 | ANO 2 - MAIN FEEDWATER MODIFICATION | | 1 | |
| 12081 | ANO 2 - VERIFICATION OF TRANSIENT ANALYSIS CODE CESEC CESEC | | 1 | |
| 12084 | ANO 2 - CORE PROTECTION CALCULATOR SYSTEM (CPCS) RESOLVE POSITIONS 1, 5, 12 | | 1 | |
| 12088 | ANO 2 - INSTRUMENT TRIP SET POINTS DRIFT ALLOWANCE | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| | (CONTINUATION) | | | |
| 12466 | AND 2 - T.S. CHANGE 2 SEISMIC INSTRUMENTATION | | 2 | |
| 12532 | AND 2 - DELTA P DURING INADVERTENT CONT. SPRAY ACTIVATION | | 2 | |
| 12543 | AND 2 - HIGH PRESSURIZER TRIP SET POINT | | 2 | |
| 13003 | AND 2 - SOFTWARE CHANGES TO CPCS | | 1 | |
| 42542 | AND 2 - EMERGENCY DIESEL-GENERATOR LOAD BLOCK TIMING CRITERIA | | 2 | |
| 42949 | AND 2 - PRESSURIZER SAFETY VALVE HYDROSETTING TIME | | 2 | |
| 42981 | AND 2 - CPCS/PLANT COMPUTER DATALINKS | | 2 | |
| 42983 | AND 2 - PHYSICS METHOD FOR MIDDLE SOUTH PWRS | | 2 | |
| 43165 | AND 2 - WASTE GAS H/O LIMITS T.S. CHANGE | | 2 | |
| 43199 | AND 2 - BORON DILUTION/SHUTDOWN MARGIN | | 2 | |
| 43861 | ARKANSAS NUCLEAR 1 - REACTOR COOLANT PUMP 5/5/81 WELD INSPECTION | | | |
| 12063 | AND-2 EFW PMP RESPONSE TIME - T.S. CHANGE | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: BEAVER VALLEY 1

PLANT LOCATION: 5 MI E OF E. LIVERPOOL, OH
 DOCKET NUMBER: 050-00334
 ARCH/ENGINEER: S&W
 IE INSPECTOR: D. BECKMAN

LICENSED POWER: 2660 MWT
 DESIGN POWER: 0852 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. CHANEY
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 08053 | BEAVER VALLEY 1 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 42330 | BEAVER VALLEY 1 - MASONRY WALL DESIGN REVIEW | | 2 | 11/26/80 |
| 11865 | BEAVER VALLEY 1 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 02/03/81 |
| 10260 | BEAVER VALLEY 1 - CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATION | | 3 | 02/04/81 |
| 10323 | BEAVER VALLEY 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | 02/04/81 |
| 43002 | BEAVER VALLEY 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 02/05/81 |
| 11685 | BEAVER VALLEY 1 - THREE MILE ISLAND FOLLOW-UP WORK | | 1 | 02/07/81 |
| 10476 | BEAVER VALLEY 1 - GUARD TRAINING PLAN REVIEW | | 1 | 02/11/81 |
| 12395 | BEAVER VALLEY 1 - LESSONS LEARNED IMPLEMENTATION | | 1 | 02/11/81 |
| 10007 | BEAVER VALLEY 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | 03/03/81 |
| 12991 | BEAVER VALLEY 1 - EVAL OF ELEC OVERRIDE/BYPASS OF SAFETY ACTUATION SIGNALS | | 1 | 03/18/81 |
| 10475 | BEAVER VALLEY 1 - VITAL AREA ANALYSIS | | 1 | 03/27/81 |
| 12162 | BEAVER VALLEY 1 - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 12769 | BEAVER VALLEY 1 - REORGANIZATION OF FACILITY MGMT | | 3 | 04/16/81 |
| 12922 | BEAVER VALLEY 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 42092 | BEAVER VALLEY 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12263 | BEAVER VALLEY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE BEAVER VALLEY 1 | | 3 | |
| 43646 | BEAVER VALLEY 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 42652 | BEAVER VALLEY 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 07240 | BEAVER VALLEY 1 - LOW HEAD SAFETY INJECTION PUMPS DEGRAD. VIBS | | 3 | |
| 06714 | BEAVER VALLEY 1 - RCS OVERPRESSURIZATION PROTECTION | | 2 | |
| 13110 | BEAVER VALLEY 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 42722 | BEAVER VALLEY 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BU'LETIN 80-06 | | 3 | |
| 12598 | BEAVER VALLEY 1 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 08597 | BEAVER VALLEY 1 - REVIEW OF ASYMMETRIC LOCA LOADS | | 2 | |
| 10386 | BEAVER VALLEY 1 - N-1 LOOP OPERATION | | 3 | |
| 10477 | BEAVER VALLEY 1 - TECH SPECS FOR HYDRAULIC SNUBBERS | | 3 | |
| 10478 | BEAVER VALLEY 1 - TECH SPECS FOR MECHANICAL SNUBBERS | | 2 | |
| 08132 | BEAVER VALLEY 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 43911 | BEAVER VALLEY UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42858 | BEAVER VALLEY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 06363 | BEAVER VALLEY 1 - INSERVICE TESTING 10 CFR 50.55A(G) RELIEF FOR PUMP AND VALVETEST REQUIREMENTS (IST) | | 1 | |
| 07972 | BEAVER VALLEY 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42553 | BEAVER VALLEY 1 - LONG TERM - REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 11111 | BEAVER VALLEY 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 43129 | BEAVER VALLEY 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 08594 | BEAVER VALLEY 1 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 42459 | BEAVER VALLEY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BEAVER VALLEY 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|-----------------------|------------------------------|---|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44002 | BEAVER VALLEY 1 - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44070 | BEAVER VALLEY 1 - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44191 | BEAVER VALLEY 1 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44212 | BEAVER VALLEY 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44282 | BEAVER VALLEY 1 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44353 | BEAVER VALLEY 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44421 | BEAVER VALLEY 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44491 | BEAVER VALLEY 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44562 | BEAVER VALLEY 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44632 | BEAVER VALLEY 1 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44674 | BEAVER VALLEY 1 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44711 | BEAVER VALLEY 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44759 | BEAVER VALLEY 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44830 | BEAVER VALLEY 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44901 | BEAVER VALLEY 1 - NUREG-0737 | II.F.1.1, NCBLE GAS MONITOR | | |
| 44972 | BEAVER VALLEY 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45043 | BEAVER VALLEY 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45114 | BEAVER VALLEY 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45229 | BEAVER VALLEY 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45275 | BEAVER VALLEY 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45323 | BEAVER VALLEY 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45392 | BEAVER VALLEY 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45441 | BEAVER VALLEY 1 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45471 | BEAVER VALLEY 1 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45501 | BEAVER VALLEY 1 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45609 | BEAVER VALLEY 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45805 | BEAVER VALLEY 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45925 | BEAVER VALLEY 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 45996 | BEAVER VALLEY 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46068 | BEAVER VALLEY 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46140 | BEAVER VALLEY 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46213 | BEAVER VALLEY 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46284 | BEAVER VALLEY 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46355 | BEAVER VALLEY 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46426 | BEAVER VALLEY 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|-----------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42948 | BEAVER VALLEY 1 - ISOLATION OF CONTROL ROOM | | | <u>COMPLETE</u> 10/15/80 |
| 43186 | BEAVER VALLEY 1 - CONTROL ROD INDICATOR CHANGE OPERABILITY | | | 11/15/80 |
| 11055 | BEAVER VALLEY 1 - CONTAINMENT WELD LINER VENTING DURING ILRT | | 1 | 11/28/80 |
| 42924 | BEAVER VALLEY 1 - CLARIFICATION OF "OPERABLE" | | | 11/23/80 |
| 43063 | BEAVER VALLEY 1 - REMOVAL OF HYDRAULIC SNUBBERS | | | 11/28/80 |
| 12902 | BEAVER VALLEY 1 - FLAME GOUGE INTO MAIN STEAM LINE | | | 01/13/81 |
| 42387 | BEAVER VALLEY 1 - CONTAINMENT TEMP/PRESS SETPOINTS | | | 02/02/81 |
| 11984 | BEAVER VALLEY 1 - PUMP SUPPORT DESIGN ALLEGATION | | 2 | 03/16/81 |
| 42623 | BEAVER VALLEY 1 - STEAM GENERATOR LOW-LOW TRIP SET POINT | | 1 | 03/20/81 |
| 43615 | BEAVER VALLEY 1 - USE OF CODE N-210 FOR ISI RELIEF | | | 03/30/81 |
| 43846 | BEAVER VALLEY 1 - PRESSURIZER SAFETY VALVE DISCHARGE LINE MODIFICATIONS | | 1 | 05/12/81 |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BEAVER VALLEY 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|---|---|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> (CONTINUATION) | | | | <u>COMPLETE</u> |
| 43586 | BEAVER VALLEY 1 - EXPAND ANALOG ROD POSITION INDICATOR DEVIATION LIMITS | | | 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 43868 | BEAVER VALLEY 1 - ENVIRONMENTAL REPORT REVIEW | | | |
| 43609 | BEAVER VALLEY 1 - AFW TECH SPEC CHANGE | | | |
| 13049 | BEAVER VALLEY 1 - RECIRC SPRAY HEAT EXCHANGER INTEGRITY | | 1 | |
| 43748 | BEAVER VALLEY 1 - REORGANIZATION-ESTABLISHMENT OF NUCLEAR DIVISION | | 1 | |
| 06605 | BEAVER VALLEY 1 - SOIL STUDY | | 1 | |
| 11011 | BEAVER VALLEY 1 - TURBINE AND LOW FEEDWATER TRIP CHANGES | | 1 | |
| 42636 | BEAVER VALLEY 1 - REANALYSIS OF SAFETY RELATED PIPING SYSTEMS | | 1 | |
| 43124 | BEAVER VALLEY 1 - REVISION OF PHYSICAL SECURITY PLAN | | 1 | |
| 08595 | BEAVER VALLEY 1 - X 17 FUEL PERFORMANCE | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: BIG ROCK POINT 1

PLANT LOCATION: 4 MI NE OF CHARLEVOIX, MICH
 DOCKET NUMBER: 050-00155
 ARCH/ENGINEER: BECH
 IE INSPECTOR: G. WRIGHT

LICENSED POWER: 0240 MWT
 DESIGN POWER: 0072 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: W. PAULSON
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|---|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10063 | BIG ROCK POINT - ENHANCED FISSION PRODUCT RELEASE FOR HIGH BURNUP LWR FUEL | | | <u>COMPLETE</u> 10/01/80 |
| 10499 | BIG ROCK POINT - CONTINGENCY PLAN REVIEW | | 1 | 10/01/80 |
| 07973 | BIG ROCK POINT - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | 12/22/80 |
| 42214 | BIG ROCK POINT - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | 01/09/81 |
| 11096 | BIG ROCK POINT - FIRE PROTECTION SLR SUPPLEMENT | | 1 | 02/06/81 |
| 10873 | BIG ROCK POINT - OFFGAS EXPLOSION PREVENTION REVIEW | | | 02/13/81 |
| 12284 | BIG ROCK POINT - HELB AND CONSEQUENTIAL SYSTEM FAILURE BIG ROCK POINT | | 3 | 02/18/81 |
| 07767 | BIG ROCK POINT - GUARD TRAINING PLAN REVIEW | | 1 | 02/25/81 |
| 10280 | BIG ROCK POINT - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 02/25/81 |
| 11141 | BIG ROCK POINT - BWR CORE SPRAY PERFORMANCE STEAM | | 3 | 02/26/81 |
| 11823 | BIG ROCK POINT - SPENT FUEL POOL EXPANSION | | 1 | 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12781 | BIG ROCK POINT - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | <u>TARGET</u> |
| 43044 | BIG ROCK POINT - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 08312 | BIG ROCK POINT - 10 CFR 50.55A(G) - INSERVICE INSPECTION - GENERIC | | 1 | |
| 08096 | BIG ROCK POINT - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08679 | BIG ROCK POINT - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 10342 | BIG ROCK POINT - LOSS OF 125 DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 12234 | BIG ROCK POINT - SPENT FUEL POOL HEARING | | 1 | |
| 08441 | BIG ROCK POINT - MECHANICAL SNUBBERS | | 2 | |
| 08442 | BIG ROCK POINT - HYDRAULIC SNUBBERS | | 3 | |
| 11269 | BIG ROCK POINT - INSERVICE TESTING (IST) | | 1 | |
| 10502 | BIG ROCK POINT - VITAL AREA ANALYSIS | | 1 | |
| 08284 | BIG ROCK POINT - FUEL HANDLING ACCIDENT CONTAINMENT | | 2 | |
| 42723 | BIG ROCK POINT - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 12423 | BIG ROCK POINT - LESSONS LEARNED IMPLEMENTATION | | 1 | |
| 42653 | BIG ROCK POINT - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 12630 | BIG ROCK POINT - EMERGENCY PLANNING & REVISIONS | | 1 | |
| 43982 | BIG ROCK POINT - EVAL OF BULLETIN 79-08 | | | |
| 43905 | BIG ROCK POINT - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43715 | BIG ROCK POINT - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42907 | BIG ROCK POINT - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42607 | BIG ROCK POINT - LONG TERM REV CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42515 | BIG ROCK POINT - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u> | | | | |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44003 | BIG ROCK POINT - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44071 | BIG ROCK POINT - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44142 | BIG ROCK POINT - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44213 | BIG ROCK POINT - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44283 | BIG ROCK POINT - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BIG ROCK POINT 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|--------------------------------------|-----------------------------|---|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS (CONTINUATION)</u> | | | | |
| 44354 | BIG ROCK POINT - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44422 | BIG ROCK POINT - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44492 | BIG ROCK POINT - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44563 | BIG ROCK POINT - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44760 | BIG ROCK POINT - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44831 | BIG ROCK POINT - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44902 | BIG ROCK POINT - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44973 | BIG ROCK POINT - NUREG-0737 | II.F.1.2, IODINE/PARTICULATE SAMPLING | | |
| 45044 | BIG ROCK POINT - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45115 | BIG ROCK POINT - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45324 | BIG ROCK POINT - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45531 | BIG ROCK POINT - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45559 | BIG ROCK POINT - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45583 | BIG ROCK POINT - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45610 | BIG ROCK POINT - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45679 | BIG ROCK POINT - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45703 | BIG ROCK POINT - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45727 | BIG ROCK POINT - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45751 | BIG ROCK POINT - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45775 | BIG ROCK POINT - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45806 | BIG ROCK POINT - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45875 | BIG ROCK POINT - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45899 | BIG ROCK POINT - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45926 | BIG ROCK POINT - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 45997 | BIG ROCK POINT - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46069 | BIG ROCK POINT - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46141 | BIG ROCK POINT - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46212 | BIG ROCK POINT - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46285 | BIG ROCK POINT - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46356 | BIG ROCK POINT - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46427 | BIG ROCK POINT - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>COMPLETE</u> |
|--------------------------|---|-----------------------|-----------------|---|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42991 | BIG ROCK POINT - TECH SPEC CHANGE REQUEST-NEW FUEL BUNDLES WITH CENTER ROD POSITION AUXILIARY NEUTRON SOURCES | | 1 | 01/12/81 |
| 42812 | BIG ROCK POINT - CONTAINMENT TEMPERATURE LEVEL FOLLOWING POSTULATED STEAM LINE BREAK | | 1 | 01/13/81 |
| 06738 | BIG ROCK POINT - RDS BATTERY RED SPECIFIC GRAVITY FROM 1.20 TO 1.5 | | 1 | 01/16/81 |
| 07796 | BIG ROCK POINT - DELETE CONTROL RAD. WITHDRAWAL LIMIT | | 1 | 01/16/81 |
| 43059 | BIG ROCK POINT - FIRE PROTECTION SYSTEM AUTOMATIC ISOLATION VALVE SURVEILLANCE REQUIREMENTS | | 1 | 01/19/81 |
| 43181 | BIG ROCK POINT - TECHNICAL SPECIFICATION CHANGE REQUEST CHANGE IN CONTROL ROD DRIVE SYSTEM ISOLATION BOUNDARY | | 1 | 01/23/81 |
| 11933 | BIG ROCK POINT - ADMIN. CONTROLS-CHG. IN CPC ORG. | | 1 | 03/06/81 |
| 12479 | BIG ROCK POINT - ADMINISTRATIVE CONTROLS CLARIFICATIONS | | 1 | 03/06/81 |
| 07030 | BIG ROCK POINT - D.C. BATTERIES | | 1 | 03/31/81 |
| 08035 | BIG ROCK POINT - DIESEL PUMP BATTERY | | 1 | 03/31/81 |
| 08055 | BIG ROCK POINT - TECH SPECS FOR D.C. POWER SUPPLIES | | 1 | 03/31/81 |
| 43470 | BIG ROCK POINT - TECHNICAL SPECIFICATION CLARIFICATIONS AND CORRECTIONS | | 2 | 05/14/81 |
| 43577 | BIG ROCK POINT - TECH. SPEC. CHANGE TO PROVIDE FOR USE OF TLD'S FOR ENVIRONMENTAL MONITORING | | 2 | 05/14/81 |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BIG ROCK POINT 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 10755 | BIG ROCK POINT - LOCA ANALYSIS | | 1 | <u>TARGET</u> |
| 11168 | BIG ROCK POINT - REACTOR PHYSICS METHODOLOGY | | 1 | |
| 42327 | BIG ROCK POINT - OPERABILITY & SURVEILLANCE TESTING REQMTS FOR FIRE PROTECTION EQUIPMENT (LIC. RESUBMITTING) | | 1 | |
| 43483 | BIG ROCK POINT - CHANGES TO SECTION 6 OF THE TECH. SPECS ADMINISTRATIVE CONTROLS | | 2 | |
| 12979 | BIG ROCK POINT - RISK ASSESSMENT | | 1 | |
| 43704 | BIG ROCK POINT - DESIGN DESCRIPTION OF ALTERNATE SAFE SHUTDOWN CAPABILITY | | 1 | |
| 43578 | BIG ROCK POINT - COMMISSIONERS GILINSKY AND BRADFORD'S REQUEST FOR DATA ON CONTAINMENT ISOLATION VALVES | | 3 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: BROWNS FERRY 1

PLANT LOCATION: 10 MI NW OF DECATUR, ALA
 DOCKET NUMBER: 050-00259
 ARCH/ENGINEER: TVA
 IE INSPECTOR: R. SULLIVAN

LICENSED POWER: 3293 MWT
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. CLARK
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|--|-------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42230 | BROWNS FERRY 1 - BWR SCRAM SCHEDULE VOLUME CAPABILITY | | 1 | 01/09/81 |
| 11364 | BROWNS FERRY 1 - REACTOR VESEL BELTLINE MATERIAL SURVEILLANCE | | 3 | 02/06/81 |
| 13164 | BROWNS FERRY 1 - GENERIC CLARIFICATION OF "OPERABILITY" (AMEND 66) | | 2 | 02/06/81 |
| 11627 | BROWNS FERRY 1 - HYDRO TEST PROGRAM | | 1 | 04/01/81 |

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|-----------------------|---|-------------|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12119 | BROWNS FERRY 1 - EMERGENCY PLAN REVIEW | | 1 | |
| 12295 | BROWNS FERRY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE BROWN'S FERRY 1 | | 3 | |
| 13104 | BROWNS FERRY 1 - ADEQUACY OF DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 13271 | BROWNS FERRY 1 - REVIEW OF CORPORATE CAPABILITIES | | 3 | |
| 43171 | BROWNS FERRY 1 - SCRAM DISCHARGE VOL. VENT & DRAIN VALVES | | 1 | |
| 07884 | BROWNS FERRY 1 - DIESEL GENERATOR LOCKOUT | | 3 | |
| 07216 | BROWNS FERRY 1 - SPENT FUEL POOL EXPANSION (AMEND.42) | | 1 | |
| 08715 | BROWNS FERRY 1 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 10354 | BROWNS FERRY 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10459 | BROWNS FERRY 1 - VITAL AREA ANALYSIS | | 1 | |
| 08720 | BROWNS FERRY 1 - MECHANICAL SNUBBERS | | 2 | |
| 08726 | BROWNS FERRY 1 - HYDRAULIC SNUBBERS | | 3 | |
| 10187 | BROWNS FERRY 1 - CONTAINMENT PURGE | | 1 | |
| 08133 | BROWNS FERRY 1 - APPENDIX I T.S. IMPLEMENTATION REVIEW REVIEW | | 3 | |
| 07929 | BROWNS FERRY 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 10291 | BROWNS FERRY 1 - CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATION | | 3 | |
| 10010 | BROWNS FERRY 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 08318 | BROWNS FERRY 1 - 10 CFR 50.55A(G) INSERVICE INSPECTION - GENERIC | | 1 | |
| 11324 | BROWNS FERRY 1 - INSERVICE TESTING | | 1 | |
| 10463 | BROWNS FERRY 1 - CONTINGENCY PLAN REVIEW | | 1 | |
| 07974 | BROWNS FERRY 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42724 | BROWNS FERRY 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 10455 | BROWNS FERRY 1 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 42654 | BROWNS FERRY 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 08929 | BROWNS FERRY 1 - RPS POWER SUPPLY | | 1 | |
| 43727 | BROWNS FERRY 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | 1 | |
| 42873 | BROWNS FERRY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42568 | BROWNS FERRY 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42481 | BROWNS FERRY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
|-----------------------|--|-------------|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 44004 | BROWNS FERRY 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44072 | BROWNS FERRY 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44143 | BROWNS FERRY 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44214 | BROWNS FERRY 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44284 | BROWNS FERRY 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | <u>(CONTINUATION)</u> | | <u>TARGET</u> |
| 44355 | BROWNS FERRY 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44423 | BROWNS FERRY 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44493 | BROWNS FERRY 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44564 | BROWNS FERRY 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44761 | BROWNS FERRY 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44832 | BROWNS FERRY 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44903 | BROWNS FERRY 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44974 | BROWNS FERRY 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45045 | BROWNS FERRY 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45116 | BROWNS FERRY 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45325 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45532 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45560 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45584 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45611 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45680 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45704 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45728 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LFCI | | |
| 45752 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45776 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45807 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45876 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45900 | BROWNS FERRY 1 - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45927 | BROWNS FERRY 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 45998 | BROWNS FERRY 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46070 | BROWNS FERRY 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46142 | BROWNS FERRY 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46214 | BROWNS FERRY 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46286 | BROWNS FERRY 1 - NUREG-0737 | III.A.1.2, METEOROLOGICAL DATA UPGRADE | | |
| 46357 | BROWNS FERRY 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46428 | BROWNS FERRY 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42421 | BROWNS FERRY 1 - EXTEND MAPLHGR LIMITS (AMEND 62) | | 1 | 10/10/80 |
| 42395 | BROWNS FERRY 1 - ELECTRICAL MODIFICATIONS TO CORRECT UNDERVOLTAGE | | 1 | 10/23/80 |
| 42397 | BROWNS FERRY 1 - ELECTRICAL CHANGES - BUSES, RELAYS AND BOARDS (AMEND 63) | | 1 | 12/02/80 |
| 42940 | BROWNS FERRY 1 - RHR SERVICE WATER CONNECTION (AMEND 64) | | 1 | 12/12/80 |
| 11536 | BROWNS FERRY 1 - ASSESSMENT OF FISH IMPINGEMENT (AMEND 65) | | 2 | 01/09/81 |
| 42291 | BROWNS FERRY 1 - OLYN CODE | | 2 | 01/12/81 |
| 43174 | BROWNS FERRY 1 - DEFINITION OF COLD SHUTDOWN (AMEND 66) | | 1 | 02/06/81 |
| 43168 | BROWNS FERRY 1 - SCRAM DELAY TIMES (AMEND 67) | | 2 | 02/24/81 |
| 11533 | BROWNS FERRY 1 - SCRAM TIMING PRESSURE LIMITS (AMEND 68) | | 2 | 02/27/81 |
| 12105 | BROWNS FERRY 1 - CHANGES TO ETS (AMEND 69) | | 2 | 03/06/81 |
| 07436 | BROWNS FERRY 1 - HPCI AND RCIC SYSTEMS (AMEND 70) | | 2 | 03/11/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 11569 | BROWNS FERRY 1 - BOTTLED UP OPERATION | | 2 | |
| 11916 | BROWNS FERRY 1 - ADMIN. CHANGES TO T.S. | | 2 | |

(CONTINUATION)

BROWNS FERRY 1

| IAC # | IAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|--------|---|----------------|----------|---------------|
| | (CONTINUATION) | | | |
| 11936 | BROWNS FERRY 1 - CONTINUATION | | 1 | TARGEI |
| 12611 | BROWNS FERRY 1 - 1.2, 3, 4, 5, 6, 7, 8, 9, 10, 11, 12, 13, 14, 15, 16, 17, 18, 19, 20, 21, 22, 23, 24, 25, 26, 27, 28, 29, 30, 31, 32, 33, 34, 35, 36, 37, 38, 39, 40, 41, 42, 43, 44, 45, 46, 47, 48, 49, 50, 51, 52, 53, 54, 55, 56, 57, 58, 59, 60, 61, 62, 63, 64, 65, 66, 67, 68, 69, 70, 71, 72, 73, 74, 75, 76, 77, 78, 79, 80, 81, 82, 83, 84, 85, 86, 87, 88, 89, 90, 91, 92, 93, 94, 95, 96, 97, 98, 99, 100, 101, 102, 103, 104, 105, 106, 107, 108, 109, 110, 111, 112, 113, 114, 115, 116, 117, 118, 119, 120, 121, 122, 123, 124, 125, 126, 127, 128, 129, 130, 131, 132, 133, 134, 135, 136, 137, 138, 139, 140, 141, 142, 143, 144, 145, 146, 147, 148, 149, 150, 151, 152, 153, 154, 155, 156, 157, 158, 159, 160, 161, 162, 163, 164, 165, 166, 167, 168, 169, 170, 171, 172, 173, 174, 175, 176, 177, 178, 179, 180, 181, 182, 183, 184, 185, 186, 187, 188, 189, 190, 191, 192, 193, 194, 195, 196, 197, 198, 199, 200, 201, 202, 203, 204, 205, 206, 207, 208, 209, 210, 211, 212, 213, 214, 215, 216, 217, 218, 219, 220, 221, 222, 223, 224, 225, 226, 227, 228, 229, 230, 231, 232, 233, 234, 235, 236, 237, 238, 239, 240, 241, 242, 243, 244, 245, 246, 247, 248, 249, 250, 251, 252, 253, 254, 255, 256, 257, 258, 259, 260, 261, 262, 263, 264, 265, 266, 267, 268, 269, 270, 271, 272, 273, 274, 275, 276, 277, 278, 279, 280, 281, 282, 283, 284, 285, 286, 287, 288, 289, 290, 291, 292, 293, 294, 295, 296, 297, 298, 299, 300, 301, 302, 303, 304, 305, 306, 307, 308, 309, 310, 311, 312, 313, 314, 315, 316, 317, 318, 319, 320, 321, 322, 323, 324, 325, 326, 327, 328, 329, 330, 331, 332, 333, 334, 335, 336, 337, 338, 339, 340, 341, 342, 343, 344, 345, 346, 347, 348, 349, 350, 351, 352, 353, 354, 355, 356, 357, 358, 359, 360, 361, 362, 363, 364, 365, 366, 367, 368, 369, 370, 371, 372, 373, 374, 375, 376, 377, 378, 379, 380, 381, 382, 383, 384, 385, 386, 387, 388, 389, 390, 391, 392, 393, 394, 395, 396, 397, 398, 399, 400, 401, 402, 403, 404, 405, 406, 407, 408, 409, 410, 411, 412, 413, 414, 415, 416, 417, 418, 419, 420, 421, 422, 423, 424, 425, 426, 427, 428, 429, 430, 431, 432, 433, 434, 435, 436, 437, 438, 439, 440, 441, 442, 443, 444, 445, 446, 447, 448, 449, 450, 451, 452, 453, 454, 455, 456, 457, 458, 459, 460, 461, 462, 463, 464, 465, 466, 467, 468, 469, 470, 471, 472, 473, 474, 475, 476, 477, 478, 479, 480, 481, 482, 483, 484, 485, 486, 487, 488, 489, 490, 491, 492, 493, 494, 495, 496, 497, 498, 499, 500, 501, 502, 503, 504, 505, 506, 507, 508, 509, 510, 511, 512, 513, 514, 515, 516, 517, 518, 519, 520, 521, 522, 523, 524, 525, 526, 527, 528, 529, 530, 531, 532, 533, 534, 535, 536, 537, 538, 539, 540, 541, 542, 543, 544, 545, 546, 547, 548, 549, 550, 551, 552, 553, 554, 555, 556, 557, 558, 559, 560, 561, 562, 563, 564, 565, 566, 567, 568, 569, 570, 571, 572, 573, 574, 575, 576, 577, 578, 579, 580, 581, 582, 583, 584, 585, 586, 587, 588, 589, 590, 591, 592, 593, 594, 595, 596, 597, 598, 599, 600, 601, 602, 603, 604, 605, 606, 607, 608, 609, 610, 611, 612, 613, 614, 615, 616, 617, 618, 619, 620, 621, 622, 623, 624, 625, 626, 627, 628, 629, 630, 631, 632, 633, 634, 635, 636, 637, 638, 639, 640, 641, 642, 643, 644, 645, 646, 647, 648, 649, 650, 651, 652, 653, 654, 655, 656, 657, 658, 659, 660, 661, 662, 663, 664, 665, 666, 667, 668, 669, 670, 671, 672, 673, 674, 675, 676, 677, 678, 679, 680, 681, 682, 683, 684, 685, 686, 687, 688, 689, 690, 691, 692, 693, 694, 695, 696, 697, 698, 699, 700, 701, 702, 703, 704, 705, 706, 707, 708, 709, 710, 711, 712, 713, 714, 715, 716, 717, 718, 719, 720, 721, 722, 723, 724, 725, 726, 727, 728, 729, 730, 731, 732, 733, 734, 735, 736, 737, 738, 739, 740, 741, 742, 743, 744, 745, 746, 747, 748, 749, 750, 751, 752, 753, 754, 755, 756, 757, 758, 759, 760, 761, 762, 763, 764, 765, 766, 767, 768, 769, 770, 771, 772, 773, 774, 775, 776, 777, 778, 779, 780, 781, 782, 783, 784, 785, 786, 787, 788, 789, 790, 791, 792, 793, 794, 795, 796, 797, 798, 799, 800, 801, 802, 803, 804, 805, 806, 807, 808, 809, 810, 811, 812, 813, 814, 815, 816, 817, 818, 819, 820, 821, 822, 823, 824, 825, 826, 827, 828, 829, 830, 831, 832, 833, 834, 835, 836, 837, 838, 839, 840, 841, 842, 843, 844, 845, 846, 847, 848, 849, 850, 851, 852, 853, 854, 855, 856, 857, 858, 859, 860, 861, 862, 863, 864, 865, 866, 867, 868, 869, 870, 871, 872, 873, 874, 875, 876, 877, 878, 879, 880, 881, 882, 883, 884, 885, 886, 887, 888, 889, 890, 891, 892, 893, 894, 895, 896, 897, 898, 899, 900, 901, 902, 903, 904, 905, 906, 907, 908, 909, 910, 911, 912, 913, 914, 915, 916, 917, 918, 919, 920, 921, 922, 923, 924, 925, 926, 927, 928, 929, 930, 931, 932, 933, 934, 935, 936, 937, 938, 939, 940, 941, 942, 943, 944, 945, 946, 947, 948, 949, 950, 951, 952, 953, 954, 955, 956, 957, 958, 959, 960, 961, 962, 963, 964, 965, 966, 967, 968, 969, 970, 971, 972, 973, 974, 975, 976, 977, 978, 979, 980, 981, 982, 983, 984, 985, 986, 987, 988, 989, 990, 991, 992, 993, 994, 995, 996, 997, 998, 999, 1000 | | 1 | |
| 12611 | BROWNS FERRY 1 - ISOLATION OF MAIN STEAM LINES ON HIGH TEMPERATURE | | 2 | |
| 41011 | BROWNS FERRY 1 - FIRE PROTECTION RESEARCH PROGRAM | | 1 | |
| 41097 | BROWNS FERRY 1 - CONTAINMENT INTEGRATED LEAK RATE TEST | | 2 | |
| 42333 | BROWNS FERRY 1 - LONG TERM STORAGE OF RADWASTE ONSITE | | 1 | |
| 42356 | BROWNS FERRY 1 - STATION BLACKOUT (A-44) | | 1 | |
| 43308 | BROWNS FERRY 1 - SDV CONTINUOUS LEVEL MAINTAINING SYSTEM | | 1 | |
| 43582 | BROWNS FERRY 1 - CABLE SPREADING ROOM SPRINKLER SYSTEM | | 1 | |
| 43592 | BROWNS FERRY 1 - CAD SYSTEM VALVES | | 1 | |
| 43598 | BROWNS FERRY 1 - TEST FREQUENCIES FOR ISOLATION VALVES | | 1 | |
| 43708 | BROWNS FERRY 1 - SECONDARY CONTAINMENT INTEGRITY | | 1 | |
| 43831 | BROWNS FERRY 1 - FIRE DAMPERS, DOORS AND BARRIER PENETRATIONS | | 1 | |
| 43834 | BROWNS FERRY 1 - CONTROL OF HIGH RADIATION AREAS | | 1 | |
| 43837 | BROWNS FERRY 1 - PUMP AND VALVE TESTING | | 1 | |
| 11625 | BROWNS FERRY 1 - ALLOWANCE TO DE-ENERGIZE 161-KV LINE, A START BUS AND A SHUTDOWN BUS | | 1 | |
| 08729 | BROWNS FERRY 1 - CONTAINMENT LEAK RATE TEST | | 2 | |
| 12621 | BROWNS FERRY 1 - REACTOR BUILDING IN LEAKAGE RATE | | 2 | |
| 07588 | BROWNS FERRY 1 - IMPROVED HYDROGEN MONITORING | | 2 | |
| 42809 | BROWNS FERRY 1 - CONTAINMENT LEAK TESTING | | 1 | |
| 42802 | BROWNS FERRY 1 - MAIN STEAM LINE SPACE HIGH TEMP ISOLATION | | 2 | |
| 42805 | BROWNS FERRY 1 - CHANGES IN ORGANIZATION | | 2 | |
| 43808 | BROWNS FERRY 1 - AUXILIARY ELECTRICAL SYSTEM MODIFICATIONS | | 2 | |
| 437051 | BROWNS FERRY 1 - RELOAD | | 1 | |
| 43800 | BROWNS FERRY 1 - RELOAD | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: BROWNS FERRY 2

PLANT LOCATION: 10 MI NW OF DECATUR, ALA
 DOCKET NUMBER: 050-00260
 ARCH/ENGINEER: TVA
 IE INSPECTOR: R. SULLIVAN

LICENSED POWER: 3293 MW
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. CLARK
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11363 | BROWNS FERRY 2 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | <u>COMPLETE</u> 02/06/81 |
| 13165 | BROWNS FERRY 2 - GENERIC CLARIFICATION OF "OPERABILITY" (AMEND 62) | | 2 | 02/06/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12120 | BROWNS FERRY 2 - EMERGENCY PLAN REVIEW | | 1 | <u>TARGET</u> |
| 12296 | BROWNS FERRY 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE BROWN'S FERRY 2 | | 3 | |
| 13105 | BROWNS FERRY 2 - ADEQUACY OF DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 13272 | BROWNS FERRY 2 - REVIEW OF CORPORATE CAPABILITIES | | 3 | |
| 08426 | BROWNS FERRY 2 - SFP STORAGE (AMEND 39) | | 1 | |
| 08328 | BROWNS FERRY 2 - 10 CFR 50.35A(G) - INSERVICE INSPECTION - GENERIC | | 1 | |
| 11325 | BROWNS FERRY 2 - INSERVICE TESTING | | 1 | |
| 08716 | BROWNS FERRY 2 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08930 | BROWNS FERRY 2 - RPS POWER SUPPLY | | 1 | |
| 10353 | BROWNS FERRY 2 - LOSS OF 125V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10460 | BROWNS FERRY 2 - VITAL AREA ANALYSIS | | 1 | |
| 08721 | BROWNS FERRY 2 - MECHANICAL SNUBBERS | | 2 | |
| 08727 | BROWNS FERRY 2 - HYDRAULIC SNUBBERS | | 3 | |
| 10186 | BROWNS FERRY 2 - CONTAINMENT PURGE | | 1 | |
| 08134 | BROWNS FERRY 2 - APPENDIX I T.S. IMPLEMENTATION REVIEW REVIEW | | 3 | |
| 07930 | BROWNS FERRY 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 10292 | BROWNS FERRY 2 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | |
| 10011 | BROWNS FERRY 2 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 10464 | BROWNS FERRY 2 - CONTINGENCY PLAN REVIEW | | 1 | |
| 07975 | BROWNS FERRY 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42725 | BROWNS FERRY 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 10456 | BROWNS FERRY 2 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 42231 | BROWNS FERRY 2 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 42655 | BROWNS FERRY 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43731 | BROWNS FERRY 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | 1 | |
| 42874 | BROWNS FERRY 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42482 | BROWNS FERRY 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44005 | BROWNS FERRY 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44073 | BROWNS FERRY 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMIT | | | |
| 44144 | BROWNS FERRY 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RD AND SRO TRAINING | | | |
| 44215 | BROWNS FERRY 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44285 | BROWNS FERRY 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44356 | BROWNS FERRY 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44424 | BROWNS FERRY 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44494 | BROWNS FERRY 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44565 | BROWNS FERRY 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TM1 ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|-----------------------------|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 44762 | BROWNS FERRY 2 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44833 | BROWNS FERRY 2 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44904 | BROWNS FERRY 2 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44975 | BROWNS FERRY 2 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45046 | BROWNS FERRY 2 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45117 | BROWNS FERRY 2 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45326 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45533 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45561 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45585 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45612 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45681 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45705 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45729 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45753 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45777 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45808 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45877 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45901 | BROWNS FERRY 2 - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45928 | BROWNS FERRY 2 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 45999 | BROWNS FERRY 2 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46071 | BROWNS FERRY 2 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46143 | BROWNS FERRY 2 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46215 | BROWNS FERRY 2 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46287 | BROWNS FERRY 2 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46358 | BROWNS FERRY 2 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46429 | BROWNS FERRY 2 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42422 | BROWNS FERRY 2 - | EXTEND MAPLHGR LIMITS (AMEND 57) | 1 | 10/10/80 |
| 42396 | BROWNS FERRY 2 - | ELECTRICAL MODIFICATIONS TO CORRECT UNDERVOLTAGE | 1 | 10/23/80 |
| 42332 | BROWNS FERRY 2 - | RELOAD (AMEND 58) | 1 | 11/12/80 |
| 42810 | BROWNS FERRY 2 - | IMPROVED HYDROGEN MONITORING (AMEND 58) | 1 | 11/12/80 |
| 42398 | BROWNS FERRY 2 - | ELECTRICAL CHANGES - BUSES, RELAYS AND BOARDS (AMEND 59) | 1 | 12/02/80 |
| 42941 | BROWNS FERRY 2 - | RHR SERVICE WATER CONNECTION | 1 | 12/12/80 |
| 11537 | BROWNS FERRY 2 - | ASSESSMENT OF FISH IMPINGEMENT (AMEND 61) | 2 | 01/09/81 |
| 43175 | BROWNS FERRY 2 - | DEFINITION OF COLD SHUTDOWN (AMEND 62) | 1 | 02/06/81 |
| 43169 | BROWNS FERRY 2 - | SCRAM DELAY TIMES (AMEND 63) | 2 | 02/24/81 |
| 11534 | BROWNS FERRY 2 - | SCRAM TIMING PRESSURE LIMITS (AMEND 64) | 2 | 02/27/81 |
| 12106 | BROWNS FERRY 2 - | CHANGES TO ETS (AMEND 65) | 2 | 03/06/81 |
| 08424 | BROWNS FERRY 2 - | HPCI RCIC (AMEND 66) | 2 | 03/11/81 |
| 43591 | BROWNS FERRY 2 - | MISLOCATED FUEL ASSEMBLIES VENT (B-24) | 1 | 05/26/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 11570 | BROWNS FERRY 2 - | BOTTLED UP OPERATION | 2 | |
| 11917 | BROWNS FERRY 2 - | ADMIN CHANGES TO T.S. | 2 | |
| 12612 | BROWNS FERRY 2 - | ISOLATION OF MAIN STEAM LINES ON HIGH TEMPERATURE | 2 | |
| 41098 | BROWNS FERRY 2 - | CONTAINMENT INTEGRATED LEAK RATE TEST | 2 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 2

| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------|------------------------|--|-----------------|----------------------|
| | <u>ACTIVE ACTIONS</u> | <u>(CONTINUATION)</u> | | <u>TARGET</u> |
| 42357 | BROWNS FERRY 2 | - LONG TERM STORAGE OF RADWASTE ONSITE | 1 | |
| 43172 | BROWNS FERRY 2 | - SCRAM DISCHARGE VOL. VENT & DRAIN VALVES | 1 | |
| 43507 | BROWNS FERRY 2 | - POWER SPIKING | 2 | |
| 43509 | BROWNS FERRY 2 | - STATION BLACKOUT (A-44) | 1 | |
| 43590 | BROWNS FERRY 2 | - POWER SPIKING LIMIT | 2 | |
| 43593 | BROWNS FERRY 2 | - SDV CONTINUOUS LEVEL MONITORING SYSTEM | 1 | |
| 43597 | BROWNS FERRY 2 | - CABLE SPREADING ROOM SPRINKLER SYSTEM | 2 | |
| 43832 | BROWNS FERRY 2 | - SECONDARY CONTAINMENT INTEGRITY | | |
| 43835 | BROWNS FERRY 2 | - FIRE DAMPERS, DOORS AND BARRIER PENETRATIONS | | |
| 43838 | BROWNS FERRY 2 | - CONTROL OF HIGH RADIATION AREAS | | |
| 11626 | BROWNS FERRY 2 | - PUMP AND VALVE TESTING | 1 | |
| 08418 | BROWNS FERRY 2 | - DE-ENERGIZING 161 KV LINE | 2 | |
| 12622 | BROWNS FERRY 2 | - CONTAINMENT LEAK RATE TEST | 2 | |
| 08420 | BROWNS FERRY 2 | - REACTOR BLDG. INLEAKAGE RATE | 2 | |
| 42803 | BROWNS FERRY 2 | - CONTAINMENT LEAK TESTING | 2 | |
| 42806 | BROWNS FERRY 2 | - MAIN STEAM LINE SPACE HIGH TEMP ISOLATIONS | 2 | |
| 43752 | BROWNS FERRY 2 | - AUXILIARY ELECTRICAL SYSTEM MODIFICATIONS | 1 | |
| 42569 | BROWNS FERRY 2 | - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | 1 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: BROWNS FERRY 3

PLANT LOCATION: 10 MI NW OF DECATUR, ALA
 DOCKET NUMBER: 050-00296
 ARCH/ENGINEER: TVA
 IE INSPECTOR: R. SULLIVAN

LICENSED POWER: 3293 MWT
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. CLARK
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11362 | BROWNS FERRY 3 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | <u>COMPLETE</u> 02/05/81 |
| 13166 | BROWNS FERRY 3 - GENERIC CLARIFICATION OF "OPERABILITY" D(AMEND 38) | | 2 | 02/06/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12121 | BROWNS FERRY 3 - EMERGENCY PLAN REVIEW | | 1 | <u>TARGET</u> |
| 12297 | BROWNS FERRY 3 - HEIB AND CONSEQUENTIAL SYSTEM FAILURE BROWN'S FERRY 3 | | 3 | |
| 13106 | BROWNS FERRY 3 - ADEQUACY OF DISTRIBUTION SYSTEMS VOLTAGES | | 1 | |
| 13273 | BROWNS FERRY 3 - REVIEW OF CORPORATE CAPABILITIES | | 3 | |
| 06867 | BROWNS FERRY 3 - ISI PUMP AND VALVE TESTING (10CFR50.55A) | | 1 | |
| 11326 | BROWNS FERRY 3 - INSERVICE TESTING | | 1 | |
| 07886 | BROWNS FERRY 3 - DIESEL GENERATOR LOCKOUT | | 3 | |
| 08427 | BROWNS FERRY 3 - SFP STORAGE (AMEND 16) | | 1 | |
| 08717 | BROWNS FERRY 3 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08931 | BROWNS FERRY 3 - RPS POWER SUPPLY | | 1 | |
| 10355 | BROWNS FERRY 3 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10461 | BROWNS FERRY 3 - VITAL AREA ANALYSIS | | 1 | |
| 08722 | BROWNS FERRY 3 - MECHANICAL SNUBBERS | | 2 | |
| 08728 | BROWNS FERRY 3 - HYDRAULIC SNUBBERS | | 3 | |
| 10185 | BROWNS FERRY 3 - CONTAINMENT PURGE | | 1 | |
| 08135 | BROWNS FERRY 3 - APPENDIX I T.S. IMPLEMENTATION REVIEW REVIEW | | 3 | |
| 07931 | BROWNS FERRY 3 - MARK I CONTAINMENT LONG TERM PROG. IMPLEMENT. | | 1 | |
| 10293 | BROWNS FERRY 3 - CONTAINMENT LEAKAGE DUE TO SEAL ON | | 3 | |
| 10012 | BROWNS FERRY 3 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 10465 | BROWNS FERRY 3 - CONTINGENCY PLAN REVIEW | | 1 | |
| 08438 | BROWNS FERRY 3 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42726 | BROWNS FERRY 3 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 10457 | BROWNS FERRY 3 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 42232 | BROWNS FERRY 3 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 42656 | BROWNS FERRY 3 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43736 | BROWNS FERRY 3 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | 1 | |
| 42875 | BROWNS FERRY 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42483 | BROWNS FERRY 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44006 | BROWNS FERRY 3 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44074 | BROWNS FERRY 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVER TIME LIMITS | | | |
| 44145 | BROWNS FERRY 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44216 | BROWNS FERRY 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44286 | BROWNS FERRY 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44357 | BROWNS FERRY 3 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44425 | BROWNS FERRY 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44495 | BROWNS FERRY 3 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 3

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|--------------------------------------|------------------------|--|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 4456F | BROWNS FERRY 3 | - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44763 | BROWNS FERRY 3 | - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44834 | BROWNS FERRY 3 | - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44905 | BROWNS FERRY 3 | - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | |
| 44976 | BROWNS FERRY 3 | - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45047 | BROWNS FERRY 3 | - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45118 | BROWNS FERRY 3 | - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45327 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45534 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45562 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45586 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45613 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45682 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY | | |
| 45706 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45730 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45754 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.25, POWER ON PUMP SEALS | | |
| 45778 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45809 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45878 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45902 | BROWNS FERRY 3 | - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45929 | BROWNS FERRY 3 | - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46000 | BROWNS FERRY 3 | - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46072 | BROWNS FERRY 3 | - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46144 | BROWNS FERRY 3 | - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | |
| 46216 | BROWNS FERRY 3 | - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46288 | BROWNS FERRY 3 | - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46359 | BROWNS FERRY 3 | - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46430 | BROWNS FERRY 3 | - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|------------------------|--|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42399 | BROWNS FERRY 3 | - ELECTRICAL CHANGES - BUSES, RELAYS AND BOARDS (AMEND 35) | 1 | <u>COMPLETE</u> 12/02/80 |
| 11538 | BROWNS FERRY 3 | - ASSESSMENT OF FISH IMPINGEMENT (AMEND 36) | 2 | 01/09/81 |
| 42335 | BROWNS FERRY 3 | - QUALIFICATION OF HYDROGEN ANALYZERS (AMEND 37) | 1 | 01/15/81 |
| 42808 | BROWNS FERRY 3 | - RELOAD (AMEND 37) | 1 | 01/15/81 |
| 42811 | BROWNS FERRY 3 | - IMPROVED HYDROGEN MONITORING (AMEND 37) (AMEND 37) | 1 | 01/15/81 |
| 43166 | BROWNS FERRY 3 | - POWER SPIKING PENALTY (AMEND 37) | 2 | 01/15/81 |
| 43167 | BROWNS FERRY 3 | - ADDITION OF MG SEIS FOR LPCI MOD (AMEND 37) | 1 | 01/15/81 |
| 43176 | BROWNS FERRY 3 | - DEFINITION OF COLD SHUTDOWN (AMEND 38) | 1 | 02/06/81 |
| 43170 | BROWNS FERRY 3 | - SCRAM DELAY TIMES (AMEND 39) | 2 | 02/24/81 |
| 11535 | BROWNS FERRY 3 | - SCRAM TIMING PRESSURE LIMITS (AMEND 40) | 2 | 02/27/81 |
| 12107 | BROWNS FERRY 3 | - CHANGES TO ETS (AMEND 41) | 2 | 03/06/81 |
| 08425 | BROWNS FERRY 3 | - HPCI RCIC (AMEND 42) | 2 | 03/11/81 |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|-----------------------|------------------------|---|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 11571 | BROWNS FERRY 3 | - BOTTLED UP OPERATION | 2 | |
| 11918 | BROWNS FERRY 3 | - ADMIN CHANGES TO T.S. | 2 | |
| 12613 | BROWNS FERRY 3 | - ISOLATION OF MAIN STEAM LINES ON HIGH TEMPERATURE | 2 | |
| 42000 | BROWNS FERRY 3 | - CONTAINMENT INTEGRATED LEAK RATE TEST | 2 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 3

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------|------------------------|--|-----------------|----------------------|
| | <u>ACTIVE ACTIONS</u> | <u>(CONTINUATION)</u> | | <u>TARGET</u> |
| 42358 | BROWNS FERRY 3 | - LONG TERM STORAGE OF RADWASTE ONSITE | 1 | |
| 43173 | BROWNS FERRY 3 | - SCRAM DISCHARGE VOL. VENT & DRAIN VALVES | 1 | |
| 43510 | BROWNS FERRY 3 | - STATION BLACKOUT (A-44) | 1 | |
| 43594 | BROWNS FERRY 3 | - SDV CONTINUOUS LEVEL MONITORING SYSTEM | 1 | |
| 43598 | BROWNS FERRY 3 | - CABLE SPREADING ROOM SPRINKLER STEM | 2 | |
| 43833 | BROWNS FERRY 3 | - SECONDARY CONTAINMENT INTEGRITY | | |
| 43836 | BROWNS FERRY 3 | - FIRE DAMPERS, DOORS AND BARRIER PENETRATIONS | | |
| 43839 | BROWNS FERRY 3 | - CONTROL HIGH RADIATION AREAS | | |
| 08419 | BROWNS FERRY 3 | - DE-ENERGIZING 161 KV LINE | 2 | |
| 12623 | BROWNS FERRY 3 | - CONTAINMENT LEAK RATE TEST | 2 | |
| 08421 | BROWNS FERRY 3 | - REACTOR BLDG. INLEAKAGE RATE | 2 | |
| 42804 | BROWNS FERRY 3 | - CONTAINMENT LEAK TESTING | 2 | |
| 42807 | BROWNS FERRY 3 | - MAIN STEAM LINE SPACE HIGH TEMP ISOLATIONS | 2 | |
| 42570 | BROWNS FERRY 3 | - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | 1 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: BRUNSWICK 1

PLANT LOCATION: 3 MI N OF SOUTHPORT, NC
 DOCKET NUMBER: 050-00325
 ARCH/ENGINEER: UEC
 IE INSPECTOR: C. MCFARLAND

LICENSED POWER: 2436 MWT
 DESIGN POWER: 0821 M^{1/2}
 NSSS VENDOR: GE

PROJECT MANAGER: J. VANVLIET
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. SHEPPARD

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12194 | BRUNSWICK 1 - MANAG. & TECH. RESOURCES NUREG 0731 | | 3 | <u>COMPLETE</u> 10/01/80 |
| 10697 | BRUNSWICK 1 - CONTINGENCY PLAN REVIEW (AMEND 31) | | 1 | 12/10/80 |
| 42984 | BRUNSWICK 1 - REACTOR VESSEL AND RECIRC PUMP SUPPORTS MATERIALS FRACTURE TOUGHNESS NUREG 0577 | | 1 | 01/26/81 |
| 10294 | BRUNSWICK 1 - CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATION DETERIORATION | | 3 | 01/27/81 |
| 11361 | BRUNSWICK 1 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | 01/28/81 |
| 10184 | BRUNSWICK 1 - CONTAINMENT PURGE | | 1 | 01/30/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42727 | BRUNSWICK 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 08106 | BRUNSWICK 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 12186 | BRUNSWICK 1 - HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE | | 3 | |
| 42657 | BRUNSWICK 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 41074 | BRUNSWICK 1 - CONTROL RODS FAIL TO FULLY INSERT | | 3 | |
| 43891 | BRUNSWICK UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 10356 | BRUNSWICK 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 43737 | BRUNSWICK 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42876 | BRUNSWICK 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 06662 | BRUNSWICK 1 - COMBUSTIBLE GAS CONTROL | | 3 | |
| 07261 | BRUNSWICK 1 - 10 CFR 50.55A REVIEW - ISI PIPING/WELDS | | 1 | |
| 07976 | BRUNSWICK 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08829 | BRUNSWICK 1 - T.S. SURVEILLANCE FOR MECHANICAL SNUBBERS | | 2 | |
| 08928 | BRUNSWICK 1 - RPS POWER SUPPLY | | 1 | |
| 10694 | BRUNSWICK 1 - GUARD TRAINING REVIEW | | 1 | |
| 10699 | BRUNSWICK 1 - VITAL AREA ANALYSIS | | 1 | |
| 11264 | BRUNSWICK 1 - 10 CFR 50.55A(G) IST PUMPS/VALVES | | 1 | |
| 11887 | BRUNSWICK 1 - SAFE SHUTDOWN REVIEW | | | |
| 12122 | BRUNSWICK 1 - EMERGENCY PLAN REVIEW | | 1 | |
| 42571 | BRUNSWICK 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 07932 | BRUNSWICK 1 - MARK CONTAIN. LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 12829 | BRUNSWICK 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | | |
| 42207 | BRUNSWICK 1 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | | |
| 42485 | BRUNSWICK 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
|-----------------------|---|-------------|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44007 | BRUNSWICK 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44075 | BRUNSWICK 1 - NUREG-0737 I.A.1.3., SHIFT MANNING OVERTIME LIMITS | | | |
| 44146 | BRUNSWICK 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44217 | BRUNSWICK 1 - NUREG-0737 I.C.1.2.A. INADEQUATE CORE COOLING GUIDELINES | | | |
| 44287 | BRUNSWICK 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44358 | BRUNSWICK 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44426 | BRUNSWICK 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44496 | BRUNSWICK 1 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|--------------------------------------|------------------------|--|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44567 | BRUNSWICK 1 | - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44764 | BRUNSWICK 1 | - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44835 | BRUNSWICK 1 | - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44906 | BRUNSWICK 1 | - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | |
| 44977 | BRUNSWICK 1 | - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45048 | BRUNSWICK 1 | - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45119 | BRUNSWICK 1 | - NUREG-0737 II.F.2.3, INADEQUATE COPE COOLING INSTRUMENTATION | | |
| 45328 | BRUNSWICK 1 | - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45535 | BRUNSWICK 1 | - NUREG-0737 II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45563 | BRUNSWICK 1 | - NUREG-0737 II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45587 | BRUNSWICK 1 | - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45614 | BRUNSWICK 1 | - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45683 | BRUNSWICK 1 | - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY | | |
| 45707 | BRUNSWICK 1 | - NUREG-0737 II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45731 | BRUNSWICK 1 | - NUREG-0737 II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45755 | BRUNSWICK 1 | - NUREG-0737 II.K.3.25, POWER ON PUMP SEALS | | |
| 45779 | BRUNSWICK 1 | - NUREG-0737 II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45810 | BRUNSWICK 1 | - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45879 | BRUNSWICK 1 | - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45903 | BRUNSWICK 1 | - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45930 | BRUNSWICK 1 | - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46001 | BRUNSWICK 1 | - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46073 | BRUNSWICK 1 | - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46145 | BRUNSWICK 1 | - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | |
| 46217 | BRUNSWICK 1 | - NUREG-0737 III.A.2.1, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46289 | BRUNSWICK 1 | - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46360 | BRUNSWICK 1 | - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46431 | BRUNSWICK 1 | - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|--------------------------|------------------------|--|-----------------|---------------------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42018 | BRUNSWICK 1 | - NRC STAFF GENERIC REPORT ON BWRs REPORT ON BWRs | 1 | <u>COMPLETE</u> 10/01/80 |
| 43209 | BRUNSWICK 1 | - REQUEST FOR SURVEILLANCE TEST INTERVAL EXTENSIONS (AMEND 33) | | 12/23/80 |
| 41077 | BSEP-1 | CLARIFY OPERABLE AS IT APPLIES TO SINGLE FAILURE (AMEND 32) | 1 | 12/29/80 |
| 42843 | BRUNSWICK 1 | - INSTRUMENT/SETPOINT DRIFT | 1 | 01/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 43535 | BRUNSWICK 1 | - TEMPORARY TS CHANGE TO ALLOW SINGLE LOOP OPERATION | 1 | |
| 43866 | BRUNSWICK 1 | - QUALIFICATION OF QA PERSONNEL (GENERIC LETTER 81-01) | | |
| 43418 | BRUNSWICK 1 | - NEW APPENDIX R TO 10 CFR 50 REGARDING FIRE PROTECTION FEATURES | 1 | |
| 42970 | BRUNSWICK 1 | - CATEGORY "A" IMI LESSONS LEARNED TS CHANGES | 1 | |
| 43584 | BRUNSWICK 1 | - SERVICE WATER OPERABILITY REQUIREMENTS | 2 | |
| 43676 | BRUNSWICK 1 | - AUGMENTED OFF-GAS SYSTEM IMPROVEMENTS | 1 | |
| 43794 | BRUNSWICK 1 | - QUALITY ASSURANCE PROGRAM REVIEW | | |
| 46500 | BRUNSWICK 1 | - ISI RELIEF REQUEST | | |
| 42947 | BRUNSWICK 2 | - EFFECT OF DC POWER SUPPLY FAILURE ON ECCS PERFORMANCE | 1 | |
| 43694 | BRUNSWICK 1 | - INSERVICE SURVEILLANCE REQUIREMENTS FOR SNUBBERS | 1 | |
| 43972 | BRUNSWICK 1 | - WASTEWATER DISCHARGE WITHOUT COOLING TOWERS | | |
| 41080 | BRUNSWICK 1 | - DIESEL GENERATOR RELIABILITY | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | <u>TARGET</u> |
| 43796 | BRUNSWICK 1 - SPENT FUEL POOL EXPANSION | | 2 | |
| 42824 | BRUNSWICK 1 - OFF-SITE HAZARD REVIEW | | 1 | |
| 42968 | BRUNSWICK 1 - FULL SCALE REPLICATION FIRE PROTECTION TESTS | | 1 | |
| 43416 | BRUNSWICK 1 - PERIODIC UPDATING OF FINAL SAFETY ANALYSIS REPORTS (FSARS) | | | |
| <u>ANTICIPATED ACTIONS</u> | | | | <u>INITIATION</u> |
| 46505 | BRUNSWICK 1 - RHR HEAT EXCHGR BLOCKAGE | | | 06/10/81 |
| 46511 | BRUNSWICK 1 - DELETION OF SA*PLE STATION NO.36 | | | 06/10/81 |

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CONDENSED MANAGEMENT REPORT

FACILITY: BRUNSWICK 2

PLANT LOCATION: 3 MI N OF SOUTHPORT, NC
 DOCKET NUMBER: 050-00324
 ARCH/ENGINEER: UEC
 IE INSPECTOR: B. RILEY

LICENSED POWER: 2436 MWT
 DESIGN POWER: 0821 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: J. HANNON
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. SHEPPARD

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12195 | BRUNSWICK 2 - MANAG. & TECH. RESOURCES CR-1656 | | 3 | <u>COMPLETE</u> 10/31/80 |
| 10695 | BRUNSWICK 2 - CONTINGENCY PLAN REVIEW | | 1 | 12/10/80 |
| 42985 | BRUNSWICK 2 - REACTOR VESSEL AND RECIRC PUMP SUPPORTS MATERIAL FRACTURE TOUGHNESS NUREG 0577 | | 1 | 01/26/81 |
| 10183 | BRUNSWICK 2 - CONTAINMENT PURGE | | 1 | 01/27/81 |
| 10295 | BRUNSWICK 2 - CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATION DETERIORATION | | 3 | 01/27/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08839 | BRUNSWICK 2 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | <u>TARGET</u> |
| 42728 | BRUNSWICK 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 10906 | BRUNSWICK 2 - APPENDIX I TECH SPEC IMPLEMENTATION | | 3 | |
| 42658 | BRUNSWICK 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 41075 | BRUNSWICK 2 - CONTROL RODS FAIL TO FULLY INSERT | | 3 | |
| 10698 | BRUNSWICK 2 - VITAL AREA ANALYSIS | | 1 | |
| 06171 | BRUNSWICK 2 - SINGLE LOOP OPERATION | | 3 | |
| 43892 | BRUNSWICK UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 10358 | BRUNSWICK 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR | | 3 | |
| 43735 | BRUNSWICK 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 07012 | BRUNSWICK 2 - CONTAINMENT LEAK RATE TESTING | | 2 | |
| 42877 | BRUNSWICK 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 06046 | BRUNSWICK 2 - COMBUSTIBLE GAS CONTROL | | 3 | |
| 07977 | BRUNSWICK 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 06828 | BRUNSWICK 2 - T.S. SURVEILLANCE REQUIREMENTS MECH. SNUBBERS | | 2 | |
| 08927 | BRUNSWICK 2 - RPS POWER SUPPLY | | 1 | |
| 10693 | BRUNSWICK 2 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10963 | BRUNSWICK 2 - 10 CFR 50.55A(G) - ISI WELDS/PIPING | | 1 | |
| 11265 | BRUNSWICK 2 - INSERVICE TESTING AND INSPECTION (IST) PUMPS/VALVES | | 1 | |
| 12123 | BRUNSWICK 2 - EMERGENCY PLAN REVIEW | | 1 | |
| 12187 | BRUNSWICK 2 - HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE | | 3 | |
| 42072 | BRUNSWICK 1 & 2 - ANALYSIS OF TURBINE DISCS IN G.E. TURBINES | | 2 | |
| 42572 | BRUNSWICK 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 07933 | BRUNSWICK 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMEN. | | 1 | |
| 12830 | BRUNSWICK 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 42208 | BRUNSWICK 2 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 42484 | BRUNSWICK 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44008 | BRUNSWICK 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44076 | BRUNSWICK 2 - NUREG-0737 I.A.1.3., SHIFT MANNING OVERTIME LIMITS | | | |
| 44147 | BRUNSWICK 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44218 | BRUNSWICK 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44288 | BRUNSWICK 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44359 | BRUNSWICK 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | <u>(CONTINUATION)</u> | | <u>TARGET</u> |
| 44427 | BRUNSWICK 2 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44497 | BRUNSWICK 2 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44568 | BRUNSWICK 2 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44765 | BRUNSWICK 2 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44836 | BRUNSWICK 2 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44907 | BRUNSWICK 2 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44978 | BRUNSWICK 2 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45049 | BRUNSWICK 2 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45120 | BRUNSWICK 2 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLIN INSTRUMENTATION | | |
| 45329 | BRUNSWICK 2 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45536 | BRUNSWICK 2 - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45564 | BRUNSWICK 2 - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45588 | BRUNSWICK 2 - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45615 | BRUNSWICK 2 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45684 | BRUNSWICK 2 - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45708 | BRUNSWICK 2 - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45732 | BRUNSWICK 2 - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45756 | BRUNSWICK 2 - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45780 | BRUNSWICK 2 - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45811 | BRUNSWICK 2 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45880 | BRUNSWICK 2 - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45904 | BRUNSWICK 2 - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45931 | BRUNSWICK 2 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46002 | BRUNSWICK 2 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46074 | BRUNSWICK 2 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46146 | BRUNSWICK 2 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46218 | BRUNSWICK 2 - NUREG-0737 | III.A.2.1, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46290 | BRUNSWICK 2 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46361 | BRUNSWICK 2 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46432 | BRUNSWICK 2 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42019 | BRUNSWICK 2 - NRC STAFF GENERIC REPORT ON BWRs | | 1 | 10/01/80 |
| 43210 | BRUNSWICK 2 - REQUEST FOR SURVEILLANCE TEST INTERVAL EXTENSIONS (AMEND 54) | | 2 | 12/23/80 |
| 41078 | BSEP-2 CLAFIRY OPERABLE AS IT APPLIES TO BSEP 2 SINGLE FAILURE (AMEND 53) | | 1 | 12/29/80 |
| 42844 | BRUNSWICK 2 - INSTRUMENT/SETPOINT DRIFT | | 1 | 01/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 43417 | BRUNSWICK 2 - NEW APPENDIX R TO 10 CFR 50 REGARDING FIRE PROTECTION FEATURES | | 1 | |
| 42971 | BRUNSWICK 2 - CATEGORY "A" TMI LESSONS LEARNED TS CHANGES | | 1 | |
| 43583 | BRUNSWICK 2 - SERVICE WATER OPERABILITY REQUIREMENTS | | 2 | |
| 43867 | BRUNSWICK 2 - QUALIFICATION OF QA PERSONNEL (GENERIC LETTER 81-01) | | | |
| 43677 | BRUNSWICK 2 - AUGMENTED OFF-GAS SYSTEM IMPROVEMENTS | | 1 | |
| 43795 | BRUNSWICK 2 - QUALITY ASSURANCE PROGRAM REVIEW | | | |
| 42946 | BRUNSWICK 1 - EFFECT OF DC POWER SUPPLY FAILURE ON ECCS PERFORMANCE | | 1 | |
| 43695 | BRUNSWICK 2 - INSERVICE SURVEILLANCE REQUIREMENTS FOR SNUBBERS | | 1 | |
| 43973 | BRUNSWICK 2 - WASTEWATER DISCHARGE WITHOUT COOLING TOWERS | | | |
| 41081 | BRUNSWICK 2 - DIESEL GENERATOR RELIABILITY | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|----------------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| (CONTINUATION) | | | | |
| 43797 | BRUNSWICK 2 - SPENT FUEL POOL EXPANSION | | 2 | |
| 42825 | BRUNSWICK 2 - OFF-SITE HAZARD REVIEW | | 1 | |
| 42969 | BRUNSWICK 2 - FULL SCALE REPLICATION FIRE PROTECTION TESTS | | 1 | |
| 43415 | BRUNSWICK 2 - PERIODIC UPDATING OF FINAL SAFETY ANALYSIS REPORTS (FSARS) | | | |
| <u>ANTICIPATED ACTIONS</u> | | | | <u>INITIATION</u> |
| 46506 | BRUNSWICK 2 - RHR HEAT EXCHGR BLOCKAGE | | | 06/10/81 |
| 46512 | BRUNSWICK 2 - DELETION OF SAMPLE STATION NO.36 | | | 06/10/81 |
| 46514 | BRUNSWICK 2 - TEMPORARY T/S CHG TO RELAX INERTING REQUIREMENT | | | 06/15/81 |

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CONDENSED MANAGEMENT REPORT

FACILITY: CALVERT CLIFFS 1

PLANT LOCATION: 40 MI S OF ANNAPOLIS, MD
 DOCKET NUMBER: 050-00317
 ARCH/ENGINEER: BECH
 IE INSPECTOR: D. HAVERKAMP

LICENSED POWER: 2700 MWT
 DESIGN POWER: 0845 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: E. CONNER
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11105 | CALVERT CLIFFS 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | <u>COMPLETE</u> 10/02/80 |
| 10818 | CALVERT CLIFFS - PWR PRESSURE - TEMPERATURE TECH SPECS | | 1 | 11/14/80 |
| 12506 | CALVERT CLIFFS - CYCLE 5 RELOAD ANALYSIS | | 1 | 12/12/80 |
| 11383 | CALVERT CLIFFS 1 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | 02/05/81 |
| 10639 | CALVERT CLIFFS 1 - GUARD TRAINING PLAN REVIEW | | 1 | 02/20/81 |
| 43025 | CALVERT CLIFFS 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 04/03/81 |
| 07224 | CALVERT CLIFFS 1 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | 04/17/81 |
| 11734 | CALVERT CLIFFS 1 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 04/21/81 |
| 43793 | CALVERT CLIFFS 1&2 - INADVERTENT SAFETY INJECTION DURING SURVEILLANCE | | 2 | 05/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 07978 | CALVERT CLIFFS 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | <u>TARGET</u> |
| 08042 | CALVERT CLIFFS 1 - INSERVICE INSPECTION | | 1 | |
| 08136 | CALVERT CLIFFS 1 - APPENDIX 1 TECH. SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08951 | CALVERT CLIFFS 1 - REVIEW OF ASYMMETRIC LOCA LOADS | | 2 | |
| 08954 | CALVERT CLIFFS 1 - TS SURVEILLANCE FOR MECHANICAL SNUBBERS | | 2 | |
| 08955 | CALVERT CLIFFS 1 - STEAM GENERATOR TUBE DENTING AND SUPPORT PLATE MODIFICATIONS - CE | | 3 | |
| 08971 | CALVERT CLIFFS 1 - TS SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 10372 | CALVERT CLIFFS 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10655 | CALVERT CLIFFS 1 - VITAL AREA ANALYSIS | | 1 | |
| 10953 | CALVERT CLIFFS 1 - BORON SOLUBILITY DURING LONG TERM COOLING FOLLOWING LOCA | | 3 | |
| 11291 | CALVERT CLIFFS 1 - INSERVICE TESTING | | 1 | |
| 12158 | CALVERT CLIFFS 1 - UPGRADED EMERGENCY PLAN | | 1 | |
| 12314 | CALVERT CLIFFS 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE CALVERT CLIFFS 1 | | 3 | |
| 12670 | CALVERT CLIFFS 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 42108 | CALVERT CLIFFS 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 42729 | CALVERT CLIFFS 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43634 | CALVERT CLIFFS 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 42659 | CALVERT CLIFFS 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 08965 | CALVERT CLIFFS 1 - CE POSITION INDICATOR CHANNELS | | 3 | |
| 08967 | CALVERT CLIFFS 1 - CE GENERIC RTD RESPONSE TIME | | 3 | |
| 07522 | CALVERT CLIFFS 1 - CONTAINMENT STRUCTURE POST-TENSIONING SYSTEM | | 2 | |
| 43800 | CALVERT CLIFFS 1 - FIRE PROTECTION NON-APP. R OPEN ITEMS | | 1 | |
| 43888 | CALVERT CLIFFS UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43802 | CALVERT CLIFFS 1 - APPENDIX R ITEMS EXCLUDING SAFE SHUTDOWN | | 1 | |
| 42890 | CALVERT CLIFFS 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43130 | CALVERT CLIFFS 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 12595 | CALVERT CLIFFS 1 - ANAL OF TURBINE DISC CRACKS | | 2 | |
| 42584 | CALVERT CLIFFS 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 08946 | CALVERT CLIFFS 1 - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 43798 | CALVERT CLIFFS 1 - APPENDIX R SAFE, ALTERNATIVE OR DEDICATED SHUTDOWN CAPABILITY | | 1 | |
| 42494 | CALVERT CLIFFS 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|------------------------|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 44009 | CALVERT CLIFFS | 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44077 | CALVERT CLIFFS | 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44148 | CALVERT CLIFFS | 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44219 | CALVERT CLIFFS | 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44289 | CALVERT CLIFFS | 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44360 | CALVERT CLIFFS | 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | |
| 44428 | CALVERT CLIFFS | 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44498 | CALVERT CLIFFS | 1 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44569 | CALVERT CLIFFS | 1 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44661 | CALVERT CLIFFS | 1 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44703 | CALVERT CLIFFS | 1 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44742 | CALVERT CLIFFS | 1 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44766 | CALVERT CLIFFS | 1 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44837 | CALVERT CLIFFS | 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44908 | CALVERT CLIFFS | 1 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | |
| 44979 | CALVERT CLIFFS | 1 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45050 | CALVERT CLIFFS | 1 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45121 | CALVERT CLIFFS | 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45260 | CALVERT CLIFFS | 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | |
| 45306 | CALVERT CLIFFS | 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | |
| 45330 | CALVERT CLIFFS | 1 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45427 | CALVERT CLIFFS | 1 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | |
| 45616 | CALVERT CLIFFS | 1 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45812 | CALVERT CLIFFS | 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45932 | CALVERT CLIFFS | 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46003 | CALVERT CLIFFS | 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46075 | CALVERT CLIFFS | 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46147 | CALVERT CLIFFS | 1 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | |
| 46219 | CALVERT CLIFFS | 1 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46291 | CALVERT CLIFFS | 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46362 | CALVERT CLIFFS | 1 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46433 | CALVERT CLIFFS | 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 07269 | CALVERT CLIFFS | 1 - ELIMINATION OF COMMON RESERVOIRS FOR SAFETY RELATED SNUBBERS | 1 | 12/12/80 |
| 11171 | CALVERT CLIFFS | 1 - TRANSITION TO 18-MO FUEL CYCLES | 1 | 12/12/80 |
| 43000 | CALVERT CLIFFS | 1 - RPS ASYMMETRIC STEAM GENERATOR TRANSIENT PROTECTION TRIP FUNCTION | 2 | 12/12/80 |
| 43451 | CALVERT CLIFFS | 1 - PRELIMINARY REVIEW OF RCS VENTS | 1 | 12/12/80 |
| 43268 | CALVERT CLIFFS | 1 - CONTAINMENT SPRAY PIPING SYSTEM | | 01/07/81 |
| 43390 | CALVERT CLIFFS | 1 - DETERMINATION OF HELB ANALYSIS FOR THE NEWLY ADDED AFW TRAIN | 2 | 01/27/81 |
| 43388 | CALVERT CLIFFS | 1 - THE APPLICABILITY OF CODES & STANDARDS FOR THE NEWLY ADDED AFW TRAIN | 2 | 01/29/81 |
| 42544 | CALVERT CLIFFS | 1 - FIESTA | 2 | 03/13/81 |
| 43561 | CALVERT CLIFFS | 1&2 - INTERPRETATION OF 10 CFR 50.72 REPORTING REQUIREMENTS | 2 | 03/23/81 |
| 42927 | CALVERT CLIFFS | 1 & 2 - ENVIRONMENTAL T.S. CHANGE TO DELETE SOME TEST REQUIREMENTS | 2 | 03/25/81 |
| 42204 | CALVERT CLIFFS | 1 - DEFINITION OF OPERABLE | 1 | 04/03/81 |
| 43560 | CALVERT CLIFFS | 1&2 - USE OF REG GUIDE 1.8, PERSONNEL QUAL & TRAINING | 2 | 04/09/81 |
| 42408 | CALVERT CLIFFS | 1 - AUTO INITIATION OF AUX FEEDWATER | 1 | 05/08/81 |
| 42545 | CALVERT CLIFFS | 1 - BASSS | 2 | 05/20/81 |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|----------------------------|--|-----------------------|-----------------|-------------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 07487 | CALVERT CLIFFS 1 - LOCA CONTROL ROOM OPERATOR DOSE | | 1 | <u>TARGET</u> |
| 11652 | CALVERT CLIFFS 1 - TWO RCPO OPPOSITE LOOP OPERATION | | 1 | |
| 43408 | CALVERT CLIFFS 1 - FOUR MISCELLANEOUS TS CHANGES | | 2 | |
| 43251 | CALVERT CLIFFS 1&2 - ISI REQUIREMENTS EXEMPTION | | | |
| 43558 | CALVERT CLIFFS 1&2 - REVIEW OF AFWS RECOMMENDATION GL-2 | | 2 | |
| 42543 | CALVERT CLIFFS 1 - STATISTICAL COMBINATION OF UNCERTAINTIES | | 2 | |
| 42546 | CALVERT CLIFFS 1 - CEAW | | 2 | |
| 43001 | CALVERT CLIFFS 1 - RELOAD ANALYSIS USING NEW METHOD | | 2 | |
| 43411 | CALVERT CLIFFS 1 - SPENT FUEL STORAGE RACKS CODE PROBLEMS | | 2 | |
| 43480 | CALVERT CLIFFS 1 - REACTOR PV SURVEILLANCE SPECIMENT NO. 263 | | 2 | |
| 43961 | CALVERT CLIFFS 1 - INOPERABLE ACOUSTIC FLOW MONITOR | | | |
| 42389 | CALVERT CLIFFS 1 - FIRE PROTECTION TECH. SPEC. CHANGE | | 2 | |
| 41012 | CALVERT CLIFFS 1 - IREP | | 1 | |
| 41025 | CALVERT CLIFFS 1 - IMPROVEMENT IN THE OFFSITE POWER SOURCES | | 1 | |
| 06560 | CALVERT CLIFFS 1 - LNG HAZARDS STUDY REVIEW | | 1 | |
| 12553 | CALVERT CLIFFS 1 - CRUD BUILDUP IN CORE | | 2 | |
| 43680 | CALVERT CLIFFS 1 - REDUCTION IN CORE BARREL MOVEMENT MONITORING | | 2 | |
| 43678 | CALVERT CLIFFS 1 - BORON DILUTION EVENT POSITIVE ALARM | | 1 | |
| 43998 | CALVERT CLIFFS 1 - LCO & SR CHGS FOR AFWS FLOW ASSURANCE | | | |
| 11960 | CALVERT CLIFFS 1 - LOSS OF FOUR RCP PROTECTION | | 1 | |
| 43682 | CALVERT CLIFFS 1 - ENVIRONMENTAL EFFECTS OF 18 MONTH REFUELING CYCLE | | 2 | |
| 42411 | CALVERT CLIFFS 1 - LONG TERM AFW SYSTEM IMPROVEMENT | | 2 | |
| <u>ANTICIPATED ACTIONS</u> | | | | |
| 06977 | CALVERT CLIFFS 1 - STEAM LINE SAFETY VALVE TOLERANCE | | 1 | <u>INITIATION</u> 07/27/81 |

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CONDENSED MANAGEMENT REPORT

FACILITY: CALVERT CLIFFS 2

PLANT LOCATION: 40 MI S OF ANNAPOLIS, MD
 DOCKET NUMBER: 050-00318
 ARCH/ENGINEER: BECH
 IE INSPECTOR: D. JOHNSON

LICENSED POWER: 2700 MWT
 DESIGN POWER: 8845 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: E. CONNER
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11106 | CALVERT CLIFFS 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | <u>COMPLETE</u> 10/02/80 |
| 08696 | CALVERT CLIFFS 2 - PWR PRESSURE-TEMPERATURE TECH SPEC. | | 1 | 11/14/80 |
| 11382 | CALVERT CLIFFS 2 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | 02/05/81 |
| 08945 | CALVERT CLIFFS 2 - CONTROL ROD GUIDE TUBE WEAR | | 3 | 02/10/81 |
| 10640 | CALVERT CLIFFS 2 - GUARD TRAINING PLAN REVIEW | | 1 | 02/20/81 |
| 43026 | CALVERT CLIFFS 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 04/03/81 |
| 08948 | CALVERT CLIFFS 2 - PWR PUMPS & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | 04/17/81 |
| 11735 | CALVERT CLIFFS 2 - THREE MILE ISLAND FOLLOW-UP WORK | | 1 | 04/21/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 06761 | CALVERT CLIFFS 2 - INSERVICE INSPECTION | | 1 | <u>TARGET</u> |
| 07968 | CALVERT CLIFFS 2 - BORON SOLUBILITY DURING LONG TERM COOLING FOLLOWING LOCA | | 3 | |
| 07979 | CALVERT CLIFFS 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08137 | CALVERT CLIFFS 2 - APPENDIX 1 TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08950 | CALVERT CLIFFS 2 - REVIEW OF ASYMMETRIC LOCA LOADS | | 2 | |
| 08953 | CALVERT CLIFFS 2 - TS SURVEILLANCE FOR MECHANICAL SNUBBERS | | 2 | |
| 08956 | CALVERT CLIFFS 2 - STEAM GENERATOR TUBE DENTING AND SUPPORT PLATE MODIFICATIONS-CE | | 3 | |
| 08970 | CALVERT CLIFFS 2 - TS SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 10373 | CALVERT CLIFFS 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10656 | CALVERT CLIFFS 2 - VITAL AREA ANALYSIS | | 1 | |
| 11292 | CALVERT CLIFFS 2 - INSERVICE TESTING | | 1 | |
| 12159 | CALVERT CLIFFS 2 - UPGRADED EMERGENCY PLAN | | 1 | |
| 12315 | CALVERT CLIFFS 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE CALVERT CLIFFS 2 | | 3 | |
| 12671 | CALVERT CLIFFS 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 42109 | CALVERT CLIFFS 2 - DECAY REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 42730 | CALVERT CLIFFS 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43635 | CALVERT CLIFFS 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 42660 | CALVERT CLIFFS 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 08966 | CALVERT CLIFFS 2 - CEA POSITION INDICATOR CHANNELS | | 3 | |
| 08968 | CALVERT CLIFFS 2 - CE GENERIC RTD RESPONSE TIME | | 3 | |
| 43801 | CALVERT CLIFFS 2 - FIRE PROTECTION NON-APP. R OPEN ITEMS | | 1 | |
| 43889 | CALVERT CLIFFS UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43803 | CALVERT CLIFFS 2 - APPENDIX R ITEMS EXCLUDING SAFE SHUTDOWN | | 1 | |
| 42891 | CALVERT CLIFFS 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43131 | CALVERT CLIFFS 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 12596 | CALVERT CLIFFS 2 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42585 | CALVERT CLIFFS 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 43799 | CALVERT CLIFFS 2 - APPENDIX R SAFE, ALTERNATIVE OR DEDICATED SHUTDOWN CAPABILITY | | 1 | |
| 42495 | CALVERT CLIFFS 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CL1-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|------------------------|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| <u>TARGET</u> | | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|------------------------|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44010 | CALVERT CLIFFS 2 | - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | <u>TARGET</u> |
| 44078 | CALVERT CLIFFS 2 | - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44149 | CALVERT CLIFFS 2 | - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44220 | CALVERT CLIFFS 2 | - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44290 | CALVERT CLIFFS 2 | - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44361 | CALVERT CLIFFS 2 | - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | |
| 44429 | CALVERT CLIFFS 2 | - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44499 | CALVERT CLIFFS 2 | - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44570 | CALVERT CLIFFS 2 | - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44662 | CALVERT CLIFFS 2 | - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44704 | CALVERT CLIFFS 2 | - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44743 | CALVERT CLIFFS 2 | - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44767 | CALVERT CLIFFS 2 | - NUREG-0737 II.E.4.1.2, DELICATED HYDROGEN PENETRATIONS | | |
| 44838 | CALVERT CLIFFS 2 | - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44909 | CALVERT CLIFFS 2 | - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | |
| 44980 | CALVERT CLIFFS 2 | - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45051 | CALVERT CLIFFS 2 | - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45122 | CALVERT CLIFFS 2 | - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45261 | CALVERT CLIFFS 2 | - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | |
| 45307 | CALVERT CLIFFS 2 | - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | |
| 45331 | CALVERT CLIFFS 2 | - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45428 | CALVERT CLIFFS 2 | - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | |
| 45617 | CALVERT CLIFFS 2 | - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45810 | CALVERT CLIFFS 2 | - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45933 | CALVERT CLIFFS 2 | - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46004 | CALVERT CLIFFS 2 | - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46076 | CALVERT CLIFFS 2 | - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46148 | CALVERT CLIFFS 2 | - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | |
| 46220 | CALVERT CLIFFS 2 | - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46292 | CALVERT CLIFFS 2 | - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46363 | CALVERT CLIFFS 2 | - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46434 | CALVERT CLIFFS 2 | - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|------------------------|--|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 43391 | CALVERT CLIFFS 2 | - DETERMINATION OF HELB ANALYSIS FOR THE NEWLY ADDED AFW TRAIN | 2 | <u>COMPLETE</u> 01/27/81 |
| 43389 | CALVERT CLIFFS 2 | - THE APPLICABILITY OF CODES & STANDARDS FOR THE NEWLY ADDED AFW TRAIN | 2 | 01/29/81 |
| 11172 | CALVERT CLIFFS 2 | - TRANSITION TO 18-MO FUEL CYCLES | 1 | 02/10/81 |
| 43249 | CALVERT CLIFFS 2 | - CYCLE 4 RELOAD REVIEW | 2 | 02/10/81 |
| 43452 | CALVERT CLIFFS 2 | - PRELIMINARY REVIEW OF RCS VENTS | 1 | 02/10/81 |
| 43453 | CALVERT CLIFFS 2 | - ELIMINATION OF COMMON RESERVOIRS FOR SAFETY RELATED SNUDDERS | 1 | 02/10/81 |
| 42205 | CALVERT CLIFFS 2 | - DEFINITION OF OPERABLE | 1 | 04/03/81 |
| 42409 | CALVERT CLIFFS 1 | - AUTO INITIATION OF AUX FEEDWATER | 1 | 05/08/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08990 | CALVERT CLIFFS 2 | - STEAM LINE SAFETY VALVE TOLERANCE | 1 | <u>TARGET</u> |
| 11289 | CALVERT CLIFFS 2 | - LOCA CONTROL ROOM OPERATOR DOSE | 1 | |
| 11653 | CALVERT CLIFFS 2 | - TWO RCP OPPOSITE LOOP OPERATION | 1 | |
| 43409 | CALVERT CLIFFS 2 | - FOUR MISCELLANEOUS TS CHANGES | 2 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------|--|-----------------------|-----------------|----------------------|
| | <u>ACTIVE ACTIONS</u> | <u>(CONTINUATION)</u> | | <u>TARGET</u> |
| 43412 | CALVERT CLIFFS 2 - SPENT FUEL STORAGE RACKS CODE PROBLEMS | | 2 | |
| 42390 | CALVERT CLIFFS 2 - FIRE PROTECTION TECH. SPEC. CHANGE | | 2 | |
| 41026 | CALVERT CLIFFS 2 - IMPROVEMENT IN THE OFFSITE POWER SOURCES | | 1 | |
| 08960 | CALVERT CLIFFS 2 - LNG HAZARDS STUDY REVIEW | | 1 | |
| 43681 | CALVERT CLIFFS 2 - REDUCTION IN CORE BARREL MOVEMENT MONITORING | | 2 | |
| 43679 | CALVERT CLIFFS 2 - BORON DILUTION EVENT POSITIVE ALARM | | 1 | |
| 43999 | CALVERT CLIFFS 2 - LCO & SR CHGS FOR AFWS FLOW ASSURANCE | | 1 | |
| 11961 | CALVERT CLIFFS 2 - LOSS OF FOUR RCP PROTECTION | | 1 | |
| 43683 | CALVERT CLIFFS 2 - ENVIRONMENTAL EFFECTS OF 18 MONTH REFUELING CYCLE | | 2 | |

DATA AS OF - 03/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: COOK 1

PLANT LOCATION: 11 MI S OF BENTON HARBOR, MI
 DOCKET NUMBER: 050-00315
 ARCH/ENGINEER: AEP
 IE INSPECTOR: R. MASSE

LICENSED POWER: 3250 MWT
 DESIGN POWER: 1054 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: S. MINER
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11866 | COOK 1 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | COMPLETE 10/31/80 |
| 10923 | COOK 1 - CONTAINMENT LEAK TESTING-APP J (GENERIC) | | 2 | 11/05/80 |
| 08654 | COOK 1 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 01/05/81 |
| 08479 | COOK 1 - PWR PUMT & S/G SUPPORTS, LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | 01/21/81 |
| 11086 | COOK 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 01/30/81 |
| 08527 | COOK 1 - TECH SPEC SURVEILLANCE FOR MECH. SNUBBERS | | 2 | 02/02/81 |
| 12774 | COOK 1 - ANALYSIS OF GE TURBINE DISK CRACKS | | 2 | 02/04/81 |
| 10261 | COOK 1 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 03/31/81 |
| 10484 | COOK 1 - CONTINGENCY PLAN REVIEW | | 1 | 04/13/81 |
| 12923 | COOK 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES ISOLATION VALVES | | 1 | 04/20/81 |
| 43003 | COOK 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 05/12/81 |
| 12171 | COOK 1 - EMERGENCY PLAN REVIEW | | 1 | 05/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 10324 | COOK 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | <u>TARGET</u> |
| 11686 | COOK 1 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | |
| 12264 | COOK 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE COOK 1 | | 3 | |
| 42093 | COOK 1 - DECLY HEAT REMOVAL CAPABILITY TECH SPECS CAPABILITY TECH SPECS | | 2 | |
| 43651 | COOK 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 08316 | COOK 1 - 10 CFR 50.55A(G) - INSERVICE INSPECTION (ISI) | | 1 | |
| 11331 | COOK 1 - INSERVICE TESTING - IST OF PUMPS & VALVES - 10 CFR 50.55A(G) | | 1 | |
| 10486 | COOK 1 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 07980 | COOK 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42661 | COOK 1 - IE BULLETIN 79-27, LOSS OF NOM C'AD3 IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 13111 | COOK 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 10170 | COOK 1 - CONTAINMENT PURGE AT POWER AND EFFECT ON LOCA | | 1 | |
| 42731 | COOK 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42087 | COOK 1 - ANALYSIS OF TURBINE DISCS AND REVIEW RESPONSES | | 2 | |
| 06805 | COOK 1 - REACTOR VESSEL OVERPRESSURE PROTECTION | | 2 | |
| 43917 | D. C. COOK UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | 1 | |
| 10481 | COOK 1 - VITAL AREA ANALYSIS | | 1 | |
| 42859 | COOK 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43132 | COOK 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 08477 | COOK 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08138 | COOK 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08526 | COOK 1 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 42460 | COOK 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>IMMEDIATE ACTIONS</u> | | | | |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44014 | COOK 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44082 | COOK 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44153 | COOK 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |

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(CONTINUATION)

COOK 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|---|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | <u>TARGET</u> |
| 44224 | COOK 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44294 | COOK 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44365 | COOK 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44433 | COOK 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44503 | COOK 1 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44573 | COOK 1 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44633 | COOK 1 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44675 | COOK 1 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |
| 44712 | COOK 1 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44771 | COOK 1 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44842 | COOK 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44913 | COOK 1 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 44984 | COOK 1 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45055 | COOK 1 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45126 | COOK 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45230 | COOK 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45276 | COOK 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45335 | COOK 1 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45393 | COOK 1 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45442 | COOK 1 - NUREG-0737 II.K.3.9, PID CONTROLLER | | | |
| 45472 | COOK 1 - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | | |
| 45502 | COOK 1 - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | | |
| 45621 | COOK 1 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45818 | COOK 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45937 | COOK 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46008 | COOK 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46080 | COOK 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46152 | COOK 1 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46224 | COOK 1 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46296 | COOK 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46367 | COOK 1 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46438 | COOK 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 43224 | COOK 1 - SHIFT STAFFING | | | 01/21/81 |
| 12981 | COOK 1 - CONTAINMENT TEMP EFFECT ON LEVEL MONITORS | | 1 | 01/28/81 |
| 12474 | COOK 1 - REACTOR TRIP TIME CONSTANT CHANGES | | 1 | 05/08/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 43072 | COOK 1 - STRUCTURAL ANALYSIS OF THE CONTAINMENTS ULTIMATE STRENGTH | | 1 | |
| 43688 | COOK 1 - INCREASE FQ CORE PEAKING FACTOR | | | |
| 43484 | DC COOK 1 - LESSONS LEARNED TECH SPECS | | | |
| 43875 | COOK 1 - REVISE SEAL BARRIER MATL TECH SPEC | | | |
| 11597 | COOK 1 - EVN MONITORING & REPORTING MOD IN APPENDIX B TECH SPECS | | 1 | |
| 11421 | COOK 1 - MIS TECH SPEC CHANGES | | 2 | |
| 42554 | COOK 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |
| 43069 | COOK 1 - HYDROGEN CONTROL MEASURES | | 1 | |
| 43523 | COOK 1 - EQUIPMENT SURVIVABILITY | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: COOK 2

PLANT LOCATION: 11 MI S OF BENTON HARBOR, MI
 DOCKET NUMBER: 050-00316
 ARCH/ENGINEER: AEP
 IE INSPECTOR: R. MASSE

LICENSED POWER: 3391 MWT
 DESIGN POWER: 1100 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: S. MINER
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|---|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11867 | COOK 2 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | <u>COMPLETE</u> 10/31/80 |
| 10924 | COOK 2 - CONTAINMENT LEAK TEST - APP J (GENERIC) APPENDIX J | | 2 | 11/05/80 |
| 08655 | COOK 2 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 01/05/81 |
| 08486 | COOK 2 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | 01/21/81 |
| 11087 | COOK 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 01/01/81 |
| 08529 | COOK 2 - TECH SPEC SURVEILLANCE FOR MECHANICAL SNUBBERS | | 2 | 02/02/81 |
| 10262 | COOK 2 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 03/31/81 |
| 10483 | COOK 2 - CONTINGENCY PLAN REVIEW | | 1 | 04/13/81 |
| 12924 | COOK 2 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES ISOLATION VALVES | | 1 | 04/20/81 |
| 12170 | COOK 2 - EMERGENCY PLAN REVIEW | | 1 | 05/20/81 |
| 43004 | COOK 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 05/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 10325 | COOK 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | <u>TARGET</u> |
| 11687 | COOK 2 - THREE MILE ISLAND FOLLOW-UP WORK | | 1 | |
| 12265 | COOK 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE DC COOK 2 | | 3 | |
| 42094 | COOK 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS CAPABILITY TECH SPECS | | 2 | |
| 43652 | COOK 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 10487 | COOK 2 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 08528 | COOK 2 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 42662 | COOK 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 13112 | COOK 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 10169 | COOK 2 - CONTAINMENT PURGE AT POWER AND EFFECT ON LOCA | | 1 | |
| 42732 | COOK 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 08344 | COOK 2 - REACTOR VESSEL OVERPRESSURE PROTECTION | | 2 | |
| 11334 | COOK 2 - INSERVICE TESTING OF PUMP AND VALVES- 10 CFR 50.55 A(G) | | 1 | |
| 43918 | D. C. COOK UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 10480 | COOK 2 - VITAL AREA ANALYSIS | | 1 | |
| 08482 | COOK 2 - 10 CFR 50.55A(G) - ISI | | 1 | |
| 07981 | COOK 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42860 | COOK 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43133 | COOK 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 08480 | COOK 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08139 | COOK 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 42461 | COOK 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u> | | | | |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44015 | COOK 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44083 | COOK 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44154 | COOK 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44225 | COOK 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44295 | COOK 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

COOK 2

| TAC # | TAC DESCRIPTION | IMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|---------------------|---|----------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44366 | COOK 2 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44434 | COOK 2 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44504 | COOK 2 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44574 | COOK 2 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44634 | COOK 2 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44676 | COOK 2 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44713 | COOK 2 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44772 | COOK 2 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44843 | COOK 2 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44914 | COOK 2 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44985 | COOK 2 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45056 | COOK 2 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45127 | COOK 2 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45231 | COOK 2 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45277 | COOK 2 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45336 | COOK 2 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45394 | COOK 2 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45443 | COOK 2 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45473 | COOK 2 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45503 | COOK 2 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45622 | COOK 2 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45819 | COOK 2 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45938 | COOK 2 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46009 | COOK 2 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46081 | COOK 2 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46153 | COOK 2 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46225 | COOK 2 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46297 | COOK 2 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46368 | COOK 2 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46439 | COOK 2 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|--------------------------|--|----------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 43225 | COOK 2 - SHIFT STAFFING | | | <u>COMPLETE</u> 01/21/81 |
| 43085 | COOK 2 - REVISED MODERATOR TEMP COEFF | | | 02/10/81 |
| 11200 | COOK 2 - INSTRUMENT TRIP SETPOINTS | | 1 | 02/12/81 |
| 12475 | COOK 2 - REACTOR TRIP TIME CONSTANT CHANGES | | 1 | 05/08/81 |
| 43818 | COOK 2 - REVISED P-7 PERMISSIVE SET POINT | | | 05/13/81 |
| 43819 | COOK 2 - REVISION TO ADPMS TURN ON POINT | | | 05/13/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 04959 | COOK 2 - CONTAINMENT SUMP TEST | | 1 | <u>TARGET</u> |
| 04997 | COOK 2 - ELEC SYS (BARTONS) QUAL FOR STEAM BREAK/LOCA LOCA | | 1 | |
| 43071 | COOK 2 - STRUCTURAL ANALYSIS OF THE CONTAINMENTS ULTIMATE STRENGTH | | 1 | |
| 43863 | DC COOK 2 - ONE TIME RELIEF FROM TECH SPEC SEC 3.0.4 AND 4.0.4 | | | |
| 43485 | DC COOK 2 - LESSONS LEARNED TECH SPECS | | | |
| 43876 | COOK 2 - REVISE SEAL BARRIER MATL T/S | | | |
| 11598 | COOK 2 - ENV MONITORING & REPORTING MOD IN APPENDIX B TECH SPECS | | 1 | |
| 11515 | COOK 2 - REQUEST OF 2-1379 PROPOSING 24 CHANGES TO APPX A A TECH SPECS | | 2 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

COOK 2

| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | (CONTINUATION) | | <u>TARGET</u> |
| 12463 | COOK 2 - CHECK VALVE TESTING - LIC COND 3(C) | | 1 | |
| 42555 | COOK 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 43070 | COOK 2 - HYDROGEN CONTROL MEASURES | | 1 | |
| 43524 | COOK 2 - EQUIPMENT SURVIVABILITY | | 1 | |
| 12619 | COOK 2 - PANEL & ELEC SWITCHGEAR QUAL-LIC. COND. 3(R) COND. 3(R) | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: COOPER STATION

PLANT LOCATION: 23 MI S OF NEBRASKA CITY, NEB
 DOCKET NUMBER: 050-00298
 ARCH/ENGINEER: B&R
 IE INSPECTOR: D. DUBOIS

LICENSED POWER: 2381 MW
 DESIGN POWER: 0778 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: S. SIEGEL
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 03760 | COOPER - COMBUSTIBLE GAS CONTROL | | 3 | <u>COMPLETE</u> 11/04/80 |
| 10439 | COOPER - CONTINGENCY PLAN REVIEW | | 1 | 11/13/80 |
| 11112 | COOPER - FIRE PROTECTION SER SUPPLEMENT | | 1 | 11/24/80 |
| 11759 | COOPER - THREE MILE ISLAND FOLLOWUP WORK | | 1 | 01/12/81 |
| 11359 | COOPER - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE SURVEILLANCE | | 3 | 01/21/81 |
| 12406 | COOPER - LESSONS LEARNED IMPLEMENTATION | | 1 | 02/23/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 07982 | COOPER - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | <u>TARGET</u> |
| 12300 | COOPER - HELB AND CONSEQUENTIAL SYSTEM FAILURE COOPER | | 3 | |
| 12470 | COOPER - EMERGENCY PLAN REVIEW | | 1 | |
| 12989 | COOPER - ADEQUACY OF STATION ELECTRIC DISTRIBUTION VOLTAGES | | 2 | |
| 13148 | COOPER - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 43402 | COOPER - BWR FEEDWATER NOZZLE AND CONTROL ROD DRIVE CRACKING | | 1 | |
| 07934 | COOPER - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 42227 | COOPER - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 08140 | COOPER - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08295 | COOPER - 10 CFR 50.551(G) INSERVICE INSPECTION - GENERIC | | 1 | |
| 08817 | COOPER - HYDRAULIC SNUBBERS | | 3 | |
| 08938 | COOPER - RPS POWER SUPPLY | | 1 | |
| 10296 | COOPER - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | |
| 10357 | COOPER - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10427 | COOPER - MECHANICAL SNUBBERS | | 2 | |
| 10435 | COOPER - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10436 | COOPER - VITAL AREA ANALYSIS | | 1 | |
| 11040 | COOPER - APPENDIX J CONTAINMENT LEAK TESTING | | 2 | |
| 11251 | COOPER - INSERVICE TESTING (IST) | | 1 | |
| 42733 | COOPER - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42663 | COOPER - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43923 | COOPER STATION - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43729 | COOPER - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42878 | COOPER - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42573 | COOPER - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| <u>TMI ACTIONS</u> | | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42012 | COOPER - OPERABILITY TESTING OF RELIEF & SAFETY VALVES & SAFETY VALVES | | 1 | <u>TARGET</u> |
| 44011 | COOPER - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44079 | COOPER - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44150 | COOPER - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44221 | COOPER - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44291 | COOPER - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44362 | COOPER - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44430 | COOPER - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

COOPER STATION

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|--|---|-----------------|-----------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44500 | COOPER - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | <u>TARGET</u> |
| 44768 | COOPER - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44839 | COOPER - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44910 | COOPER - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44981 | COOPER - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45057 | COOPER - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45123 | COOPER - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45332 | COOPER - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45537 | COOPER - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45565 | COOPER - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45589 | COOPER - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45618 | COOPER - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45685 | COOPER - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45709 | COOPER - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45733 | COOPER - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45757 | COOPER - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45781 | COOPER - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45814 | COOPER - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45881 | COOPER - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45905 | COOPER - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45934 | COOPER - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46005 | COOPER - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46077 | COOPER - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46149 | COOPER - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46221 | COOPER - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46293 | COOPER - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46364 | COOPER - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46435 | COOPER - NUREG-0737 | III D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42009 | COOPER - CONTROL RODS FAIL TO FULLY INSERT | | | <u>COMPLETE</u> 11/04/80 |
| 42014 | COOPER - RESPONSE TO I&E BULLETIN 79-08 | | | 11/04/80 |
| 42370 | COOPER - MAX DISCHARGES TEMPERATURE | | 1 | 11/13/80 |
| 42010 | COOPER - INTERACTION BETWEEN NON SAFETY & SAFETY GRADE SYSTEM | | | 12/30/80 |
| 43233 | COOPER - LICENSING AMENDMENTS TO EXTEND MAPLHGR EXPOSURE LIMITS | | 1 | 01/30/81 |
| 43271 | COOPER - TMI LESSONS LEARNED TECHNICAL SPECIFICATION CHANGES | | | 02/23/81 |
| 43550 | COOPER - RELOAD LICENSING SUBMITTAL - RELOAD 6, CYCLE 7 | | 1 | 05/22/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42007 | COOPER - REVIEW OF CORPORATE CAPABILITIES | | 3 | <u>TARGET</u> |
| 43419 | COOPER - NEW APPENDIX R TO 10 CFR 50 REGARDING FIRE PROTECTION FEATURES | | | |
| 43747 | COOPER - CHANGE TO APPENDIX B TECHNICAL SPECIFICATIONS- DELETION OF RATE OF TEMPERATURE CHANGE | | 2 | |
| 42418 | COOPER - SINGLE LOOP OPERATION | | 1 | |
| 43367 | COOPER - PERIODIC UPDATING OF FSAR | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: CRYSTAL RIVER 3

PLANT LOCATION: 7 MI NW OF CRYSTAL RIVER FLA
 DOCKET NUMBER: 050-00302
 ARCH/ENGINEER: GIL
 IE INSPECTOR: D. QUICK

LICENSED POWER: 2452 MWT
 DESIGN POWER: 0825 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: P. ERICKSON
 BRANCH CHIEF: R. REID
 LIC. ASSISTANT: R. INGRAM

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11109 | CRYSTAL RIVER 3 - FIRE PROTECTION SER SUPPLEMENT | | 1 | <u>COMPLETE</u> 10/14/80 |
| 07468 | CRYSTAL RIVER 3 - SPENT FUEL POOL MODIFICATION | | 1 | 11/17/80 |
| 11796 | CRYSTAL RIVER 3 - FEEDWATER LINE CRACKS | | 2 | 12/09/80 |
| 08883 | CRYSTAL RIVER 3 - MECHANICAL SNUBBERS | | 2 | 02/19/81 |
| 08702 | CRYSTAL RIVER 3 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 03/16/81 |
| 10211 | CRYSTAL RIVER 3 - CONTAINMENT PURGE | | 1 | 03/16/81 |
| 11861 | CRYSTAL RIVER 3 - TMI FOLLOWUP | | 1 | 03/16/81 |
| 12451 | CRYSTAL RIVER 3 - LESSONS LEARNED IMPLEMENTATION | | 1 | 04/17/81 |
| 12883 | CRYSTAL RIVER 3 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12321 | CRYSTAL RIVER 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE CRYSTAL RIVER 3 | | 3 | <u>TARGET</u> |
| 12743 | CRYSTAL RIVER 3 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGE | | 2 | |
| 42121 | CRYSTAL RIVER 3 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43076 | CRYSTAL RIVER 3 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SI DURING SURVEILLANCE TESTING | | 2 | |
| 43263 | CRYSTAL RIVER 3 - B&W ACCIDENT INDUCED NEUTRON FLUX ERRORS (MULTI-PLANT ITEM B-64) | | 3 | |
| 43647 | CRYSTAL RIVER 3 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 42734 | CRYSTAL RIVER 3 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42664 | CRYSTAL RIVER 3 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43912 | CRYSTAL RIVER 3 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42902 | CRYSTAL RIVER 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43037 | CRYSTAL RIVER 3 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 42596 | CRYSTAL RIVER 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |
| 04549 | CRYSTAL RIVER 3 - REVIEW REQUEST FOR RELIEF FROM ASME SEC. XI PUMP & VALVE TESTING REQUIREMENTS | | 1 | |
| 10317 | CRYSTAL RIVER 3 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | |
| 10379 | CRYSTAL RIVER 3 - LOSS OF 125V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08881 | CRYSTAL RIVER 3 - HYDRAULIC SNUBBERS | | 3 | |
| 07983 | CRYSTAL RIVER 3 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08985 | CRYSTAL RIVER 3 - APPENDIX I REVIEW | | 3 | |
| 08843 | CRYSTAL RIVER 3 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 42512 | CRYSTAL RIVER 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| 10729 | CRYSTAL RIVER 3 - CONTINGENCY PLAN REVIEW | | 1 | |
| 10731 | CRYSTAL RIVER 3 - VITAL AREA ANALYSIS | | 1 | |
| 10733 | CRYSTAL RIVER 3 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 08880 | CRYSTAL RIVER 3 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10017 | CRYSTAL RIVER 3 - POTENTIAL EQUIPMENT FAILURES SYSTEM COMPLETE (TASK G) | | 2 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44012 | CRYSTAL RIVER 3 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44080 | CRYSTAL RIVER 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44151 | CRYSTAL RIVER 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |

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CRYSTAL RIVER 3

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|-----------------|--|----------|-------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44222 | CRYSTAL RIVER 3 | - NUREG-0737 I.C.1.2.A. INADEQUATE CORE COOLING GUIDELINES | | |
| 44292 | CRYSTAL RIVER 3 | - NUREG-0737 I.C.1.3A. ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44363 | CRYSTAL RIVER 3 | - NUREG-0737 II.B.1. RCS HIGH POINT VENTS | | |
| 44431 | CRYSTAL RIVER 3 | - NUREG-0737 II.B.3.2. POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44501 | CRYSTAL RIVER 3 | - NUREG-0737 II.B.4.1. TRAINING FOR MITIGATING CORE | | |
| 44571 | CRYSTAL RIVER 3 | - NUREG-0737 II.D.1.2. RELIEF AND SAFETY VALVE TESTING | | |
| 44668 | CRYSTAL RIVER 3 | - NUREG-0737 II.E.1.1. AFW SYSTEM EVALUATION | | |
| 44708 | CRYSTAL RIVER 3 | - NUREG-0737 II.E.1.2.1. AFW SAFETY GRADE AUTO INITIATION | | |
| 44750 | CRYSTAL RIVER 3 | - NUREG-0737 II.E.1.2.2. AFW SAFETY GRADE FLOW INDICATION | | |
| 44769 | CRYSTAL RIVER 3 | - NUREG-0737 II.E.4.1.2. DEDICATED HYDROGEN PENETRATIONS | | |
| 44840 | CRYSTAL RIVER 3 | - NUREG-0737 II.E.4.2. CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44911 | CRYSTAL RIVER 3 | - NUREG-0737 II.F.1.1. NOBLE GAS MONITOR | | |
| 44982 | CRYSTAL RIVER 3 | - NUREG-0737 II.F.1.2. IODINE/ PARTICULATE SAMPLING | | |
| 45053 | CRYSTAL RIVER 3 | - NUREG-0737 II.F.1.3. CONTAINMENT HIGH RANGE MONITOR | | |
| 45124 | CRYSTAL RIVER 3 | - NUREG-0737 II.F.2.3. INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45183 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.2.10. SAFETY GRADE ARTS | | |
| 45190 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.2.11. OPERATOR TRAINING AND DRILLING | | |
| 45197 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.2.13. THERMAL-MECHANICAL REPORT | | |
| 45204 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.2.14. LIFT FREQUENCY OF PORV'S AND SV'S | | |
| 45211 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.2.16. RCP SEAL DAMAGE | | |
| 45217 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.2.17. POTENTIAL FOR VOIDING IN RCS | | |
| 45223 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.2.20. SYSTEM RESPONSE TO SB LOCA | | |
| 45268 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.3.1. AUTO PORV ISOLATION | | |
| 45314 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.3.2. REPORT ON PORV FAILURES | | |
| 45333 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.3.3. REPORT ON RV/SV FAILURES | | |
| 45435 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.3.5. AUTO TRIP OF RCPS | | |
| 45619 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.3.17. ECC SYSTEM OUTAGES | | |
| 45815 | CRYSTAL RIVER 3 | - NUREG-0737 II.K.3.30. SB LOCA OUTLINE | | |
| 45935 | CRYSTAL RIVER 3 | - NUREG-0737 III.A.1.2. TECHNICAL SUPPORT CENTER | | |
| 46006 | CRYSTAL RIVER 3 | - NUREG-0737 III.A.1.2. OPERATIONAL SUPPORT CENTER | | |
| 46078 | CRYSTAL RIVER 3 | - NUREG-0737 III.A.1.2. EMERGENCY OPERATIONS FACILITY | | |
| 46150 | CRYSTAL RIVER 3 | - NUREG-0737 III.A.1.2. NUCLEAR DATA DATA | | |
| 46222 | CRYSTAL RIVER 3 | - NUREG-0737 III.A.1.2. EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46294 | CRYSTAL RIVER 3 | - NUREG-0737 III.A.2.2. METEOROLOGICAL DATA UPGRADE | | |
| 46365 | CRYSTAL RIVER 3 | - NUREG-0737 III.D.3.3. INPLANT RADIATION MONITORING | | |
| 46436 | CRYSTAL RIVER 3 | - NUREG-0737 III.D.3.4. CONTROL ROOM HABITABILITY | | |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|--------------------------|-----------------|---|----------|---------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11951 | CRYSTAL RIVER 3 | - BATTERY SURVEILLANCE | | COMPLETE |
| 11952 | CRYSTAL RIVER 3 | - TECH SPECS RE TMI 2 ORDER | 1 | 12/11/80 |
| 43394 | CRYSTAL RIVER 3 | - ITEM II K.2.15 SLUG FLOW EFFECTS IN OTSC | | 12/31/80 |
| 42266 | CRYSTAL RIVER 3 | - FA HOLDDOWN SPRINGS | | 02/03/81 |
| 11194 | CRYSTAL RIVER 3 | - REVIEW IMPINGEMENT REPORT | | 02/23/81 |
| 11484 | CRYSTAL RIVER 3 | - COMPLETION OF ENVIRONMENTAL STUDIES | 1 | 02/25/81 |
| 42054 | CRYSTAL RIVER 3 | - DELFTION OF NON-RADIOLOGICAL T.S. T.S. | 1 | 02/25/81 |
| 08892 | CRYSTAL RIVER 3 | - STEAM LINE RUPTURE MATRIX SURVEILLANCE | | 02/25/81 |
| 43475 | CRYSTAL RIVER 3 | - II K.2.19 BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW | 1 | 03/02/81 |
| 13014 | CRYSTAL RIVER 3 | - TECH. SEPC MAINTENANCE & REVISION | 1 | 03/06/81 |

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CRYSTAL RIVER 3

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|---|--|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS (CONTINUATION)</u> | | | | <u>COMPLETE</u> |
| 42961 | CRYSTAL RIVER 3 - NUREG 0578 CATEGORY A TECHNICAL SPECIFICATIONS | | | 04/17/81 |
| 43749 | CRYSTAL RIVER 3 - OPERATION WITH ONE MSIV CLOSED | | | 04/17/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12961 | CRYSTAL RIVER 3 - LOSS OF NNI, REACTOR TRIP & ESF ACTUATION | | 1 | |
| 13056 | CRYSTAL RIVER 3 - TENDON SURVEILLANCE TS CHANGE 13 REV 1 | | 3 | |
| 42062 | CRYSTAL RIVER 3 - SAFETY STUDY | | | |
| 42792 | CRYSTAL RIVER 3 - CONFIRMATORY ORDER - ICS RELIABILITY ANALYSIS | | 1 | |
| 11945 | CRYSTAL RIVER 3 - MAIN STREAM SAFETY VALVES | | 1 | |
| 11946 | CRYSTAL RIVER 3 - SYSTEM CYCLES | | 1 | |
| 11950 | CRYSTAL RIVER 3 - AUX FEEDWATER | | 1 | |
| 12347 | CRYSTAL RIVER 3 - REALIGNMENT OF THE QA/QC ORGANIZATION | | 2 | |
| 12541 | CRYSTAL RIVER 3 - OFFSITE ORGANIZATION - TITLE CHANGES | | 2 | |
| 42027 | CRYSTAL RIVER 3 - STEAM GEN. RUPTURE MATRIX DISCONNECT FROM AFW VALVES | | 1 | |
| 12599 | CRYSTAL RIVER 3 - ANALYSIS OF TURBINE DISC CRACKS | | | |
| 42951 | CRYSTAL RIVER 3 - GUIDELINGS FOR HPI USAGE AND OTSG | | | |
| 43385 | CRYSTAL RIVER 3 - SAFETY GRADE ANTICIPATORY TRIP | | | |
| 43426 | CRYSTAL RIVER 3 - ITEM II K.2.13 THERMAL-MECH EFFECTS OF HPI ON VESSEL | | 1 | |
| 11008 | CRYSTAL RIVER 3 - RCP POWER MONITOR MODIFICATION AND TECH. SPECS | | 1 | |
| 10836 | CRYSTAL RIVER 3 - ADMINISTRATIVE CHANGES | | 1 | |
| 08886 | CRYSTAL RIVER 3 - LOWER LIMIT OF DETECTION DEFINITION | | 3 | |
| 08887 | CRYSTAL RIVER 3 - POTASSIUM UNITS | | 3 | |
| 12518 | CRYSTAL RIVER 3 - IMBALANCE ERROR TYPO CORRECTION | | 3 | |
| 12526 | CRYSTAL RIVER 3 - CALIBRATION OF NUCLEAR INSTRUMENTATION CHANGE 53 | | 3 | |

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FACILITY: DAVIS-BESSE 1

PLANT LOCATION: 21 MI E OF TOLEDO, OH
 DOCKET NUMBER: 050-00346
 ARCH/ENGINEER: BECH
 IE INSPECTOR: L. REYES

LICENSED POWER: 2772 MWT
 DESIGN POWER: 0906 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: D. GARNER
 BRANCH CHIEF: R. REID
 LIC. ASSISTANT: R. INGRAM

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10538 | DAVIS BESSE 1 - CONTINGENCY PLAN REVIEW | | 1 | <u>COMPLETE</u> 10/01/80 |
| 10983 | DAVIS BESSE 1 - PWR PRESSURE TEMP LIMITS | | 1 | 11/13/80 |
| 11797 | DAVIS BESSE 1 - FEEDWATER LINE CRACKS | | 2 | 02/02/81 |
| 11377 | DAVIS BESSE 1 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | 02/06/81 |
| 12825 | DAVIS BESSE 1 - EMERGENCY PLANNING REVIEW | | 1 | 04/01/81 |
| 17390 | DAVIS BESSE 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12322 | DAVIS BESSE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE DAVIS BESSE 1 | | 3 | <u>TARGET</u> |
| 12745 | DAVIS BESSE 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 42122 | DAVIS BESSE 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43038 | DAVIS BESSE 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43665 | DAVIS BESSE 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 10541 | DAVIS BESSE 1 - VITAL AREA ANALYSIS | | 1 | |
| 11312 | DAVIS BESSE 1 - HYDRAULIC SNUBBERS | | 3 | |
| 07889 | DAVIS BESSE 1 - DIESEL GENERATOR LOCKOUT | | 3 | |
| 08378 | GENERIC - PWR MODERATOR DILUTION | | 3 | |
| 10993 | DAVIS BESSE 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 11311 | DAVIS BESSE 1 - MECHANICAL SNUBBERS | | 2 | |
| 10984 | DAVIS BESSE 1 - FRACTURE TOUGHNESS & LAMELLAR TEARING OF SG & RCP MATERIALS | | 2 | |
| 11958 | DAVIS BESSE 1 - TECH SPEC CHANGES REG'D BY ORDER | | 1 | |
| 08141 | DAVIS BESSE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 10991 | DAVIS BESSE 1 - LOSS OF 125 VDC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10210 | DAVIS BESSE 1 - CONTAINMENT PURGE | | 1 | |
| 10989 | DAVIS BESSE 1 - CONTAINMENT LEAKAGE DUE TO SEAL DEGRADATION | | 3 | |
| 10535 | DAVIS BESSE 1 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10986 | DAVIS BESSE 1 - RPV SUPPORTS (ASYMETRIC LOCA LOAD) | | 2 | |
| 12615 | DAVIS BESSE 1 - QUALITY ASSURANCE REQUIREMENTS REGARDING DIESEL GENERATOR FUEL OIL | | 3 | |
| 10985 | DAVIS BESSE 1 - REVISED INSERVICE INSPECTION PROGRAM | | 1 | |
| 11309 | DAVIS BESSE 1 - SECONDARY WATER CHEMISTRY | | 2 | |
| 11316 | DAVIS BESSE 1 - INSERVICE TESTING PROGRAM | | 1 | |
| 11119 | DAVIS BESSE 1 - FIRE PROTECTION SUPPLEMENT SER | | 1 | |
| 11322 | DAVIS BESSE 1 - 1 CEA GUIDE TUBE WEAR | | 3 | |
| 42735 | DAVIS BESSE 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42665 | DAVIS BESSE 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43266 | DAVIS BESSE 1 - B&W ACCIDENT INDUCED NEUTRON FLUX ERRORS (MULTI-PLANT ITEM B-64) | | 3 | |
| 43940 | DAVIS-BESSE 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42903 | DAVIS BESSE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42508 | DAVIS BESSE 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TMI ACTIONS</u> | | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42964 | DAVIS BESSE 1 - ACTION PLAN II.E.1.2 AFW INITIATION AND FLOW | | | <u>TARGET</u> |

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(CONTINUATION)

DAVIS-BESSE 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--------------------------|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | <u>(CONTINUATION)</u> | | <u>TARGET</u> |
| 44013 | DAVIS BESSE - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44081 | DAVIS BESSE - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44152 | DAVIS BESSE - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44223 | DAVIS BESSE - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44293 | DAVIS BESSE - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44364 | DAVIS BESSE - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44432 | DAVIS BESSE - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44502 | DAVIS BESSE - NUREG-0737 | II.D.4.1, TRAINING FOR MITIGATING CORE | | |
| 44572 | DAVIS BESSE - NUREG-0737 | II.E.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44669 | DAVIS BESSE - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44751 | DAVIS BESSE - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44770 | DAVIS BESSE - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44841 | DAVIS BESSE - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44912 | DAVIS BESSE - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44983 | DAVIS BESSE - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45054 | DAVIS BESSE - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45125 | DAVIS BESSE - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45184 | DAVIS BESSE - NUREG-0737 | II.K.2.10, SAFETY GRADE ARTS | | |
| 45191 | DAVIS BESSE - NUREG-0737 | II.K.2.11, OPERATOR TRAINING AND DRILLING | | |
| 45198 | DAVIS BESSE - NUREG-0737 | II.K.2.13, THERMAL-MECHANICAL REPORT | | |
| 45205 | DAVIS BESSE - NUREG-0737 | II.K.2.14, LIFT FREQUENCY OF PORV'S AND SV'S | | |
| 45212 | DAVIS BESSE - NUREG-0737 | II.K.2.16, RCP SEAL DAMAGE | | |
| 45218 | DAVIS BESSE - NUREG-0737 | II.K.2.17, POTENTIAL FOR VOIDING IN RCS | | |
| 45224 | DAVIS BESSE - NUREG-0737 | II.K.2.20, SYSTEM RESPONSE TO SB LOCA | | |
| 45269 | DAVIS BESSE - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45315 | DAVIS BESSE - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45334 | DAVIS BESSE - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45436 | DAVIS BESSE - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45620 | DAVIS BESSE - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45817 | DAVIS BESSE - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45936 | DAVIS BESSE - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46007 | DAVIS BESSE - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46079 | DAVIS BESSE - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46151 | DAVIS BESSE - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46223 | DAVIS BESSE - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46295 | DAVIS BESSE - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46366 | DAVIS BESSE - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46437 | DAVIS BESSE - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10990 | DAVIS BESSE 1 - DIVERSE POWER FOR AUX FW SYSTEM (2C(3)(F)) | | 1 | COMPLETE 10/01/80 |
| 11005 | DAVIS BESSE 1 - REACTOR COOLANT SYS FLOW INDICATION (2C(3)(E)) | | 1 | 10/01/80 |
| 11435 | DAVIS BESSE 1 - AUTO ALIGNMENT OF HP1/LPI | | 1 | 10/01/80 |
| 12958 | DAVIS BESSE 1 - CYCLE 2 RELOAD | | 1 | 10/01/80 |
| 43188 | DAVIS BESSE 1 - EMERGENCY TS CHANGE-CONTAINMENT PRESSURE INSTRUMENTS | | | 01/05/81 |
| 43252 | DAVIS BESSE 1 - CHANGE TO BWST AND RECIRCULATION LOGIC | | | 01/05/81 |
| 11956 | DAVIS BESSE 1 - REVIEW RECA HOST VALVE FAIL CLOSE | | 1 | 01/19/81 |
| 42047 | DAVIS BESSE 1 - FA HOLDDOWN SPRINGS | | | 02/09/81 |

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DAVIS-BESSE 1

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|---|--|----------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> (CONTINUATION) | | | | <u>COMPLETE</u> |
| 43392 | DAVIS BESSE 1 - ITEM IIC.2.15 SLUG FLOW EFFECTS IN DTSG | | | 02/17/81 |
| 43477 | DAVIS BESSE 1 - II K.2.19 BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW | | | 02/27/81 |
| 42639 | DAVIS BESSE 1 - DISCONTINUATION OF TS APP. B. PROGRAMS | | 3 | 04/14/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12826 | DAVIS BESSE 1 - INTERMEDIATE RANGE NI SURVEILLANCE | | 3 | |
| 12827 | DAVIS BESSE 1 - BATTERY SURVEILLANCE | | 3 | |
| 42050 | DAVIS BESSE 1 - POTENTIAL FOR TURBINE DISC CRACKS | | | |
| 42152 | DAVIS BESSE 1 - ELIMINATION OF NON-RADIOLOGICAL TS | | 3 | |
| 42793 | DAVIS BESSE 1 - CONFIRMATORY ORDER - ICS RELIABILITY ANALYSIS | | | |
| 43253 | DAVIS BESSE 1 - REVIEW OF SFAS LEVEL 5 SINGLE FAILURE CRITERIA | | 3 | |
| 43297 | DAVIS BESSE 1 - ELIMINATION OF SFAS LEVEL 5 ACTUATION | | 1 | |
| 43333 | DAVIS BESSE 1 - ORGANIZATIONAL CHANGE NO. 2 | | 2 | |
| 43334 | DAVIS BESSE 1 - ASPR INSERTION LIMIT TECH SPECS | | 2 | |
| 43335 | DAVIS BESSE 1 - OVERPRESSURE PROTECTION TECH SPECS | | 1 | |
| 43513 | DAVIS BESSE 1 - DIVERSE POWERED AFW PUMP | | 1 | |
| 43590 | DAVIS BESSE 1 - MODIFICATION TO MEET APPENDIX R, SECTION III.G.3 | | | |
| 43843 | DAVIS BESSE - REVISED DATE FOR SUBMITTING PERSONNEL MONITORING DATA | | | |
| 43964 | DAVIS-BESSE 1 - NUREG 0737 ORDER | | | |
| 11050 | DAVIS BESSE 1 - TRANSIENT ANALYSIS AGAINST SC SET PH | | 3 | |
| 11599 | DAVIS BESSE 1 - TESTING SCHEDULE, RCS MAKEUP ISOLATION ETC ISOLATION ETC | | 1 | |
| 11408 | DAVIS BESSE 1 - TESTING NOISE ISOLATION IN RPS ESFAE | | 3 | |
| 11720 | DAVIS BESSE 1 - DEFEAT OF UNDERVOLTAGE RELAYS | | 1 | |
| 10996 | DAVIS BESSE 1 - INSTRUMENT STATION GRD GRID SYSTEM GRD GRID SYSTEM | | 3 | |
| 04925 | DAVIS BESSE 1 - STAFF GUIDELINES TO APPLICATION FOR SEISMIC REANALYSES | | 3 | |
| 11558 | DAVIS BESSE 1 - AM BC CU SOURCE REMOVAL LIC COND. 2.C.(3).5 | | 3 | |
| 42956 | DAVIS BESSE 1 - NUREG 0578 - CATEGORY A TECHNICAL SPECIFICATIONS | | 1 | |
| 43428 | DAVIS BESSE 1 - ITEM II K.2.13 THERMAL-MECH EFFECTS OF HPI ON VESSEL | | | |
| 42597 | DAVIS BESSE 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: DRESDEN 2

PLANT LOCATION: 9 MI E OF MORRIS, ILL
 DOCKET NUMBER: 050-00237
 ARCH/ENGINEER: S&L
 IE INSPECTOR: T. TONGUE

LICENSED POWER: 2527 MW
 DESIGN POWER: 0794 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: P. OCONNOR
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 08190 | DRESDEN 2 - BWR RELIEF VALVE | | | 10/01/80 |
| 12127 | DRESDEN 2 - EMERGENCY PLAN REVIEW | | 1 | 10/02/80 |
| 08552 | DRESDEN 2 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | 11/13/80 |
| 10668 | DRESDEN 2 - CONTINGENCY PLAN REVIEW | | 1 | 11/17/80 |
| 11098 | DRESDEN 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 12/02/80 |
| 08941 | DRESDEN 2 - RPS POWER SUPPLY | | 1 | 12/11/80 |
| 07986 | DRESDEN 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | 12/22/80 |
| 10800 | DRESDEN 2 - INSERVICE INSPECTION (ISI) | | 1 | 12/31/80 |
| 42213 | DRESDEN 2 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | 01/09/81 |
| 07935 | DRESDEN 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | 01/13/81 |
| 10673 | DRESDEN 2 - GUARD TRAINING PLAN REVIEW | | 1 | 02/11/81 |
| 10763 | DRESDEN 2 - GUARD TRAINING PLANS MER MARGIN + MER MARGIN | | 1 | 02/11/81 |
| 10683 | DRESDEN 2 - JET PUMP INSTRUMENTATION | | 2 | 02/12/81 |
| 12286 | DRESDEN 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE DRESDEN 2 | | 3 | 02/18/81 |
| 08548 | DRESDEN 2 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS | | 3 | 03/23/81 |
| 11131 | DRESDEN 2 - MECHANICAL SNUBBER SURVEILLANCE | | 2 | 03/23/81 |
| 11132 | DRESDEN 2 - HYDRAULIC SNUBBER SURVEILLANCE | | 3 | 03/23/81 |
| 06184 | DRESDEN 2 - LICENSE AMDNMENT FOR SINGLE LOOP OPERATION | | 3 | 05/27/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 10345 | DRESDEN 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10663 | DRESDEN 2 - VITAL AREA ANALYSIS | | 1 | |
| 12765 | DRESDEN 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 43046 | DRESDEN 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 11213 | DRESDEN 2 - INSERVICE TESTING (IST) | | 1 | |
| 07635 | DRESDEN 2 - HIGH DENSITY FUEL STORAGE | | 2 | |
| 08668 | DRESDEN 2 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08110 | DRESDEN 2 - APPENDIX I | | 3 | |
| 42667 | DRESDEN 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43960 | DRESDEN 2 - TEMP AUTHORIZATION FOR SINGLE LOOP OPERATION | | | |
| 43884 | DRESDEN 2 - REVIEW L.P. TURBINE INSPECTION REPORT | | | |
| 43895 | DRESDEN UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43991 | DRESDEN 2 REV PROPOSED MECH SNUBBER T/S | | | |
| 43719 | DRESDEN 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 43988 | DRESDEN 2 - SINGLE LOOP OPERATION (LONG TERM) | | | |
| 43989 | DRESDEN 2 - REVIEW OF NUREG 0612 IMPLEMENTATION | | | |
| 42909 | DRESDEN 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42608 | DRESDEN 2 - LONG TERM REV CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42519 | DRESDEN 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 44027 | INDIAN POINT 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44001 | DRESDEN 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DRESDEN 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|-----------------------|---|-----------------------|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> | | <u>(CONTINUATION)</u> | | |
| 44096 | INDIAN POINT 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44156 | DRESDEN 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44168 | INDIAN POINT 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44227 | DRESDEN 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44239 | INDIAN POINT 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44297 | DRESDEN 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44309 | INDIAN POINT 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44368 | DRESDEN 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44378 | INDIAN POINT 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44436 | DRESDEN 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44447 | INDIAN POINT 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44506 | DRESDEN 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44518 | INDIAN POINT 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44576 | DRESDEN 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44588 | INDIAN POINT 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44639 | INDIAN POINT 2 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44681 | INDIAN POINT 2 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |
| 44718 | INDIAN POINT 2 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44774 | DRESDEN 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44786 | INDIAN POINT 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44845 | DRESDEN 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44857 | INDIAN POINT 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44916 | DRESDEN 2 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 44928 | INDIAN POINT 2 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 44987 | DRESDEN 2 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 44999 | INDIAN POINT 2 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45058 | DRESDEN 2 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45070 | INDIAN POINT 2 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45129 | DRESDEN 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45141 | INDIAN POINT 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45236 | INDIAN POINT 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45282 | INDIAN POINT 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45338 | DRESDEN 2 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45350 | INDIAN POINT 2 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45399 | INDIAN POINT 2 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45448 | INDIAN POINT 2 - NUREG-0737 II.K.3.9, PID CONTROLLER | | | |
| 45478 | INDIAN POINT 2 - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | | |
| 45508 | INDIAN POINT 2 - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | | |
| 45539 | DRESDEN 2 - NUREG-0737 II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | | |
| 45555 | DRESDEN 2 - NUREG-0737 II.K.3.14, ISOLATION OF ISOL. CONDENSERS ON HIGH RADIATION | | | |
| 45567 | DRESDEN 2 - NUREG-0737 II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | | |
| 45591 | DRESDEN 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | | |
| 45624 | DRESDEN 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45636 | INDIAN POINT 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45687 | DRESDEN 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY | | | |
| 45711 | DRESDEN 2 - NUREG-0737 II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | | |
| 45735 | DRESDEN 2 - NUREG-0737 II.K.3.21, RESTART OF CSS AND LPCI | | | |
| 45759 | DRESDEN 2 - NUREG-0737 II.K.3.25, POWER ON PUMP SEALS | | | |
| 45783 | DRESDEN 2 - NUREG-0737 II.K.3.27, COMMON REFERENCE LEVEL | | | |
| 45799 | DRESDEN 2 - NUREG-0737 II.K.3.29, PERFORMANCE OF ISOLATION CONDENSER | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DRESDEN 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 45821 | DRESDEN 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | <u>TARGET</u> |
| 45833 | INDIAN POINT 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45883 | DRESDEN 2 - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | | |
| 45907 | DRESDEN 2 - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION | | | |
| 45940 | DRESDEN 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 45952 | INDIAN POINT 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46011 | DRESDEN 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46023 | INDIAN POINT 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46083 | DRESDEN 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46095 | INDIAN POINT 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46155 | DRESDEN 2 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46167 | INDIAN POINT 2 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46227 | DRESDEN 2 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46239 | INDIAN POINT 2 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46299 | DRESDEN 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46311 | INDIAN POINT 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46370 | DRESDEN 2 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46382 | INDIAN POINT 2 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46441 | DRESDEN 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |
| 43220 | DRESDEN 2 - SHIFT TECHNICAL ADVISOR | | 1 | 12/10/80 |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10954 | DRESDEN 2 - ENVIRONMENTAL TECH SPECS | | | <u>COMPLETE</u> |
| 43267 | DRESDEN 2 - CHIMNEY MONITOR OUT OF SERVICE | | 1 | 10/10/80 |
| 10686 | DRESDEN 2 - OPERATION WITH ADS VALVE OUT OF SERVICE | | | 12/29/80 |
| 43456 | DRESDEN 2 - T.S. FOR TMI 2 CATEGORY A LESSONS LEARNED ITEMS | | 1 | 01/08/81 |
| 08556 | DRESDEN 2 - SUMP FLOW MONITOR SURVEILLANCE FREQUENCY | | | 02/06/81 |
| 43454 | DRESDEN 2 - TECH. SPECS. TO IMPLEMENT 10 CFR 50.59 RELOADS | | 3 | 03/27/81 |
| 12487 | DRESDEN 2 - LOAD LINE LIMIT ANALYSIS | | 1 | 03/31/81 |
| | | | 2 | 04/30/81 |

| <u>ACTIVE ACTIONS</u> | | | | |
|-----------------------|---|--|---|---------------|
| 06820 | DRESDEN 2 - LPRM DRIFT DUE TO SEAL LEAKAGE | | | <u>TARGET</u> |
| 07578 | DRESDEN 2 - CONTAINMENT LEAK RATE LIST - REDUCE TESTING TIME | | 3 | |
| 07644 | DRESDEN 2 - CONTROL ROD COUPLING VERIFICATION | | 1 | |
| 11237 | DRESDEN 2 - EXEMPTION TO APPEN J SECT III A.1.(D) | | 3 | |
| 12212 | DRESDEN 2 - IPCLRT SCHEDULE | | 1 | |
| 08578 | DRESDEN 2 - INTERFACILITY TRANSFER OF SPENT FUEL BETWEEN QUAD CITIES AND DRESDEN STATIONS | | 2 | |
| 42642 | DRESDEN 2 - ORGANIZATIONAL CHANGES - STATION AND CORPORATE | | 3 | |
| 43824 | DELETE HYDRAULIC SNUBBER TECH SPECS | | 2 | |
| 11222 | DRESDEN 2 - CONVERSION POL TO FTOL | | | |
| 43987 | DRESDEN 2 - REQUEST FOR DELAY IN USING ODYN TRANSIENT ANALYSIS MODEL | | 3 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: DRESDEN 3

PLANT LOCATION: 9 MI E OF MORRIS, ILL
 DOCKET NUMBER: 050-00249
 ARCH/ENGINEER: S&L
 IE INSPECTOR: T. TONGUE

LICENSED POWER: 2527 MWT
 DESIGN POWER: 0794 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. BEVAN
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12128 | DRESDEN 3 - EMERGENCY PLAN REVIEW | | 1 | 10/02/80 |
| 08483 | DRESDEN 3 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | 11/13/80 |
| 10669 | DRESDEN 3 - CONTINGENCY PLAN REVIEW | | 1 | 11/17/80 |
| 08942 | DRESDEN 3 - RPS POWER SUPPLY | | 1 | 12/11/80 |
| 10674 | DRESDEN 3 - GUARD TRAINING PLAN REVIEW | | 1 | 12/15/80 |
| 07987 | DRESDEN 3 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | 12/22/80 |
| 08288 | DRESDEN 3 - INSERVICE INSPECTION (ISI)-GENERIC ING - GENERIC | | 1 | 12/30/80 |
| 08540 | DRESDEN 3 - COMMONWEALTH COMBUSTIBLE GAS CONTROL (CAD/CAM) (CAD/CAM) | | 3 | 01/15/81 |
| 10798 | DRESDEN 3 - COMMONWEALTH EDISON CO. JET PUMP INSTRUMENTATION | | 3 | 02/01/81 |
| 11721 | DRESDEN 3 - TMI 2 FOLLOW-UP | | 1 | 02/01/81 |
| 08633 | DRESDEN 3 - PHYSICAL SECURITY PROGRAM | | 1 | 02/10/81 |
| 11120 | DRESDEN 3 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/12/81 |
| 42226 | DRESDEN 3 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | 05/05/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12301 | DRESDEN 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE DRESDEN 3 | | 3 | <u>TARGET</u> |
| 13231 | DRESDEN 3 - REVIEW OF CORPORATE CAPABILITIES | | 3 | |
| 13243 | DRESDEN 3 - GENERIC CLARIFICATION OF "OPERABILITY" | | 2 | |
| 42738 | DRESDEN 3 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42668 | DRESDEN 3 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 08546 | DRESDEN 3 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SHUBBERS | | 2 | |
| 10359 | DRESDEN 3 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 43896 | DRESDEN UNIT 3 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 10021 | DRESDEN 3 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH DEGRADED GRID VOLTAGE | | 2 | |
| 43722 | DRESDEN 3 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 11278 | DRESDEN 3 - INSERVICE TESTING | | 1 | |
| 10664 | DRESDEN 3 - VITAL AREA ANALYSIS | | 1 | |
| 42879 | DRESDEN 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 10956 | DRESDEN 3 - SINGLE LOOP OPERATION | | 3 | |
| 07936 | DRESDEN 3 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 08111 | DRESDEN 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08669 | DRESDEN 3 - CONTAINMENT LEAK TESTING - APP. J (GEN. RIC) | | 2 | |
| 42574 | DRESDEN 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42487 | DRESDEN 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44086 | DRESDEN 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | <u>TARGET</u> |
| 44157 | DRESDEN 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44228 | DRESDEN 3 - NUREG-0737 I.C.1.2.A, INADEQUATE RE COOLING GUIDELINES | | | |
| 44298 | DRESDEN 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44369 | DRESDEN 3 - NUREG-0737 II.B.1, RCS HIGH POINT NTS | | | |
| 44437 | DRESDEN 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44507 | DRESDEN 3 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DRESDEN 3

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
|--------------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| (CONTINUATION) | | | | |
| 44577 | DRESDEN 3 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44775 | DRESDEN 3 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44846 | DRESDEN 3 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44917 | DRESDEN 3 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 44988 | DRESDEN 3 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45059 | DRESDEN 3 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45130 | DRESDEN 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45339 | DRESDEN 3 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45540 | DRESDEN 3 - NUREG-0737 II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | | |
| 45556 | DRESDEN 3 - NUREG-0737, II.K.3.14, ISOLATION OF ISOL. CONDENSERS ON HIGH RADIATION | | | |
| 45568 | DRESDEN 3 - NUREG-0737 II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | | |
| 45592 | DRESDEN 3 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | | |
| 45625 | DRESDEN 3 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45688 | DRESDEN 3 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY | | | |
| 45712 | DRESDEN 3 - NUREG-0737 II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | | |
| 45736 | DRESDEN 3 - NUREG-0737 II.K.3.21, RESTART OF CSS AND LPCI | | | |
| 45760 | DRESDEN 3 - NUREG-0737 II.K.3.25, POWER ON PUMP SEALS | | | |
| 45784 | DRESDEN 3 - NUREG-0737 II.K.3.27, COMMON REFERENCE LEVEL | | | |
| 45800 | DRESDEN 3 - NUREG-0737 II.K.3.29, PERFORMANCE OF ISOLATION CONDENSER | | | |
| 45822 | DRESDEN 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45884 | DRESDEN 3 - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | | |
| 45908 | DRESDEN 3 - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION | | | |
| 45941 | DRESDEN 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46012 | DRESDEN 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46084 | DRESDEN 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46156 | DRESDEN 3 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46228 | DRESDEN 3 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46300 | DRESDEN 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46371 | DRESDEN 3 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46442 | DRESDEN 3 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |
| 43221 | DRESDEN 3 - SHIFT TECHNICAL ADVISOR | | 1 | 12/10/80 |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42381 | DRESDEN 3 - THREE CPS SRM COUNT RATE | | 3 | 11/01/80 |
| 08569 | DRESDEN 3 - OPERATION WITH ONE ADS VALVE OUT OF SERVICE | | 3 | 12/30/80 |
| 42368 | DRESDEN 3 - MODEL TECH SPECS FOR IMPLEMENTATION OF TMI-2 CAT. A LL ITEMS | | 1 | 02/06/81 |
| 10958 | DRESDEN 3 - SUMP FLOW MONITORING REQUIREMENTS | | 1 | 03/27/81 |
| 43703 | DRESDEN 3 - REMOVAL OF SNUBBERS FROM HPCI STEAM LINE | | 1 | 04/01/81 |
| 12488 | DRESDEN 3 - LOAD LINF LIMIT ANALYSIS | | 1 | 04/30/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12807 | DRESDEN 3 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | | |
| 42999 | DRESDEN 3 - CURRENT EVENTS-ACTIVITIES RESULTING FROM DAILY PLANT OPERATION | | 1 | |
| 43618 | DRESDEN 3 - ODDN CODE | | | |
| 43620 | DRESDEN 3 - UPDATING OF FSAR | | 3 | |
| 43859 | DRESDEN 3 - QUALITY ASSURANCE REQUIREMENTS | | 1 | |
| 42643 | DRESDEN 3 - ORGANIZATIONAL CHANGES - STATION AND CORPORATE | | 1 | |
| 08586 | DRESDEN 3 - CONTROL ROD COUPLING VERIFICATION | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DRESDEN 3

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| | (CONTINUATION) | | | |
| 08579 | DRESDEN 3 - INTERFACILITY TRANSFER OF SF BETWEEN QC & DRESDEN STATIONS | | 2 | |
| 43587 | DRESDEN 3 - USE OF XN-3 FUEL FOR RELOAD 8, PER CECO LTR OF 2/20/81 AND 3/5/81 | | 1 | |
| 43977 | DRESDEN 3 - BWR SCRAM DISCHARGE SYSTEM TECH SPEC CHANGES | | | |
| 43978 | DRESDEN 3 - BWR SCRAM DISCH SYS-LONG TERM FIX TO SDV-IV COUPLING | | | |
| 43990 | DRESDEN 3 - FIRE PROTECTION TECH SPEC | | | |
| 12213 | DRESDEN 3 - IPCLRT SCHEDULE | | 1 | |
| 08585 | DRESDEN 3 - PCILRT - REDUCE TESTING TIME | | 1 | |

DATE AC OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: DUANE ARNOLD

PLANT LOCATION: 8 MI NW OF CEDAR RAPIDS, IA
 DOCKET NUMBER: 050-00331
 ARCH/ENGINEER: BECH
 IE INSPECTOR: G. WRIGHT

LICENSED POWER: 1658 MWT
 DESIGN POWER: 0538 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: K. ECCLESTON
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. SHEPPARD

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|---|--|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10298 | DUANE ARNOLD - CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATION | | 3 | <u>COMPLETE</u> 11/14/80 |
| 42222 | DUANE ARNOLD - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | 01/09/81 |
| 41058 | DUANE ARNOLD - DIESEL FUEL OIL-GENERIC | | 3 | 01/19/81 |
| 12169 | DUANE ARNOLD - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 42739 | DUANE ARNOLD - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | 05/13/81 |
| 08718 | DUANE ARNOLD - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | 05/22/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12302 | DUANE ARNOLD - HELB AND CONSEQUENTIAL SYSTEM FAILURE DUANE ARNOLD | | 3 | <u>TARGET</u> |
| 12859 | DUANE ARNOLD - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 13264 | DUANE ARNOLD - REVIEW OF CORPORATE CAPABILITIES | | 3 | |
| 43018 | DUANE ARNOLD - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 08940 | DUANE ARNOLD - RPS POWER SUPPLY | | 1 | |
| 06500 | DUANE ARNOLD - SINGLE LOOP OPERATION | | 3 | |
| 10360 | DUANE ARNOLD - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 11035 | DUANE ARNOLD - LEAK RATE TESTING | | 2 | |
| 10458 | DUANE ARNOLD - VITAL AREA ANALYSIS | | 1 | |
| 08723 | DUANE ARNOLD - MECHANICAL SNUBBERS | | 2 | |
| 08724 | DUANE ARNOLD - HYDRAULIC SNUBBERS | | 3 | |
| 11323 | DUANE ARNOLD - INSERVICE TESTING | | 1 | |
| 08142 | DUANE ARNOLD - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 07937 | DUANE ARNOLD - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 08434 | DUANE ARNOLD - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | |
| 11083 | DUANE ARNOLD - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 10462 | DUANE ARNOLD - CONTINGENCY PLAN REVIEW | | 1 | |
| 07988 | DUANE ARNOLD - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 10454 | DUANE ARNOLD - GUARD TRAINING PLAN REVIEW | | 1 | |
| 42669 | DUANE ARNOLD - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43730 | DUANE ARNOLD - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42880 | DUANE ARNOLD - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42575 | DUANE ARNOLD - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42488 | DUANE ARNOLD - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC # TAC DESCRIPTION MULTI-ACTIONS PRIORITY CRITICAL DATE</u> | | | | |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44017 | DUANE ARNOLD - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44087 | DUANE ARNOLD - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44158 | DUANE ARNOLD - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44229 | DUANE ARNOLD - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44299 | DUANE ARNOLD - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44370 | DUANE ARNOLD - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44438 | DUANE ARNOLD - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44508 | DUANE ARNOLD - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DUANE ARNOLD

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|--------------------------------------|---------------------------|---|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44578 | DUANE ARNOLD - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44776 | DUANE ARNOLD - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44847 | DUANE ARNOLD - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44918 | DUANE ARNOLD - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44989 | DUANE ARNOLD - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45060 | DUANE ARNOLD - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45131 | DUANE ARNOLD - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45340 | DUANE ARNOLD - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45541 | DUANE ARNOLD - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45569 | DUANE ARNOLD - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45593 | DUANE ARNOLD - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45626 | DUANE ARNOLD - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45689 | DUANE ARNOLD - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45713 | DUANE ARNOLD - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45737 | DUANE ARNOLD - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45761 | DUANE ARNOLD - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45785 | DUANE ARNOLD - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45823 | DUANE ARNOLD - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45885 | DUANE ARNOLD - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45909 | DUANE ARNOLD - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45942 | DUANE ARNOLD - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46013 | DUANE ARNOLD - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46085 | DUANE ARNOLD - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46157 | DUANE ARNOLD - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46229 | DUANE ARNOLD - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46301 | DUANE ARNOLD - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46372 | DUANE ARNOLD - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46443 | DUANE ARNOLD - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>COMPLETE</u> |
|--------------------------|--|-----------------------|-----------------|---|
| <u>COMPLETED ACTIONS</u> | | | | |
| 13263 | DUANE ARNOLD - NRC STAFF GENERIC REPORT ON BWRs | | 1 | 10/24/80 |
| 41060 | DUANE ARNOLD - GENERIC CLARIFICATION OF | | 1 | 01/19/81 |
| 43499 | DUANE ARNOLD - 2/20/80 APPLICATION RE: REPLACEMENT OF NITROGEN SUPPLY POWER OPERATED VALVE WITH STOP CHECK | | 2 | 03/23/81 |
| 42168 | DUANE ARNOLD - REQUEST RE: DEFINITION OF OPERABLE ET AL | | 2 | 04/03/81 |
| 43858 | DAEC - TECH SPEC REVISIONS TO CORRECT ISOLATION SIGNALS RECEIVED | | | 05/15/81 |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|-----------------------|---|-----------------------|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 13090 | DUANE ARNOLD - STANDBY GAS TREATMENT SYSTEM FILTERS | | 2 | |
| 13091 | DUANE ARNOLD - REACTIVITY ANOMALITIES | | 2 | |
| 13093 | DUANE ARNOLD - MAPLHGR SURVEILLANCE REQUIREMENTS | | 2 | |
| 41057 | DUANE ARNOLD - DIESEL GENERATOR RELIABILITY | | 1 | |
| 41059 | DUANE ARNOLD - CONTROL RODS FAIL TO FULLY INSERT | | 1 | |
| 41062 | DUANE ARNOLD - DC POWER SUPPLY FAILURE EFFECT ECCS | | 1 | |
| 41063 | DUANE ARNOLD - TURBINE CRACKING (GE) | | 1 | |
| 42255 | DUANE ARNOLD - LESSONS LEARNED TECH SPECS | | 1 | |
| 42258 | DUANE ARNOLD - SCRAM DISCHARGE VOL TECH SPECS | | 1 | |
| 42420 | DUANE ARNOLD - ADDITIONAL TMI RELATED REQUIREMENTS | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DUANE ARNOLD

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| | (CONTINUATION) | | | |
| 43692 | DUANE ARNOLD - ALTERNATIVE SHUTDOWN CAPABILITY | | 1 | |
| 06395 | DUANE ARNOLD - VERIFICATION OF SENSOR RESPONSE TIME | | 1 | |
| 07263 | DUANE ARNOLD - MAIN STEAM LINE ISOLATION VALVES - LEAKAGE CONTROL SYSTEMS | | 1 | |
| 07396 | DUANE ARNOLD - RECATEGORIZATION OF TURBINE TRIP WITHOUT BYPASS | | 1 | |
| 42066 | DUANE ARNOLD - REQUEST RE: CLARIFICATION OF FUEL MOVING EQUIPMENT | | 2 | |
| 43126 | DUANE ARNOLD - SINGLE RECIRCULATION LOOP OPERATION | | 1 | |
| 43958 | DUANE ARNOLD - SURVEILLANCE REQUIREMENTS FOR RIVER WATER SUPPLY PUMPS | | 1 | |
| 43493 | DUANE ARNOLD - SNUBBER SURVEILLANCE TECH SPECS - DAEC | | 1 | |
| 43495 | DUANE ARNOLD - PERIODIC UPDATING OF FINAL SAFETY ANALYSIS REPORT - INITIAL SUBMITTAL | | 1 | |
| 43498 | DUANE ARNOLD - 50.48 AND 10 CFR 50 APPENDIX R FIRE PROTECTION REQUIREMENTS | | 1 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: FARLEY 1

PLANT LOCATION: 28 MI SE OF DOTHAN, ALA
 DOCKET NUMBER: 050-00348
 ARCH/ENGINEER: SSC
 IE INSPECTOR: W. BRADFORD

LICENSED POWER: 2652 MWT
 DESIGN POWER: 0829 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: E. REEVES
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 08676 | FARLEY 1 - CONTROL ROD GUIDE TUBE WEAR | | 3 | <u>COMPLETE</u> 10/03/80 |
| 08656 | FARLEY 1 - PWR PRESSURE TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 10471 | FARLEY 1 - CONTINGENCY PLAN REVIEW | | 1 | 12/10/80 |
| 11868 | FARLEY 1 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 12/12/80 |
| 04836 | FARLEY 1 - DEGRADED GRID VOLTAGE PROTECTION | | 2 | 03/09/81 |
| 13006 | FARLEY 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | 03/09/81 |
| 12168 | FARLEY 1 - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 12925 | FARLEY 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 11688 | FARLEY 1 - THREE MILE ISLAND FOLLOWUP WORK (79-06A) | | 1 | 05/27/81 |

ACTIVE ACTIONS

| | | | | |
|-------|--|--|---|---------------|
| 10326 | FARLEY 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | <u>TARGET</u> |
| 12266 | FARLEY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE JOSEPH M. FARLEY 1 | | 3 | |
| 43631 | FARLEY 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 08673 | FARLEY 1 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUCHNESS | | 2 | |
| 11277 | FARLEY 1 - 10 CFR 50.55A 1ST-PUMP AND VALVE PROGRAM (SECOND 20 MONTHS) | | 1 | |
| 42670 | FARLEY 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 10472 | FARLEY 1 - VITAL AREA ANALYSIS | | 1 | |
| 10470 | FARLEY 1 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 08094 | FARLEY 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 11099 | FARLEY 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 42095 | FARLEY 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 42740 | FARLEY 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43005 | FARLEY 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 08674 | FARLEY 1 - TECH SPEC SURV FOR HYDRAULIC SNUBBERS | | 3 | |
| 08675 | FARLEY 1 - TECH SPECS FOR MECHANICAL SNUBBERS | | 2 | |
| 12594 | FARLEY 1 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 07989 | FARLEY 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 43885 | FARLEY UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42861 | FARLEY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43134 | FARLEY 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42556 | FARLEY 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42462 | FARLEY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-ELI-21) | | 1 | |

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
|-----------------------|--|-------------|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44018 | FARLEY 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44088 | FARLEY 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44159 | FARLEY 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44230 | FARLEY 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44300 | FARLEY 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44371 | FARLEY 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44439 | FARLEY 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FAPLEY 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| (CONTINUATION) | | | | |
| 44509 | FARLEY 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44579 | FARLEY 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44635 | FARLEY 1 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44677 | FARLEY 1 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44714 | FARLEY 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44777 | FARLEY 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44848 | FARLEY 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44919 | FARLEY 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44990 | FARLEY 1 - NUREG-0737 | II.F.1.2, IODINE/PARTICULATE SAMPLING | | |
| 45061 | FARLEY 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45132 | FARLEY 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45232 | FARLEY 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45278 | FARLEY 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45341 | FARLEY 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45395 | FARLEY 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45444 | FARLEY 1 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45474 | FARLEY 1 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45504 | FARLEY 1 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45627 | FARLEY 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45824 | FARLEY 1 - NUREG-0737 | II.K.3.30, SB LGCA OUTLINE | | |
| 45943 | FARLEY 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46014 | FARLEY 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46086 | FARLEY 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46158 | FARLEY 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46230 | FARLEY 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46302 | FARLEY 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46373 | FARLEY 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46444 | FARLEY 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42829 | FARLEY 1 - CHANGE FXY LIMIT IN 20 TO 60 PERCENT AXIAL HEIGHT REGION (T.S. 4.2.2.2.E) | | | 10/30/80 |
| 12502 | FARLEY 1 - HIGH RADIATION AREA ADMINISTRATIVE CHANGE | | 1 | 12/10/80 |
| 12569 | FARLEY 1 - ADD FEEDWATER BYPASS VALVES TO T.S. | | 1 | 12/10/80 |
| 13075 | FARLEY 1 - TECH SPEC CHANGES; ROD BOW PENALTY | | 1 | 02/13/81 |
| 43368 | FARLEY 1 - HEATUP AND COOLDOWN CURVES FOR CAPSULE Y ANALYSIS | | | 02/13/81 |
| 43608 | FARLEY 1&2 - TEMPORARY (TECH SPEC 3.7.4) CHANGE FOR SERVICE RECIRC LINE MODS (EXPEDITED ACTION) | | | 04/03/81 |
| 42173 | FARLEY 1 - CLARIFY RECORDS REQUIRED BY 10 CFR 20 | | 2 | 04/22/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 43711 | FARLEY 1&2 - UPGRADING OF EMERGENCY SUPPORT FACILITIES | | 1 | |
| 11547 | FARLEY 1 - FOUR SPECIAL ENVIRONMENTAL REPORTS | | 2 | |
| 12963 | FARLEY 1 - DELETE TECH SPEC 4.7.1.2.2.B.1.A MAIN FEED PUMP LESS TO AUX FEED PUMPS | | 2 | |
| 13080 | FARLEY 1 - TECH SPEC CHANGE FOR ORGANIZATIONAL MODIFICATION | | 1 | |
| 42174 | FARLEY 1 - DELETE APPENDIX B-NON RADIOLOGICAL ENVIRONMENTAL TECH SPECS | | 2 | |
| 43073 | FARLEY 1 - CHANGE HYDRAULIC SNUBBERS DESIGNATIONS | | 2 | |
| 43382 | FARLEY 1 - CHANGE TECH SPECS UNIT 1 TO AGREE WITH UNIT 2 TECH SPECS | | 2 | |
| 43479 | FARLEY 1 - REQUEST FOR EXEMPTION TO 10 CFR 50 APPENDIX J PARAGRAPH III.D.2(B) | | 2 | |
| 43860 | FARLEY 1/2 - ONE TIME TECH SPEC CHANGE AL EXTENSION OF OUTAGE TIME | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: FARLEY 2

PLANT LOCATION: 28 MI SE OF DOTHAN, ALA
 DOCKET NUMBER: 050-00364
 ARCH/ENGINEER: SSC
 IE INSPECTOR: F. JAPE

LICENSED POWER: 000 MWT
 DESIGN POWER: 0829 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: J. THOMA
 BRANCH CHIEF: B. YOUNGBLOOD
 LIC. ASSISTANT: M. RUSHBROOK

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 43952 | FARLEY 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | <u>TARGET</u> |
| 42534 | FARLEY 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44019 | FARLEY 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |

| | | | | |
|-------|---|--|--|--|
| 44089 | FARLEY 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44160 | FARLEY 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44231 | FARLEY 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44301 | FARLEY 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44372 | FARLEY 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44440 | FARLEY 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44510 | FARLEY 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44580 | FARLEY 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44636 | FARLEY 2 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44678 | FARLEY 2 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |
| 44715 | FARLEY 2 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44778 | FARLEY 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44849 | FARLEY 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44920 | FARLEY 2 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 44991 | FARLEY 2 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45062 | FARLEY 2 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45133 | FARLEY 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45233 | FARLEY 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45279 | FARLEY 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45342 | FARLEY 2 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45396 | FARLEY 2 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45445 | FARLEY 2 - NUREG-0737 II.K.3.9, PID CONTROLLER | | | |
| 45475 | FARLEY 2 - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | | |
| 45505 | FARLEY 2 - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | | |
| 45628 | FARLEY 2 - NUREG-0737 II.K.3.17, ECC SYSTEM GUTAGES | | | |
| 45825 | FARLEY 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45944 | FARLEY 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46015 | FARLEY 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46087 | FARLEY 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46159 | FARLEY 2 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46231 | FARLEY 2 - NUREG-0737 III.A.1.2, EMERGENCY PLAN GRADE TO MEET RULE | | | |
| 46303 | FARLEY 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46374 | FARLEY 2 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46445 | FARLEY 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|------------------------|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| | | | | <u>TARGET</u> |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FARLEY 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | TARGET |
| 05120 | DESIGN OF ONSITE AUXILIARY POWER SYSTEM FOR FARLEY UNIT 2 | | | |
| 43200 | FARLEY 2 - RELIEF FROM DIESEL GEN SURVEILLANCE REQUIREMENTS | | | |
| 43213 | FARLEY 2 - AUGMENTED LOW POWER TEST PROGRAM AND PROCEDURES | | | |
| 43701 | FARLEY 2 - SCHEDULE FOR IMPLEMENTING R.G. 1.97 ON INSTRUMENTS NECESSARY DURING AND FOLLOWING ACCIDENTS | | | |
| 43830 | FARLEY 2 - AMENDMENT 2 - D.G. 2C TESTING | | | |
| 43574 | FARLEY 2 - AUGMENTED LOW POWER TEST OF BORON MIXING | | | |
| 43966 | FARLEY 2 - REEVALUATION OF CONCRETE MASONRY WALLS | | | |
| 43965 | FARLEY 2 - CONTROL OF HEAVY LOADS (GENERIC ISSUE) | | | |
| <u>ANTICIPATED ACTIONS</u> | | | | <u>INITIATION</u> |
| 46492 | FARLEY 2 - DESCRIP OF CONTAINM PRESSURE & WATER LEVEL INSTRUMENTS & HYDROGEN MONITORS | | | 05/09/81 |
| 46493 | FARLEY 2 - DRESCRIP OF CONTAINM PRESSURE & WATER LEVEL INSTRUMENTS & HYDROGEN MONITORS | | | 06/09/81 |
| 46494 | FARLEY 2 - DESCRIP OF CONTAINM PRESSURE & WATER LEVEL INSTRUMENTS & HYDROGEN MONITORS | | | 06/09/81 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: FITZPATRICK

PLANT LOCATION: 8 MI NE OF OSWEGO, NY
 DOCKET NUMBER: 050-00333
 ARCH/ENGINEER: S&W
 IE INSPECTOR: J. LINVILLE

LICENSED POWER: 2436 MWT
 DESIGN POWER: 0821 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: P. POLK
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|---|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10416 | FITZPATRICK - CONTINGENCY PLAN REVIEW | | 1 | <u>COMPLETE</u> 02/28/81 |
| 10419 | FITZPATRICK - GUARD TRAINING PLAN REVIEW | | 1 | 02/28/81 |
| 11075 | FITZPATRICK - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/28/81 |
| 11738 | FITZPATRICK - TMI FOLLOW UP WORK | | 1 | 04/01/81 |
| 12166 | FITZPATRICK - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 07938 | FITZPATRICK - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | 04/17/81 |
| 11355 | FITZPATRICK - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | 04/17/81 |
| 08218 | FITZPATRICK - STRESS CORROSION CRACKING-BWR RCSPB-GENERIC | | 1 | 05/01/81 |
| 10083 | FITZPATRICK - ENHANCED FISSION PRODUCT RELEASE FOR HIGH BURNUP LWR FUEL | | 9 | 05/01/81 |
| 42220 | FITZPATRICK - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | 05/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12303 | FITZPATRICK - HELB AND CONSEQUENTIAL SYSTEM FAILURE FITZPATRICK | | 3 | <u>TARGET</u> |
| 43019 | FITZPATRICK - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 08045 | FITZPATRICK - APPENDIX I TECH SPECS (EVALUATE RESPONSE TO NRC LETTER OF 7/11/78.) | | 3 | |
| 10780 | FITZPATRICK - PUMP AND VALVE OPERABILITY TEST PROGRAM | | 1 | |
| 07789 | FITZPATRICK - SPENT FUEL POOL INCREASE STORAGE FROM 760 TO 2244 FUEL ASSEMBLIES | | 1 | |
| 07990 | FITZPATRICK - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08315 | FITZPATRICK - 10 CFR 50.55A(G) - INSERVICE INSPECTION - GENERIC | | 1 | |
| 08932 | FITZPATRICK - RPS POWER SUPPLY | | 1 | |
| 10361 | FITZPATRICK - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08598 | FITZPATRICK - HYDRAULIC SNUBBERS | | 3 | |
| 10756 | FITZPATRICK - MECHANICAL SNUBBERS | | 2 | |
| 11146 | FITZPATRICK - APPENDIX J CONTAINMENT LEAK TESTING | | 2 | |
| 42741 | FITZPATRICK - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42671 | FITZPATRICK - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43934 | FITZPATRICK - S-TATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43734 | FITZPATRICK - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | 1 | |
| 42881 | FITZPATRICK - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42489 | FITZPATRICK - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u> | | | | |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44020 | FITZPATRICK - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44090 | FITZPATRICK - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44161 | FITZPATRICK - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44232 | FITZPATRICK - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44302 | FITZPATRICK - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44373 | FITZPATRICK - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44441 | FITZPATRICK - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44511 | FITZPATRICK - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44581 | FITZPATRICK - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44779 | FITZPATRICK - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FITZPATRICK

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|---|---|----------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44850 | FITZPATRICK - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44921 | FITZPATRICK - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44992 | FITZPATRICK - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45063 | FITZPATRICK - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45134 | FITZPATRICK - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45343 | FITZPATRICK - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45542 | FITZPATRICK - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45570 | FITZPATRICK - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45594 | FITZPATRICK - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45629 | FITZPATRICK - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45690 | FITZPATRICK - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45714 | FITZPATRICK - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45738 | FITZPATRICK - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45762 | FITZPATRICK - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45786 | FITZPATRICK - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45826 | FITZPATRICK - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45886 | FITZPATRICK - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45910 | FITZPATRICK - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45945 | FITZPATRICK - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46016 | FITZPATRICK - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46088 | FITZPATRICK - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46160 | FITZPATRICK - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46232 | FITZPATRICK - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46304 | FITZPATRICK - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46375 | FITZPATRICK - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46446 | FITZPATRICK - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>PLANT SPECIFIC</u> | | | | |
| TAC # | TAC DESCRIPTION | | PRIORITY | CRITICAL DATE |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42290 | FITZPATRICK - CONTAINMENT LEAK TESTING (MSIV) | | 1 | COMPLETE 04/17/81 |
| 43371 | FITZPATRICK - MISCELLANEOUS TECHNICAL SPECIFICATION CHANGE | | 1 | 04/17/81 |
| 43372 | FITZPATRICK - ADMINISTRATIVE CHANGE | | 1 | 04/17/81 |
| 43567 | FITZPATRICK - MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION PLATE | | 1 | 04/17/81 |
| 07264 | FITZPATRICK - CONTROL ROD SCRAM TIME TESTING | | 1 | 05/01/81 |
| 11456 | FITZPATRICK - DOSE RATE MONITORING | | 1 | 05/01/81 |
| 41090 | FITZPATRICK - DIESEL GENERATOR RELIABILITY | | 3 | 05/01/81 |
| 42289 | FITZPATRICK - SECURITY FORCE HUMAN ERROR | | 1 | 05/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 41069 | FITZPATRICK - DC POWER SUPPLY FAILURE EFFECT ON ECCS | | 1 | TARGET |
| 41091 | FITZPATRICK - TECHNICAL SPECIFICATION OPERABILITY DEFINITION | | 1 | |
| 41092 | FITZPATRICK - CONTROL RODS FAIL TO FULLY INSERT TO FULLY INSERT | | 1 | |
| 4J093 | FITZPATRICK - STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 3 | |
| 41094 | FITZPATRICK - REVIEW OF CORPORATE CAPABILITIES | | 1 | |
| 06953 | FITZPATRICK - CHANGE TO VAC BREAKERS & INSTRUMENTATION | | 1 | |
| 06864 | FITZPATRICK - MSIV LEAK DETECTION SYSTEM | | 1 | |
| 11897 | FITZPATRICK - EMERGENCY S.W. PUMP TESTING | | 1 | |
| 12559 | FITZPATRICK - SPIRAL LOADING | | 1 | |
| 43527 | FITZPATRICK - ISI ADMINISTRATIVE CHANGE | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FITZPATRICK

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | (CONTINUATION) | | | |
| 43530 | FITZPATRICK - FLOW BIASING ROD BLOCK | | 2 | <u>TARGET</u> |
| 43705 | FITZPATRICK - DRYWELL SUMP MONITORING SYSTEM | | 1 | |
| 42576 | FITZPATRICK - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 43851 | FITZPATRICK - SRV SETPOINTS | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: FORT CALHOUN 1

PLANT LOCATION: 19 MI N OF OMAHA, NEB
 DOCKET NUMBER: 050-00285
 ARCH/ENGINEER: GHDR
 IE INSPECTOR: D. KELLEY

LICENSED POWER: 1500 MWT
 DESIGN POWER: 0473 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: C. TRAMMELL
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|--|-------------|----------|-----------------|
| | | | | <u>COMPLETE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 07376 | FT. CALHOUN 1 - CHARCOAL FILTER FLOW RATES | | 3 | 10/14/80 |
| 42371 | FT. CALHOUN 1 - CLARIFICATION OF TERM OPERABLE | | 2 | 10/14/80 |
| 43027 | FT. CALHOUN 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 10/14/80 |
| 11107 | FT. CALHOUN 1 - FIRE PROTECTION SUPPLEMENT | | 1 | 11/17/80 |
| 11912 | FT. CALHOUN 1 - FIRE WATER PIPING FAILURE EFFECT ON SRE ON SRE | | 1 | 11/17/80 |
| 11919 | FT. CALHOUN 1 - FIRE PROTECTION, ALTERNATE SHUTDOWN CAPABILITY | | 1 | 11/17/80 |
| 13095 | FT. CALHOUN 1 - QUALITY ASSURANCE REQUIREMENTS REGARDING DIESEL FUEL OIL | | 3 | 11/19/80 |
| 42931 | FT. CALHOUN 1 - INADVERTENT SAFETY ACTIONS DURING SURVEILLANCE TESTS - B-52 | | 2 | 11/21/80 |
| 42990 | FT. CALHOUN 1 - TMI LL TECH SPEC PER 7/2/80 GUIDANCE | | 1 | 01/19/81 |
| 12363 | FT. CALHOUN 1 - LESSONS LEARNED IMPLEMENTATION | | 1 | 02/05/81 |
| 42110 | FORT CALHOUN - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 02/10/81 |
| 10466 | FT. CALHOUN 1 - SOLUBILITY DURING LONG TERM COOLING FOLLOWING LOCA | | 3 | 03/10/81 |
| 07991 | FT. CALHOUN 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | 03/25/81 |
| 12888 | FT. CALHOUN 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 10622 | FT. CALHOUN 1 - CONTINGENCY PLAN REVIEW | | 1 | 04/28/81 |
| 43292 | FT. CALHOUN 1 - SNUBBER SURVEILLANCE TECH. SPEC. | | 2 | 05/20/81 |
| | | | | <u>TARGET</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 43659 | FORT CALHOUN - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 10618 | FT. CALHOUN 1 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10620 | FT. CALHOUN 1 - VITAL AREA ANALYSIS | | 1 | |
| 42742 | FT. CALHOUN 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42672 | FT. CALHOUN 1 - IE BULLETIN 79-27, LOS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 08919 | FT. CALHOUN 1 - GENERIC RTD RESPONSE TIME | | 3 | |
| 08926 | FT. CALHOUN 1 - REACTOR PROTECTION SYSTEM LOGIC | | 3 | |
| 08143 | FT. CALHOUN 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 42077 | FT. CALHOUN 1 - ANALYSIS OF TURBINE DISCS IN G.E. TURBINES | | 2 | |
| 08922 | FT. CALHOUN 1 - REVIEW OF ASYMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08910 | FT. CALHOUN 1 - LEAK TESTING - APP J (GENERIC) | | 2 | |
| 08925 | FT. CALHOUN 1 - PWR PUMP S/G SUPPORTS-LAMELLAR TEARING FRACTURE TOUGHNESS | | 2 | |
| 12316 | FT. CALHOUN 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE FT. CALHOUN | | 3 | |
| 43929 | FT. CALHOUN - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42892 | FT. CALHOUN - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43135 | FT. CALHOUN 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42586 | FT. CALHOUN 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 43198 | FT. CALHOUN 1 - FIRE PROTECTION - APP. R IMPLEMENTATION | | 1 | |
| 43216 | FT. CALHOUN 1 - EMERGENCY PREPAREDNESS REVIEW | | 1 | |
| 12738 | FT. CALHOUN - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 43319 | FT. CALHOUN - CONTROL OF HEAVY LOADS, NUREG-0612 | | 2 | |
| 42491 | FT. CALHOUN 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| 43435 | FT. CALHOUN - NUREG 0737 REVIEW | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FORT CALHOUN 1

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|----------------------------|--|---|----------|-------------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44021 | FT. CALHOUN 1 - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44162 | FT. CALHOUN 1 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44233 | FT. CALHOUN 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44374 | FT. CALHOUN 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 444-2 | FT. CALHOUN 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 4451 | FT. CALHOUN 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44582 | FT. CALHOUN 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44663 | FT. CALHOUN 1 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44744 | FT. CALHOUN 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44780 | FT. CALHOUN 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44851 | FT. CALHOUN 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44922 | FT. CALHOUN 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44993 | FT. CALHOUN 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45064 | FT. CALHOUN 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45135 | FT. CALHOUN 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45262 | FT. CALHOUN 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45308 | FT. CALHOUN 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45344 | FT. CALHOUN 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45429 | FT. CALHOUN 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45630 | FT. CALHOUN 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45827 | FT. CALHOUN 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45946 | FT. CALHOUN 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46017 | FT. CALHOUN 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46089 | FT. CALHOUN 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46161 | FT. CALHOUN 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46233 | FT. CALHOUN 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46305 | FT. CALHOUN 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46376 | FT. CALHOUN 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46447 | FT. CALHOUN 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| 43433 | FT. CALHOUN 1 - AUX. FEED WATER AUTO INITIATION - SAFETY GRADE | | | 12/31/81 |
| 43434 | FT. CALHOUN 1 - SHIFT STAFFING REQUIREMENTS I.A.1.3 | | | 07/01/82 |
| <u>PLANT SPECIFIC</u> | | | | |
| TAC # | TAC DESCRIPTION | | PRIORITY | CRITICAL DATE |
| <u>COMPLETED ACTIONS</u> | | | | |
| 07020 | FT. CALHOUN 1 - CORE BARREL MOVEMENT MONITORING | | 1 | <u>COMPLETE</u> 10/09/80 |
| 43324 | FT. CALHOUN- INCORE DETECTOR T.S. FOR CYCLE 6 | | | 02/02/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42932 | FT. CALHOUN 1 - COOLANT ACTIVITY TECH SPECS | | 1 | TARGET |
| 41003 | FT. CALHOUN 1 - ADMIN. CHANGE REQUEST OF 3/14/78 | | 2 | |
| 43522 | FT. CALHOUN - INSERVICE TESTING OF VALVES SI-159 AND SI-160 | | 2 | |
| 43741 | FT. CALHOUN 1 - REVISED BORON DILUTION (SHUTDOWN MARGIN) TECH. SPECS. (OPPD 3-30-81) | | 2 | |
| 11447 | FT. CALHOUN 1 - REQUEST TO ELIMINATE CERTAIN ENVIRONMENTAL TECH SPECS | | 2 | |
| <u>ANTICIPATED ACTIONS</u> | | | | |
| 46513 | FT. CALHOUN - ADMIN. CHGS OF 6/2/81 | | | <u>INITIATION</u> 06/02/81 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: FORT ST VRAIN

PLANT LOCATION: 35 MI N OF DENVER, COL
 DOCKET NUMBER: 050-00267
 ARCH/ENGINEER: S&L
 IE INSPECTOR: R. HALL

LICENSED POWER: 0842 MWT
 DESIGN POWER: 0330 MWE
 NSSS VENDOR: GAC

PROJECT MANAGER: G. KUZMYCZ
 BRANCH CHIEF: T. SPEIS
 LIC. ASSISTANT: H. GEARIN

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 11025 | FORT ST VRAIN - CONTROL OF HEAVY LOADS PER SPENT FUEL | | 2 | <u>TARGET</u> |
| 12155 | FORT ST VRAIN - UPGRADED EMERGENCY PLAN | | 1 | |
| 46503 | FT ST VRAIN-ADEQUACY OF STA EL. VOLTAGE | | | |
| 46504 | FT ST VRAIN-DEGRADED GRID VOLTAGE | | | |
| 42743 | FORT ST VRAIN - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43935 | FT. ST. VRAIN - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42925 | FORT ST VRAIN - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42527 | FORT ST VRAIN - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44022 | FORT ST VRAIN - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44091 | FORT ST VRAIN - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44163 | FORT ST VRAIN - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44234 | FORT ST VRAIN - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44304 | FORT ST VRAIN - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44375 | FORT ST VRAIN - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44443 | FORT ST VRAIN - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44513 | FORT ST VRAIN - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44583 | FORT ST VRAIN - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44731 | FORT ST VRAIN - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44852 | FORT ST VRAIN - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44923 | FORT ST VRAIN - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 44994 | FORT ST VRAIN - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45065 | FORT ST VRAIN - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45136 | FORT ST VRAIN - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45345 | FORT ST VRAIN - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45631 | FORT ST VRAIN - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45828 | FORT ST VRAIN - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45947 | FORT ST VRAIN - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46018 | FORT ST VRAIN - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46090 | FORT ST VRAIN - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46162 | FORT ST VRAIN - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46234 | FORT ST VRAIN - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RJE | | | |
| 46306 | FORT ST VRAIN - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46377 | FORT ST VRAIN - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46448 | FORT ST VRAIN - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 01736 | FORT ST VRAIN - ABNORMAL OCCURRENCE RPT. RE ELECTRICAL DESIGN AND INSTALLATION CRITERIA | | | <u>TARGET</u> |
| 03738 | FORT ST VRAIN - REVIEW TECH SPECS | | | |
| 05204 | FORT ST VRAIN - REVIEW OF RESEARCH COPPER DATA | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FORT ST VRAIN

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|-----------------------|------------------------|--|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> | <u>(CONTINUATION)</u> | | | |
| 11577 | FORT ST VRAIN - | GUARD TRAINING | 1 | |
| 01549 | FORT ST VRAIN - | STEAM INGRESS PROBLEM | | |
| 01465 | FORT ST VRAIN - | PROPOSED TECH SPECS FOR INSERVICE INSPECTION REQUIREMENTS FOR HE CIRCULATORS | | |
| 03262 | FORT ST VRAIN - | REVISIONS TO TECH SPECS | | |
| 03680 | FORT ST VRAIN - | ESF FILTER SYSTEM TECH SPECS | | |
| 03658 | FORT ST VRAIN - | REVIEW OF PROPOSED CORRECTIVE ACTION BY PSC OF COL. | 2 | |
| 04483 | FORT ST VRAIN - | AMEND ENVIRONMENTAL TECH SPEC | | |
| 06802 | FORT ST VRAIN - | ENVIRONMENTAL TECHNICAL SPECS | | |
| 04696 | FORT ST VRAIN - | REVIEW FSV ISOLATION VALES BETWEEN SEISMIC CLASS 1 & 2 PIPING SYSTEMS | 2 | |
| 04938 | FORT ST VRAIN - | PROPOSED PERM MOD TO STEAM/WATER DUMP SYSTEM LOGIC CIRCUITRY | | |
| 05295 | FORT ST VRAIN - | DECALIBRATION OF EX-CORE NEUTRON DETECTORS | 2 | |
| 43298 | TS FUEL IN-CO:RE | RESIDENCE TIMES | | |
| 11576 | FORT ST VRAIN - | CONTINGENCY PLAN REVIEW | 1 | |
| 04970 | FORT ST VRAIN - | PRIMARY & SECONDARY SYSTEM POWER/TEMP LLATIONS | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: GINNA

PLANT LOCATION: 15 MI NE OF ROCHESTER, NY
 DOCKET NUMBER: 050-00244
 ARCH/ENGINEER: GIL
 IE INSPECTOR: R. ZIMMERMAN

LICENSED POWER: 1520 MWT
 DESIGN POWER: 0470 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. SNAIDER
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11118 | GINNA - FIRE PROTECTION SER SUPPLEMENT | | 1 | <u>COMPLETE</u> 02/06/81 |
| 12287 | GINNA - HELB AND CONSEQUENTIAL SYSTEM FAILURE GINNA | | 3 | 02/18/81 |
| 08413 | GINNA - PWR PUMP S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | 03/02/81 |
| 11706 | GINNA - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 03/02/81 |
| 42128 | GINNA - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 03/02/81 |
| 11757 | GINNA - FEEDWATER LINES-CRACKS IN NOZZLE TO PIPE WELDS TO PIPE WELDS | | 2 | 03/06/81 |
| 10927 | GINNA - CONTINGENCY PLAN REVIEW | | 1 | 03/11/81 |
| 10024 | GINNA - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | 03/26/81 |
| 10194 | GINNA - VENT & PURG | | 1 | 04/01/81 |
| 10284 | GINNA - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 04/01/81 |
| 11065 | GINNA - APPEND J CONTAINMENT LEAK TEST | | 2 | 04/01/81 |
| 12811 | GINNA - EMERGENCY PLANNING | | 1 | 04/01/81 |
| 12915 | GINNA - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES VALVES | | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12761 | GINNA - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | <u>TARGET</u> |
| 43047 | GINNA - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43662 | GINNA - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 2 | |
| 06409 | GINNA - REDUCE LOSS ON UNDERFREQUENCY SCRAM AND RCP TRIP TO 55HZ. | | 2 | |
| 06792 | GINNA - PUMP AND VALVE TESTING (10 CFR 50.55A.(G)) INSERVICE TESTING | | 2 | |
| 06137 | GINNA - REACTOR VESSEL SUPPORT (NRR GENERIC A-2) | | 2 | |
| 10136 | GINNA - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT | | 2 | |
| 08097 | GINNA - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 07739 | GINNA - ECCS EVAL THAT ALLOWS FOR UPPER PLENUM INJECTION(UPI) | | 3 | |
| 10926 | GINNA - VITAL AREA ANALYSIS | | 3 | |
| 07992 | GINNA - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 1 | |
| 10346 | GINNA - LOSS OF 125-V BUS VOLTAGE WITH LOS OF ANNUNCIATOR SYSTEM | | 2 | |
| 08691 | GINNA - PWR PRESSURE-TEMPERATURE TECH SPECS | | 3 | |
| 10925 | GINNA - GUARD TRAINING PLAN REVIEW | | 1 | |
| 08414 | GINNA - TS SURVEILLANCE - HYDRAULIC SNUBBERS | | 1 | |
| 08416 | GINNA - TS SURVEILLANCE FOR MECHANICAL SNUBBERS TEST RESULTS | | 3 | |
| 12437 | GINNA - LESSONS LEARNED IMPLEMENTATION | | 2 | |
| 12590 | GINNA - ANALYSIS OF TURBINE DISC CRACKS | | 1 | |
| 42744 | GINNA - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 2 | |
| 42673 | GINNA - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER, REV. & EVAL. RESPONSES | | 3 | |
| 43937 | R. E. GINNA 1 - STATION BLACKOUT PROCEDURES & TRAINING | | 2 | |
| 43613 | GINNA - FIRE PROTECTION - 10 CFR 50.48 & APP. R IMPLEMENTATION | | 1 | |
| 42910 | GINNA - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 1 | |
| 43551 | GINNA - DECAY HEAT REMOVAL CAPABILITY TECH. SPECS. | | 2 | |
| 43136 | GINNA - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42520 | GINNA - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| <u>TARGET</u> | | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

GINNA

| TAC # | TAC DESCRIPTION | IMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|--------------------|---|----------|-------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44023 | GINNA - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44092 | GINNA - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44164 | GINNA - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44235 | GINNA - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44305 | GINNA - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44376 | GINNA - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44444 | GINNA - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44514 | GINNA - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44584 | GINNA - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44637 | GINNA - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44679 | GINNA - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44716 | GINNA - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44782 | GINNA - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44853 | GINNA - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44924 | GINNA - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44995 | GINNA - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45066 | GINNA - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45137 | GINNA - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45234 | GINNA - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45280 | GINNA - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45346 | GINNA - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45397 | GINNA - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45446 | GINNA - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45476 | GINNA - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45506 | GINNA - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45632 | GINNA - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45829 | GINNA - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45948 | GINNA - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46019 | GINNA - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46091 | GINNA - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46163 | GINNA - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46235 | GINNA - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46307 | GINNA - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46378 | GINNA - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46449 | GINNA - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|--------------------------|---|----------------|----------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12491 | GINNA - STAEM GENERATOR-SECONDARY SIDE-REQ TO REDUCE HYDRO PRESS (10 YR TEST)/110 INSTEAD OF 125% | | 1 | COMPLETE 12/02/80 |
| 43099 | GINNA - STEAM GENERATOR SLEEVING | | | 12/12/80 |
| 43201 | GINNA - T.S. CHANGE MANAGEMENT ORGANIZATION CHART | | | 02/10/81 |
| 43443 | GINNA - 10 CFR 50.48 EXTENSION REQUEST | | 1 | 02/13/81 |
| 43468 | GINNA - ADDITIONAL FIRE PROTECTION TECH. SPECS. | | 1 | 04/01/81 |
| 43244 | GINNA - T.S. AMENDMENT-CONTROL ROD MISALIGNMENT | | 1 | 04/17/81 |
| 11495 | GINNA - DIESEL GEN TEST CLARIFICATION | | 1 | 04/23/81 |
| 10997 | GINNA - CONTAINMENT LEAK. TEST. (APP J 10 CFR 50) | | 1 | 05/06/81 |
| 43283 | GINNA - LESSONS LEARNED TECH. SPECS | | 1 | 05/11/81 |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

GINNA

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 10960 | GINNA - T.S. CHANGE TO INCLUDE REQUIREMENTS FOR 2 NEW 75 AMPS BATTERY CHARGES | | 1 | <u>TARGET</u> |
| 12461 | GINNA - TENDON SURVEILLANCE PROGRAM | | 1 | |
| 43202 | GINNA - APPROVAL OF SPENT FUEL POOL MODIFICATIONS | | 2 | |
| 43196 | GINNA - REVISED ISI PROGRAM-S/G TUBE SLEEVING | | 1 | |
| 43585 | GINNA - CYCLE 11 RELOAD 50.59 LETTER | | 1 | |
| 43976 | GINNA - SNUBBER TECHNICAL SPECIFICATIONS | | | |
| 11220 | GINNA - CONVERSION POL TO FTOL | | 1 | |
| 42609 | GINNA - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |
| 43570 | GINNA - AFW SEISMIC QUALIFICATION/POST-TMI RELIABILITY EVALUATION | | | |
| 43307 | GINNA - NON-TECH SPEC TMI RELATED ITEMS | | 1 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: HADDAM NECK

PLANT LOCATION: 13 MI E OF MERIDEN, CONN
 DOCKET NUMBER: 050-00213
 ARCH/ENGINEER: S&W
 IE INSPECTOR: T. SMITH

LICENSED POWER: 1825 MWT
 DESIGN POWER: 0580 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. CARUSO
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11748 | HADDAM NECK - TMI FOLLOW UP WORK | | 1 | <u>COMPLETE</u> 11/21/80 |
| 10064 | HADDAM NECK GH BURNUP LWR FUEL | | | 12/01/80 |
| 06968 | HADDAM NECK - INCORPORATE 10 CFR 50.55 A(G) PROVISIONS INTO TECHNICAL SPECIFICATIONS | | 1 | 02/28/81 |
| 07893 | HADDAM NECK - DIESEL GENERATOR LOCKOUT | | 3 | 02/28/81 |
| 12288 | HADDAM NECK - HELB AND CONSEQUENTIAL SYSTEM FAILURE HADDAM NECK | | 3 | 02/28/81 |
| 12916 | HADDAM NECK - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 02/28/81 |
| 12142 | HADDAM NECK | | 1 | 04/01/81 |
| 12438 | HADDAM NECK - LESSONS LEARNED IMPLEMENTATION | | 1 | 04/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 10025 | HADDAM NECK - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 10343 | HADDAM NECK - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10933 | HADDAM NECK - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10934 | HADDAM NECK - VITAL AREA ANALYSIS | | 1 | |
| 10935 | HADDAM NECK - CONTINGENCY PLAN REVIEW | | 1 | |
| 12548 | HADDAM NECK - PERSONNEL HATCH LEAK RATE TESTING | | 2 | |
| 12782 | HADDAM NECK - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 43048 | HADDAM NECK - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43639 | HADDAM NECK - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 06128 | HADDAM NECK - CONTAINMENT LEAK TESTING EXEMPTION REQUEST | | 2 | |
| 06119 | HADDAM NECK - AIR FILTRATION SYSTEM | | 3 | |
| 08382 | HADDAM NECK - GENERIC - PWR MODERATOR DILUTION | | 3 | |
| 08098 | HADDAM NECK - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08859 | HADDAM NECK - CONTAINMENT LEAK TESTING - APPENDIX J (GENERIC) | | 2 | |
| 07993 | HADDAM NECK - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08690 | HADDAM NECK - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | |
| 08470 | HADDAM NECK - REVIEW MATERIAL SURVEILLANCE RESULTS | | 3 | |
| 08464 | HADDAM NECK - PWR PUMP S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 11234 | HADDAM NECK - INSERVICE TESTING (IST) | | 1 | |
| 11113 | HADDAM NECK - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 11976 | HADDAM NECK - REACTOR CAVITY SEAL RING GENERIC ISSUE | | 2 | |
| 08463 | HADDAM NECK - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08465 | HADDAM NECK - TECH SPEC SURVEILLANCE FOR HYD. SNUBBERS | | 3 | |
| 08469 | HADDAM NECK - TECH SPEC SURVEILLANCE FOR MECH. SNUBBERS | | 2 | |
| 08237 | HADDAM NECK - PWR SECONDARY MONITORING REQUIREMENTS GENERIC IC | | 2 | |
| 12597 | HADDAM NECK - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 08857 | HADDAM NECK - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 42745 | HADDAM NECK - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 2 | |
| 42674 | HADDAM NECK - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43625 | HADDAM NECK - APPENDIX R-EXEMPTION REQUEST | | 1 | |
| 43001 | CONNECTICUT YANKEE (HADDAM NECK) STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42911 | HADDAM NECK - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43137 | HADDAM NECK - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42521 | HADDAM NECK - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HADDAM NECK

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|---|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 41066 | HADDAM NECK - POST ACCIDENT SAMPLING SYSTEM LESSONS LEARNED ITEM 2.1.8A | | 1 | COMPLETE 04/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44024 | HADDAM NECK - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44093 | HADDAM NECK - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44165 | HADDAM NECK - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44236 | HADDAM NECK - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44306 | HADDAM NECK - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44377 | HADDAM NECK - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44515 | HADDAM NECK - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44585 | HADDAM NECK - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44638 | HADDAM NECK - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44680 | HADDAM NECK - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44717 | HADDAM NECK - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44783 | HADDAM NECK - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44854 | HADDAM NECK - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44925 | HADDAM NECK - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 44996 | HADDAM NECK - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45067 | HADDAM NECK - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45138 | HADDAM NECK - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45235 | HADDAM NECK - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45261 | HADDAM NECK - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45347 | HADDAM NECK - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45398 | HADDAM NECK - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45447 | HADDAM NECK - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45477 | HADDAM NECK - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45507 | HADDAM NECK - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45633 | HADDAM NECK - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45830 | HADDAM NECK - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45949 | HADDAM NECK - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46020 | HADDAM NECK - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46092 | HADDAM NECK - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46164 | HADDAM NECK - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46236 | HADDAM NECK - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46308 | HADDAM NECK - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46379 | HADDAM NECK - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46450 | HADDAM NECK - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>COMPLETED ACTIONS</u> | | | | |
| 07184 | HADDAM NECK - REVISION #5 OF EMERGENCY PLAN | | 1 | COMPLETE 10/01/80 |
| 42023 | HADDAM NECK - RX PRESSURE VESSEL NOZZLE WELDS; ISI T.S AMENDMENT | | | 10/01/80 |
| 42024 | HADDAM NECK - ON SIT STORAGE OF LOW LEVEL WASTE WASTE | | | 11/25/80 |
| 10932 | HADDAM NECK - ADMINISTRATIVE CONTROLS | | 1 | 11/26/80 |
| 43257 | HADDAM NECK - CHARGE SURVEILLANCE REQUIREMENTS FOR ECCS MOVES | | | 02/28/81 |
| 42992 | HADDAM NECK - DELETE SHAD MONITORING T.S. REQUIT. | | | 03/26/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42309 | HADDAM NECK - ENVIRONMENTAL TECH SPEC AMENDMENT | | 2 | TARGET |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HADDAM NECK

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|---|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 43555 | HADDAM NECK - LOCA MODEL LICENSING | | | <u>TARGET</u> |
| 07159 | HADDAM NECK - RHR PUMP OPERATION DURING REFUELING & BORON DILUTION | | 1 | |
| 43422 | HADDAM NECK - CONTAINMENT PRESSURE LIMIT TECHNICAL SPECIFICATION CHANGE | | 1 | |
| 42610 | HADDAM NECK - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |

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CONDENSED MANAGEMENT REPORT

FACILITY: HATCH 1

PLANT LOCATION: 11 MI N OF BAXLEY, GA
 DOCKET NUMBER: 050-00321
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. ROGERS

LICENSED POWER: 2436 MWT
 DESIGN POWER: 0777 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: M. FAIRTILE
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE | |
|------------------------------|---|-------------|----------|-----------------|----------------------|
| | | | | <u>COMPLETE</u> | |
| 10444 | HATCH 1 - CONTINGENCY PLAN REVIEW | | 1 | 10/01/80 | |
| 42246 | HATCH 1 - L2 CAT A T. SPECS (#79) | | 1 | 10/28/80 | |
| 10452 | HATCH 1 - GUARD TRAINING PLAN REVIEW | | 1 | 04/20/81 | |
| | | | | <u>TARGET</u> | |
| <u>COMPLETED ACTIONS</u> | | | | | |
| 12831 | HATCH 1 - ADEQUACY OF STATION ELECTRICAL DISTRIBUTION SYSTEMS VOLTAGES | | 2 | | |
| 10710 | HATCH 1 - CONTAINMENT LEAK TESTING APP J(GENERIC) | | 2 | | |
| 10026 | HATCH 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED W. H GRID VOLTAGE | | 2 | | |
| 08043 | HATCH 1 - APPENDIX I TECH SPECS | | 3 | | |
| 11260 | HATCH 1 - IST | | 1 | | |
| 10362 | HATCH 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | | |
| 10449 | HATCH 1 - VITAL AREA ANALYSIS | | 1 | | |
| 08019 | HATCH 1 - INSERVICE IMS PROGRAM | | 1 | | |
| 07939 | HATCH 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | | |
| 42219 | HATCH 1 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | | |
| 42675 | HATCH 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE-INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | | |
| 42746 | HATCH 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | | |
| 08310 | HATCH 1 - SURV. OF MECHANICAL SNUBBERS | | 2 | | |
| 43915 | EDWIN I. HATCH UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | | |
| 43733 | HATCH 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | | |
| 42920 | HATCH 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | | |
| 42505 | HATCH 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | | |
| <u>TAC # TAC DESCRIPTION</u> | | | | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | | <u>COMPLETE</u> |
| 42155 | HATCH 1 - HATCH L2 B - RCS VENT | | 1 | 04/12/81 | |
| <u>ACTIVE ACTIONS</u> | | | | | <u>TARGET</u> |
| 44025 | HATCH 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | | |
| 44094 | HATCH 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | | |
| 44166 | HATCH 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | | |
| 44237 | HATCH 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | | |
| 44307 | HATCH 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | | |
| 44445 | HATCH 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | | |
| 44516 | HATCH 1 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | | |
| 44586 | HATCH 1 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | | |
| 44784 | HATCH 1 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | | |
| 44855 | HATCH 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | | |
| 44926 | HATCH 1 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | | |
| 44997 | HATCH 1 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | | |
| 45068 | HATCH 1 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | | |
| 45139 | HATCH 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | | |
| 45348 | HATCH 1 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | | |
| 45543 | HATCH 1 - NUREG-0737 II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HATCH 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|------------------------|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| (CONTINUATION) | | | | |
| 45571 | HATCH 1 - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45595 | HATCH 1 - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45634 | HATCH 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45691 | HATCH 1 - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45715 | HATCH 1 - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45739 | HATCH 1 - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45763 | HATCH 1 - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45787 | HATCH 1 - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45831 | HATCH 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45887 | HATCH 1 - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45911 | HATCH 1 - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45950 | HATCH 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46021 | HATCH 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46093 | HATCH 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46165 | HATCH 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46237 | HATCH 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46309 | HATCH 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46380 | HATCH 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46451 | HATCH 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42457 | HATCH 1 - ORGN CHANGES | | | 10/30/80 |
| 43082 | HATCH 1 - STANDBY SW PUMP (#80) | | | 11/13/80 |
| 43481 | HATCH 1 - FIRE AUDITS | | | 02/27/81 |
| 42393 | HATCH 1 - II.K.3.46 MICHELSON CONCERNS | | | 04/12/81 |
| 43627 | HATCH 1 - DIESEL GENERATOR OPERABILITY TS CHANGE REQUEST | | | 04/18/81 |
| 43822 | HATCH 1 - DELETION OF NON SAFETY RELATED SURV. TECH. SPEC. 4.6.H.5 | | 2 | 05/26/81 |
| 43823 | HATCH 1 - ADDITION OF 2 VALVES & TWO FLANGES TO TECH. SPEC. LIST OF ISOLATION VALVES | | 2 | 05/26/81 |
| 43589 | HATCH 1 - RELOAD FOR CYCLE 5 OPERATION | | 1 | 05/28/81 |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12345 | HATCH 1 - AMENDED EMERGENCY PLAN | | 1 | |
| 13277 | HATCH 1 - DEFN OF OPERABLE | | 1 | |
| 13283 | HATCH 1 - DC POWER SUPPLY/ECCS PERFORMANCE | | | |
| 41019 | HATCH 1 - LP TURBINE CRACKING | | | |
| 42283 | HATCH 1 - REVIEW 7/17/80 TORUS WATER TEMP INCREASE REQUEST | | | |
| 11520 | HATCH 1 - ILRT PROGRAM | | 1 | |
| 07054 | HATCH 1 - OVERHEAD CRANE SPECIFICATIONS | | 1 | |
| 12454 | HATCH 1 - EVALUATION OF IMPACT ON SHORT NOSE STURGEON STURGEON | | 2 | |
| 42437 | HATCH 1 - ORGN CHANGES | | 1 | |
| 43083 | HATCH 1 - RPS POWER SUPPLY SURVEILLANCE | | 2 | |
| 43820 | HATCH 1 - TECH SPEC CHANGE RELATED TO RV BOTTOM HEAD WELD INSP. INTERVAL | | | |
| 43684 | HATCH 1 - ADDS 3 VALVES IN SCRAM DISCH. VOL. SYST. TO T.S LIST OF ISOLATION VALVES | | 2 | |
| 43954 | HATCH 1 - PREPARATION OF ORDERS CONFIRMING SUBMITTAL DATES THRU JUNE 30, 1981 (0757) | | 1 | |
| 42603 | HATCH 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |
| 43754 | HATCH 1 - FIRE PROTECTION PROGRAM IMPLEMENTATION OF 10CFR 50.48 - APP. R | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: HATCH 2

PLANT LOCATION: 11 MI N OF BAXLEY, GA
 DOCKET NUMBER: 050-00366
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. ROGERS

LICENSED POWER: 2436 MWT
 DESIGN POWER: 0784 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: M. FAIRTILE
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10445 | HATCH 2 - CONTINGENCY PLAN REVIEW | | 1 | <u>COMPLETE</u> 10/01/80 |
| 42245 | HATCH 2 - L2 CAT A T. SPECS (#18) | | 1 | 10/28/80 |
| 10177 | HATCH 2 - CONTAINMENT PURGE | | 1 | 11/05/80 |
| 10387 | HATCH 2 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 11/05/80 |
| 11121 | HATCH 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 11/12/80 |
| 10450 | HATCH 2 - GUARD TRAINING PLAN REVIEW | | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12346 | HATCH 2 - AMENDED EMERGENCY PLAN | | 1 | <u>TARGET</u> |
| 12832 | HATCH 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 13278 | HATCH 2 - DEFN OF OPERABLE | | 2 | |
| 08095 | HATCH 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 11262 | HATCH 2 - DEGRADED GRID VOLTAGE | | 2 | |
| 10330 | HATCH 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10448 | HATCH 2 - VITAL AREA ANALYSIS | | 1 | |
| 07940 | HATCH 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 07811 | HATCH 2 - ISI (50.55A) | | 1 | |
| 07814 | HATCH 2 - RPS POWER SUPPLY | | 1 | |
| 11261 | HATCH 2 - IST | | 1 | |
| 42218 | HATCH 2 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 42676 | HATCH 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42747 | HATCH 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 08298 | HATCH 2 - SURV. OF MECHANICAL SNUBBERS | | 2 | |
| 43916 | EDWIN I. HATCH UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43738 | HATCH 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42921 | HATCH 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42505 | HATCH 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42156 | HATCH 2 - L2 B - RCS VENT | | 1 | <u>COMPLETE</u> 04/12/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44026 | HATCH 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44095 | HATCH 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44167 | HATCH 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44238 | HATCH 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44308 | HATCH 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44446 | HATCH 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44517 | HATCH 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44587 | HATCH 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44785 | HATCH 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44856 | HATCH 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44927 | HATCH 2 - NUREG-0737 II.F.1.1, NOLLE GAS MONITOR | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HATCH 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|--------------------------------------|------------------------|---|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44993 | HATCH 2 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45069 | HATCH 2 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45140 | HATCH 2 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45349 | HATCH 2 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45544 | HATCH 2 - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45572 | HATCH 2 - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45596 | HATCH 2 - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45635 | HATCH 2 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45692 | HATCH 2 - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45716 | HATCH 2 - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45740 | HATCH 2 - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45764 | HATCH 2 - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45788 | HATCH 2 - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45832 | HATCH 2 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45888 | HATCH 2 - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45912 | HATCH 2 - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45951 | HATCH 2 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46022 | HATCH 2 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46094 | HATCH 2 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46166 | HATCH 2 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46238 | HATCH 2 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46310 | HATCH 2 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46381 | HATCH 2 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46452 | HATCH 2 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>COMPLETE</u> |
|--------------------------|------------------------|-------------------------------------|-----------------|---|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42458 | HATCH 2 - | ORGN CHANGES | | 10/30/80 |
| 43125 | HATCH 2 - | FSAR REVISION (QA) | | 11/07/80 |
| 43293 | HATCH 2 - | FIRE PROTECTION MODIFICATIONS (#20) | | 01/31/81 |
| 07815 | HATCH 2 - | BOILING TRANSITION (#21) | 1 | 02/10/81 |
| 07816 | HATCH 2 - | MCPR REANALYSIS (#21) | 1 | 02/10/81 |
| 07817 | HATCH 2 - | FUEL PERFORMANCE (#21) | 1 | 02/10/81 |
| 43086 | HATCH 2 - | RELOAD, CYCLE 2 OPERATION (#21) | | 02/10/81 |
| 43482 | HATCH 2 - | FIRE AUDITS | | 02/27/81 |
| 42394 | HATCH 2 - | II.K.3.46 MICHELSON CONCERNS | | 04/12/81 |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|-----------------------|------------------------|---|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 13284 | HATCH 2 - | DC POWER SUPPLY/ECCS PERFORMANCE | | |
| 41020 | HATCH 2 - | LP TURBINE CRACKING | | |
| 42284 | HATCH 2 - | REVIEW 7/17/80 TORUS WATER TEMP INCREASE REQUEST | 1 | |
| 11158 | HATCH 2 - | SETTLEMENT OF CLASS I STRUCTURES | 1 | |
| 12455 | HATCH 2 - | EVALUATION OF IMPACT ON SHORT NOSE STURGEON STURGEON | 1 | |
| 42438 | HATCH 2 - | ORGN CHANGES | 2 | |
| 07813 | HATCH 2 - | SEISMIC QUALIFICATION OF D/G EXCITER | 1 | |
| 43685 | HATCH 2 - | ADDS 3 VALVES IN SCRAM DISCH. VOL. SYST. TO T.S. LIST OF ISOLATION VALVES | 1 | |
| 43955 | HATCH 2 - | PREPARATION OF ORDERS CONFIRMING SUBMITAL DATES THRU JUNE 30, 1981 (0737) | 1 | |
| 43282 | HATCH 2 - | RPS POWER SUPPLY SURVEILLANCE | | |
| 42604 | HATCH 2 - | LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | 1 | |

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TECHNICAL ASSIGNMENT CONTROL SYSTEM

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HATCH 2

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ACTIVE ACTIONS (CONTINUATION)

43755 HATCH 2 - FIRE PROTECTION PROGRAM IMPLEMENTATION OF 10CFR 50.48 - APP. R

1

TARGET

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: INDIAN POINT 2

PLANT LOCATION: 25 MI N OF NEW YORK CITY, NY
 DOCKET NUMBER: 050-00247
 ARCH/ENGINEER: UEC
 IE INSPECTOR: J. STREETER

LICENSED POWER: 2758 MWT
 DESIGN POWER: 0873 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: L. OLSHAN
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 08762 | INDIAN POINT 2 - MECHANICAL SNUBBERS | | 2 | <u>COMPLETE</u> 10/08/80 |
| 11066 | INDIAN POINT 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 10/31/80 |
| 08659 | INDIAN POINT 2 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 10914 | INDIAN POINT 2 - GUARD TRAINING PLAN REVIEW | | 1 | 11/18/80 |
| 06544 | INDIAN POINT 2 - FLOODING OF EQUIPMENT IMPORTANT TO SAFETY | | 2 | 12/18/80 |
| 11608 | INDIAN POINT 2 - REVISED EMEPGENCY PLAN | | 1 | 04/01/81 |
| 43569 | INDIAN POINT 2 - S.G. INSPECTION RESULTS | | | 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08296 | INDIAN POINT 2 - 10 CFR 50.55A(G) - INSERVICE - IST | | 1 | <u>TARGET</u> |
| 08763 | INDIAN POINT 2 - HYDRAULIC SNUBBERS | | 3 | |
| 10327 | INDIAN POINT 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 12992 | INDIAN POINT 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 1 | |
| 42096 | INDIAN POINT 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43006 | INDIAN POINT 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43640 | INDIAN POINT 2 - SEISMIC QUALIFICATION OF : - AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 06603 | INDIAN POINT 2 - REACTOR VESSEL OVERPRESSURE PROTECTION SYSTEM | | 2 | |
| 06901 | INDIAN POINT 2 - ASYMETRIC LOCA LOADS | | 2 | |
| 07105 | INDIAN POINT 2 - INSERVICE INSPECTION PROGRAM | | 1 | |
| 08113 | INDIAN POINT 2 - APPENDIX I TECH SPEC IMPLEMENTATION | | 3 | |
| 08700 | INDIAN POINT 2 - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT | | 2 | |
| 08776 | INDIAN POINT 2 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 08759 | INDIAN POINT 2 - FILTER TECH SPECS | | 3 | |
| 12174 | INDIAN POINT 2 - SPENT FUEL POOL EXPANSION | | 1 | |
| 10920 | INDIAN POINT 2 - VITAL AREA ANALYSIS | | 1 | |
| 07996 | INDIAN POINT 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 12576 | INDIAN POINT 2 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42679 | INDIAN POINT 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42750 | INDIAN POINT 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43903 | INDIAN POINT UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42862 | INDIAN POINT 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43138 | INDIAN POINT 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42464 | INDIAN POINT 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-20) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12836 | INDIAN POINT 2 - CONTROL ROOM HABITABILITY | | 1 | <u>TARGET</u> |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 12834 | INDIAN POINT 2 - S.G. LEVEL OPERATION | | 1 | <u>COMPLETE</u> 12/04/80 |
| 12973 | INDIAN POINT UNIT 2-SEP ENVIRONMENTAL QUALIFICATION | | | 01/23/81 |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

INDIAN POINT 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|---|---|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS (CONTINUATION)</u> | | | | <u>COMPLETE</u> |
| 41046 | INDIAN POINT 2 - LOPAR FUEL | | | 02/09/81 |
| 43420 | INDIAN POINT 2 - REACTOR VESSEL SURVEILLANCE CAPSULE | | 1 | 02/10/81 |
| 42243 | INDIAN POINT 2 - UPDATE ETS | | | 02/27/81 |
| 43568 | INDIAN POINT 2 - RE-ORGANIZATION | | | 03/27/81 |
| 43274 | INDIAN POINT 2 - POTENTIAL UNREVIEWED SAFETY QUESTIONS | | 1 | 04/03/81 |
| 43294 | INDIAN POINT 2 - MANAGEMENT RE-REVIEW | | | 04/10/81 |
| 12927 | INDIAN POINT 2 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | " | 04/22/81 |
| 42189 | INDIAN POINT 2 - 12% PLUGGING OF SG TUBES | | | 04/22/81 |
| 43095 | INDIAN POINT 2 - WATER IN CONTAINMENT | | | 04/22/81 |
| 11530 | INDIAN POINT 2 - AQUATIC ENV MONITORING PROGRAM | | 1 | 04/24/81 |
| 12847 | INDIAN POINT 2 - LER REVIEW | | 1 | 05/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12816 | INDIAN POINT 2 - STANDARD TECH SPECS | | 1 | |
| 12842 | INDIAN POINT 2 - GAS TURBINE OPERATION | | 1 | |
| 12844 | INDIAN POINT 2 - DIFFERENCES BETWEEN UNITS 2 & 3 | | 1 | |
| 12845 | INDIAN POINT 2 - CONTROL ROOM DESIGN | | 1 | |
| 12849 | INDIAN POINT 2 - CONFORMANCE TO CURRENT RULES & REGULATIONS | | 1 | |
| 12851 | INDIAN POINT 2 - FAILURE MODE EFFECTS ANALYSIS | | 1 | |
| 42038 | INDIAN POINT 2 - ATWS SCHEDULE | | | |
| 43442 | INDIAN POINT 2 - GRID STRAP DAMAGE | | | |
| 43963 | INDIAN POINT 2 - SAFE SHUTDOWN CAPABILITY | | | |
| 06752 | INDIAN POINT 2 - UNDERFREQUENCY TRIP OF REACTOR COOLANT | | 1 | |
| 41042 | INDIAN POINT 2 - RESIDUAL RISKS | | | |
| 42557 | INDIAN POINT 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: INDIAN POINT 3

PLANT LOCATION: 25 MI N OF NEW YORK CITY, NY
 DOCKET NUMBER: 050-00286
 ARCH/ENGINEER: UEC
 IE INSPECTOR: J. STREETER

LICENSED POWER: 3025 MWT
 DESIGN POWER: 0965 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: L. OLSHAN
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 08764 | INDIAN POINT 3 - MECHANICAL SNUBBERS | | 2 | <u>COMPLETE</u> 10/08/80 |
| 08660 | INDIAN POINT 3 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 12268 | INDIAN POINT 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE INDIAN POINT 3 | | 3 | 02/24/81 |
| 11609 | INDIAN POINT 3 - REVISED EMERGENCY PLAN | | 1 | 04/01/81 |
| 11690 | INDIAN POINT 3 - THREE MILE ISLAND FOLLOWUP WORK | | 1 | 05/06/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08765 | INDIAN POINT 3 - HYDRAULIC SNUBBERS | | 3 | <u>TARGET</u> |
| 10328 | INDIAN POINT 3 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10915 | INDIAN POINT 3 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 12944 | INDIAN POINT 3 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | |
| 12993 | INDIAN POINT 3 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 42097 | INDIAN POINT 3 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43641 | INDIAN POINT 3 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 08777 | INDIAN POINT 3 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 08331 | INDIAN POINT 3 - 10 CFR 50.55A(G) - INSERVICE INSPECTION - GENERIC (ISI) | | 1 | |
| 08773 | INDIAN POINT 3 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08337 | INDIAN POINT 3 - REACTOR VESSEL OVERPRESSURE PROTECTION | | 2 | |
| 10029 | INDIAN POINT 3 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 10921 | INDIAN POINT 3 - VITAL AREA ANALYSIS | | 1 | |
| 11209 | INDIAN POINT 3 - IST PUMPS AND VALVES | | 1 | |
| 07997 | INDIAN POINT 3 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42680 | INDIAN POINT 3 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42751 | INDIAN POINT 3 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43904 | INDIAN POINT UNIT 3 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42863 | INDIAN POINT 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43139 | INDIAN POINT 3 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42474 | INDIAN POINT 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12837 | INDIAN POINT 3 - CONTROL ROOM HABITABILITY | | 1 | <u>TARGET</u> |
| 44028 | INDIAN POINT 3 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44097 | INDIAN POINT 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44169 | INDIAN POINT 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRQ TRAINING | | | |
| 44240 | INDIAN POINT 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44310 | INDIAN POINT 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44379 | INDIAN POINT 3 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44448 | INDIAN POINT 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44519 | INDIAN POINT 3 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44589 | INDIAN POINT 3 - NUREG-0737 II.D.1.1, RELIEF AND SAFETY VALVE TESTING | | | |
| 44640 | INDIAN POINT 3 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44682 | INDIAN POINT 3 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

INDIAN POINT 3

| TAC # | TAC DESCRIPTION | IMI ACTIONS | PRIORITY | CRITICAL DATE |
|--------------------------|--|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | (CONTINUATION) | | <u>TARGET</u> |
| 44719 | INDIAN POINT 3 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44787 | INDIAN POINT 3 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44858 | INDIAN POINT 3 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44929 | INDIAN POINT 3 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45000 | INDIAN POINT 3 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45071 | INDIAN POINT 3 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45142 | INDIAN POINT 3 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45237 | INDIAN POINT 3 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45283 | INDIAN POINT 3 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45351 | INDIAN POINT 3 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45400 | INDIAN POINT 3 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45449 | INDIAN POINT 3 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45479 | INDIAN POINT 3 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45509 | INDIAN POINT 3 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45637 | INDIAN POINT 3 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45834 | INDIAN POINT 3 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45953 | INDIAN POINT 3 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46024 | INDIAN POINT 3 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46096 | INDIAN POINT 3 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46168 | INDIAN POINT 3 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46240 | INDIAN POINT 3 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46312 | INDIAN POINT 3 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46383 | INDIAN POINT 3 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| <u>TAC #</u> | | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 12835 | INDIAN POINT 3 - S.G. LEVEL OPERATION | | | <u>COMPLETE</u> |
| 12974 | INDIAN POINT UNIT 3-SEP ENVIRONMENTAL QUALIFICATION | | 1 | 12/04/80 |
| 42244 | INDIAN POINT 3 - UPDATE ETS | | | 01/23/81 |
| 11531 | INDIAN POINT 3 - AQUATIC ENVIRONMENTAL MONITORING PROGRAM | | | 02/27/81 |
| 12848 | INDIAN POINT 3 - LER REVIEW | | 1 | 04/24/81 |
| | | | 1 | 05/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12817 | INDIAN POINT 3 - STANDARD TECH SPECS | | 1 | |
| 12846 | INDIAN POINT 3 - CONTROL ROOM DESIGN | | 1 | |
| 12850 | INDIAN POINT 3 - CONFORMANCE TO CURRENT RULES & REGULATIONS | | 1 | |
| 12852 | INDIAN POINT 3 - FAILURE MODE EFFECTS ANALYSIS | | 1 | |
| 42039 | INDIAN POINT 3 - ATWS SCHEDULE | | 1 | |
| 41043 | INDIAN POINT 3 - RESIDUAL RISKS | | | |
| 12577 | INDIAN POINT 3 - ANALYSIS OF TURBINE DISC CRACKS | | | |
| 42558 | INDIAN POINT 3 - LONG TERM CONTAINMENT PURGE AND VENT (B-24) | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: KEWAUNEE

PLANT LOCATION: TOWN OF CARLTON, KEWAUNEE, CO.
 DOCKET NUMBER: 050-00305
 ARCH/ENGINEER: PSE
 IE INSPECTOR: N. CHOULES

LICENSED POWER: 1650 MWT
 DESIGN POWER: 0535 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. LICCIARDO
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 08661 | KEWAUNEE - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 08523 | KEWAUNEE - CONTROL ROD GUIDE TUBE WEAR | | 3 | 01/29/81 |
| 08524 | KEWAUNEE - TECH SPEC SURVEILLANCE REQ. FOR MECH. SNUBBERS | | 2 | 03/13/81 |
| 12928 | KEWAUNEE - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 11871 | KEWAUNEE - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 04/23/81 |
| 10482 | KEWAUNEE - CONTINGENCY PLAN REVIEW | | 1 | 05/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 11691 | KEWAUNEE - THREE MILE ISLAND FOLLOW UP WORK | | 1 | |
| 12269 | KEWAUNEE - HELB AND CONSEQUENTIAL SYSTEM FAILURE KEWAUNEE | | 3 | |
| 12994 | KEWAUNEE - ADEQUACY OF STATION ELECTRICAL DISTRBN SYST VOLTAGES | | 1 | |
| 42098 | KEWAUNEE - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43008 | KEWAUNEE - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43671 | KEWAUNEE - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 06886 | KEWAUNEE - REACTOR VESSEL OVERPRESSURE PROTECTION | | 2 | |
| 06510 | KEWAUNEE - AMEND TECH SPECS TO CONFORM TO APPENDIX J WITH CERTAIN EXEMPTIONS | | 2 | |
| 07998 | KEWAUNEE - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08350 | KEWAUNEE - WESTINGHOUSE UPPER PLENUM ECCS INJECTION - LONG RANGE | | 3 | |
| 06781 | KEWAUNEE - INSERVICE INSPECTION - 10 CFR 50.55A (ISI) | | 1 | |
| 08559 | KEWAUNEE - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10329 | KEWAUNEE - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08145 | KEWAUNEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08525 | KEWAUNEE - TECH SPEC SURVEILLANCE OF HYDRAULIC SNUBBERS | | 3 | |
| 10754 | KEWAUNEE - ASYMMETRIC LOCA LOADS | | 2 | |
| 11295 | KEWAUNEE - INSERVICE TESTING 10 CFR 50.55A (IST) | | 1 | |
| 08239 | KEWAUNEE - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | |
| 12609 | KEWAUNEE - PWR CONTROL ROD MISALIGNMENT - T.S ADEQUACY | | 2 | |
| 12593 | KEWAUNEE - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 10030 | KEWAUNEE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 42681 | KEWAUNEE - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42752 | KEWAUNEE - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 07011 | KEWAUNEE - RADIATION PROTECTION MANAGER | | | |
| 07896 | KEWAUNEE - DIESEL GENERATOR LOCKOUT | | 3 | |
| 10485 | KEWAUNEE - GUARD TRAINING PLAN REVIEW | | 1 | |
| 43946 | KEWAUNEE - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43766 | KEWAUNEE - ITEM III G.3; APPENDIX R. DED. SHUTDOWN CAPABILITY | | 1 | |
| 42864 | KEWAUNEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 10479 | KEWAUNEE - VITAL AREA ANALYSIS | | 1 | |
| 43140 | KEWAUNEE - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 43765 | KEWAUNEE - ITEM III J., APP. R. EMERGENCY LIGHTING | | 1 | |
| 42465 | KEWAUNEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

KEWAUNEE

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|------------------------|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44029 | KEWAUNEE - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | <u>TARGET</u> |
| 44098 | KEWAUNEE - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44170 | KEWAUNEE - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44241 | KEWAUNEE - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44311 | KEWAUNEE - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44380 | KEWAUNEE - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44449 | KEWAUNEE - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44520 | KEWAUNEE - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44590 | KEWAUNEE - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44641 | KEWAUNEE - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44683 | KEWAUNEE - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44720 | KEWAUNEE - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44788 | KEWAUNEE - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44859 | KEWAUNEE - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44930 | KEWAUNEE - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45001 | KEWAUNEE - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45072 | KEWAUNEE - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45143 | KEWAUNEE - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45238 | KEWAUNEE - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45284 | KEWAUNEE - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45352 | KEWAUNEE - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45401 | KEWAUNEE - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45450 | KEWAUNEE - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45480 | KEWAUNEE - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45540 | KEWAUNEE - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45638 | KEWAUNEE - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45835 | KEWAUNEE - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45954 | KEWAUNEE - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46025 | KEWAUNEE - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46097 | KEWAUNEE - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46169 | KEWAUNEE - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46241 | KEWAUNEE - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46313 | KEWAUNEE - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46384 | KEWAUNEE - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46453 | KEWAUNEE - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 43386 | KEWAUNEE - | STOP & GOVERNOR VALVE TESTING - T.S. AMENDMENT | | <u>COMPLETE</u> |
| 13157 | KEWAUNEE - | REVISION OF EMERGENCY PLAN TO PROP - OSED APPENDIX E | | 03/09/81 |
| 43288 | KEWAUNEE - | T.S AMENDMENT - BORIC ACID TRANSFER PUMPS | 1 | 04/01/81 |
| 12779 | KEWAUNEE - | TESTING OF DIESEL GENERATORS & HEPA/CHARCOAL FILTERS | 1 | 04/08/81 |
| 42379 | KEWAUNEE - | REVIEW OF CYCLE 6 START UP REPORT | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42378 | KEWAUNEE - | AMENDMENT TO PHYSICAL SECURITY PLAN | | <u>TARGET</u> |
| 43553 | KEWAUNEE - | TS AMENDMENTS FOR FIRE PROT'N SYSTEM | 1 | |
| 43194 | KEWAUNEE - | MAIN STEAM LINE BREAK WITH CONT. F.W. ADDN | | |
| 43552 | KEWAUNEE - | REMOVAL OF RCP FOAM SUPPRESSION SYSTEM | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

KEWAUNEE

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| | (CONTINUATION) | | | |
| 43772 | KEWAUNEE - NUCL FLUX TRIP SETPOINT, PRESS HEATERS, COND. STORAGE TANK | | | |
| 43773 | KEWAUNEE - ENV. T.S. DELETE APP. B/AMEND APP.A. | | | |
| 43096 | KEWAUNEE - REORGANIZATION OF NUCLEAR DEPT | | | |
| 42993 | KEWAUNEE - BVS FAULT AND LOSS OF INSTR. BUS INVERTERS | | 1 | |
| 42559 | KEWAUNEE - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: LA CROSSE

PLANT LOCATION: 19 MI S OF LACROSSE, WISC
 DOCKET NUMBER: 050-00409
 ARCH/ENGINEER: S&L
 IE INSPECTOR: K. RIDGEWAY

LICENSED POWER: 0165 MWT
 DESIGN POWER: 0050 MWE
 NSSS VENDOR: AC

PROJECT MANAGER: R. CARUSO
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10888 | LACROSSE - OFF GAS EXPLOSION PREVENTION REVIEW | | | <u>COMPLETE</u> 10/01/80 |
| 06787 | LACBWR - REVISED EMERGENCY PLAN (50-409) | | 1 | 10/07/80 |
| 11747 | LACROSSE - TMI FOLLOW UP WORK | | 1 | 10/14/80 |
| 10285 | LACROSSE - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 10/25/80 |
| 11136 | LACROSSE - VENTING AND PURGING CONTAINMENT AT FULL POWE | | 1 | 12/12/80 |
| 12289 | LACROSSE - HELB AND CONSEQUENTIAL SYSTEM FAILURE LA CROSSE | | 3 | 02/19/81 |
| 12917 | LACROSSE - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 02/25/81 |

ACTIVE ACTIONS

| | | | | |
|-------|--|--|---|---------------|
| 12172 | LACROSSE EMERGENCY PLAN REVIEW | | 1 | <u>TARGET</u> |
| 12763 | LACBWR ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 1 | |
| 43049 | LACROSSE - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 08099 | LACROSSE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 10928 | LACROSSE - CONTINGENCY PLAN REVIEW | | 1 | |
| 07999 | LACROSSE - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08196 | LACROSSE - BWR SAFETY/RELIEF VALVE SURVEILLANCE | | | |
| 08332 | LACROSSE - 10 CFR 50.55(G)-INSERVICE INSPECTION GENERIC GENERIC | | 1 | |
| 11235 | LACROSSE - INSERVICE TESTING (IST) | | 1 | |
| 10031 | LACROSSE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 07897 | LACROSSE - DIESEL GENERATOR LOCKOUT | | 3 | |
| 07957 | LACROSSE - NON JET PUMP SPRAY PERFORMANCE | | 3 | |
| 08466 | LACROSSE - TECH SPEC SURVEILLANCE FOR HYD. SNUBBERS | | 3 | |
| 08468 | LACROSSE - TECH SPEC SURVEILLANCE FOR MECH SNUBBERS | | 2 | |
| 10929 | LACROSSE - GUARD TRAINING PLAN REVIEW | | 1 | |
| 11100 | LACROSSE - FIRE PROTECTION FOR SER SUPPLEMENT | | 1 | |
| 12449 | LACROSSE - LESSONS LEARNED IMPLEMENTATION | | 1 | |
| 10930 | LACROSSE - VITAL AREA ANALYSIS | | 1 | |
| 08467 | LACROSSE - RESPIRATORY PROTECTION PROGRAM | | 2 | |
| 08858 | LACROSSE - CONTAINMENT LEAK TESTING - APPENDIX J (GENERIC) | | 2 | |
| 42682 | LACROSSE - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42753 | LACROSSE - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43907 | LACROSSE - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43716 | LACROSSE - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42912 | LACROSSE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 10347 | LACROSSE - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 42522 | LACROSSE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

ANTICIPATED ACTIONS

43981 LACROSSE - IEB 79-08 CLOSEOUT

INITIATION
06/04/81

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44030 | LACROSSE - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44099 | LACROSSE - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

LA CROSSE

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | <u>(CONTINUATION)</u> | | <u>TARGET</u> |
| 44171 | LACROSSE - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44242 | LACROSSE - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44312 | LACROSSE - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44381 | LACROSSE - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44450 | LACROSSE - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44521 | LACROSSE - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44591 | LACROSSE - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44789 | LACROSSE - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44860 | LACROSSE - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44931 | LACROSSE - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45002 | LACROSSE - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45073 | LACROSSE - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45144 | LACROSSE - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45353 | LACROSSE - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45545 | LACROSSE - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45573 | LACROSSE - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45597 | LACROSSE - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45639 | LACROSSE - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45693 | LACROSSE - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45717 | LACROSSE - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45741 | LACROSSE - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45765 | LACROSSE - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45789 | LACROSSE - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45836 | LACROSSE - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45889 | LACROSSE - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45913 | LACROSSE - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45955 | LACROSSE - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46026 | LACROSSE - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46098 | LACROSSE - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46170 | LACROSSE - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46242 | LACROSSE - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46314 | LACROSSE - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46385 | LACROSSE - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46454 | LACROSSE - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 07405 | LACROSSE - DELETE REQUIREMENT FOR RESPIRATORY PROTECTION PGM | | 1 | 10/28/80 |
| 11623 | LACROSSE - LACBWR-ROD DROP ANALYSIS | | 1 | 12/05/80 |
| 42818 | LACBWR - CONVERSION TO STS ADM SECTION AND TS ORGANIZATIONAL CHANGES | | 3 | 02/04/81 |
| 12372 | LACBWR LIQUEFACTION POTENTIAL | | 1 | 02/25/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 41051 | LACBWR-SUMMARY DISPOSITION AFFADAVITS NO. 8 | | 1 | |
| 41052 | LACBWR-SUMMARY DISPOSITION AFFADAVITS NO 9 | | 1 | |
| 43055 | LACBWR QUALITY ASSURANCE PROGRAM | | | |
| 11233 | LACROSSE - FTL CONVERSION | | 1 | |
| 42611 | LACROSSE - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |
| 43758 | LACROSSE - ADEQUACY OF SAFETY MARGINS FOR TURBINE BY PASS OPERATION AT 100% POWER | | 2 | |

R-1208620-001

DATA AS OF - 05/31/81

(CONTINUATION)

IAC # IAC DESCRIPTION

ANTICIPATED ACTIONS

43980 LACROSSE - APPEND-

REVIEW

TECHNICAL ASSIGNMENT CONTROL SYSTEM
CONDENSED MANAGEMENT REPORT

LA CROSSE

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

INITIATION

06/04/81

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: MAINE YANKEE

PLANT LOCATION: 10 MI N OF BATH, ME
 DOCKET NUMBER: 050-00309
 ARCH/ENGINEER: S&W
 IE INSPECTOR: W. LAZARUS

LICENSED POWER: 2630 MWT
 DESIGN POWER: 0825 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: G. REQUA
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10215 | MAINE YANKEE - CONTAINMENT PURGE | | 1 | <u>COMPLETE</u> 10/22/80 |
| 08695 | MAINE YANKEE - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 43028 | MAINE YANKEE - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 01/21/81 |
| 06741 | MAINE YANKEE - RCS OVERPRESSURIZATION PROTECTION CTION | | 2 | 02/27/81 |
| 08240 | MAINE YANKEE - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | 02/27/81 |
| 42111 | MAINE YANKEE - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 03/05/81 |
| 08905 | MAINE YANKEE - MECHANICAL SNUBBERS | | 2 | 03/06/81 |
| 12744 | MAINE YANKEE - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGE | | 2 | 03/26/81 |
| 12876 | MAINE YANKEE - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/23/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 07517 | MAINE YANKEE - CONTROL ROD GUIDE TUBE WEAR | | 3 | <u>TARGET</u> |
| 08902 | MAINE YANKEE - RPS CHANNEL INOPERBILITY | | 3 | |
| 12140 | MAINE YANKEE - EMERGENCY PLAN REVIEW | | 1 | |
| 12317 | MAINE YANKEE - HELB AND CONSEQUENTIAL SYSTEM FAILURE MAINE YANKEE | | 3 | |
| 43328 | MAINE YANKEE - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANE | | | |
| 43653 | MAINE YANKEE - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 08897 | MAINE YANKEE - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08903 | MAINE YANKEE - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10375 | MAINE YANKEE - LOSS OF 125-V DC BUS VOLTAGE W/LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 07752 | MAINE YANKEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08906 | MAINE YANKEE - IN REACTOR GROWTH OF CE POISON RODS | | 3 | |
| 10032 | MAINE YANKEE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 08323 | MAINE YANKEE - 10 CFR 50.55(G) INSERVICE INSPECTION NG GENERIC | | 1 | |
| 08900 | MAINE YANKEE - CEA POSITION INDICATTOR CHANNELS | | 3 | |
| 08901 | MAINE YANKEE - CE GENERIC RTD RESPONSE TIME | | 3 | |
| 12364 | MAINE YANKEE - LESONS LEARNED IMPLEMENTATION | | 1 | |
| 42490 | MAINE YANKEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| 08898 | MAINE YANKEE - REVIEW OF ASYMMCTRIC LOCA LOADS SUBMITTAL | | 2 | |
| 10858 | MAINE YANKEE - 1ST PROGRAM | | 1 | |
| 08904 | MAINE YANKEE - HYDRAULIC SNUBBERS | | 3 | |
| 42683 | MAINE YANKEE - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 42754 | MAINE YANKEE - ESF RESET CONTROL DES'GN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 10732 | MAINE YANKEE - VITAL AREA ANALYSIS | | 1 | |
| 12614 | MAINE YANKEE - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 10730 | MAINE YANKEE - CONTINGENCY PLAN REVIEW | | 1 | |
| 08386 | MAINE YANKEE - GENERIC - PWR MODERATOR DILUTION | | 3 | |
| 08909 | MAINE YANKEE - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT | | 2 | |
| 10734 | MAINE YANKEE - GUARD TRAINING PLAN REVIEW | | 1 | |
| 06580 | MAINE YANKEE - FLOOD OF EQUIP IMPORTANT TO SAFETY | | 2 | |
| 42893 | MAINE YANKEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43141 | MAINE YANKEE - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MAINE YANKEE

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
|-----------------------|---|-------------|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 43092 | MAINE YANKEE - SAFETY GRADE AUTOMATIC INITIATION OF AFWS AFWS | | 1 | <u>TARGET</u> |
| 44031 | MAINE YANKEE - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44100 | MAINE YANKEE - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44172 | MAINE YANKEE - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44243 | MAINE YANKEE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44313 | MAINE YANKEE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44382 | MAINE YANKEE - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44451 | MAINE YANKEE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44522 | MAINE YANKEE - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44592 | MAINE YANKEE - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44664 | MAINE YANKEE - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44745 | MAINE YANKEE - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44790 | MAINE YANKEE - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44861 | MAINE YANKEE - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44932 | MAINE YANKEE - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45003 | MAINE YANKEE - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45074 | MAINE YANKEE - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45145 | MAINE YANKEE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45263 | MAINE YANKEE - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45309 | MAINE YANKEE - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45354 | MAINE YANKEE - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45430 | MAINE YANKEE - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45640 | MAINE YANKEE - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45837 | MAINE YANKEE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45956 | MAINE YANKEE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46027 | MAINE YANKEE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46099 | MAINE YANKEE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46171 | MAINE YANKEE - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46243 | MAINE YANKEE - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46315 | MAINE YANKEE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46386 | MAINE YANKEE - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46455 | MAINE YANKEE - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|--------------------------|---|----------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10759 | MAINE YANKEE - ADMINISTRATIVE CONTROLS CHANGE | | 1 | <u>COMPLETE</u> |
| 06895 | MAINE YANKEE - S. G: TUBE LEAKAGE ALLOWED | | 1 | 12/01/80 |
| 42445 | MAINE YANKEE - HYDROGEN MONITORING-NUREG 0578 | | 1 | 02/27/81 |
| 07638 | MAINE YANKEE - ECCS MODEL CHANGE | | 1 | 05/04/81 |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|-----------------------|---|----------------|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 43208 | MAINE YANKEE - OPERATIONAL QUALITY ASSURANCE PROGRAM | | 2 | <u>TARGET</u> |
| 13572 | MAINE YANKEE - CONTAINMENT PRESSURE MODEL | | 1 | |
| 12225 | MAINE YANKEE - MODIFIED SPENT FUEL PIN STORAGE | | 1 | |
| 42025 | MAINE YANKEE - CRUD BUILDUP IN CORE | | 1 | |
| 43969 | MAINE YANKEE - IMPROVED POST ACCIDENT SAMPLING CAPABILITY NUREG-0678 IT 218.A | | 1 | |
| 42052 | MAINE YANKEE - RV FLUENCE CALCS. AND PRESSURE TEMPERATURE LIMITS | | 2 | |
| 43490 | MAINE YANKEE - CYCLE 6 RELOAD | | 1 | |
| 42587 | MAINE YANKEE - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: MCGUIRE 1

PLANT LOCATION: 17 MI N OF CHARLOTTE, NC
 DOCKET NUMBER: 050-00369
 ARCH/ENGINEER: DUKE
 IE INSPECTOR: W. COTTLE

LICENSED POWER: 000 MWT
 DESIGN POWER: 1180 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. BIRKEL
 BRANCH CHIEF: E. ADENSAM
 LIC. ASSISTANT: .

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 12165 | MCGUIRE 1, EMERGENCY PLAN REVIEW | | 1 | <u>TARGET</u> |
| 43951 | MCGUIRE 1 - STATION BLACKOUT PROCEDURES & TRAINING | | 1 | |
| 42538 | MCGUIRE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|------------------------|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44032 | MCGUIRE - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | <u>TARGET</u> |
| 44102 | MCGUIRE - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44173 | MCGUIRE - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44244 | MCGUIRE - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44314 | MCGUIRE - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44383 | MCGUIRE - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44452 | MCGUIRE - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44523 | MCGUIRE - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44593 | MCGUIRE - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44642 | MCGUIRE - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44684 | MCGUIRE - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44721 | MCGUIRE - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44791 | MCGUIRE - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44862 | MCGUIRE - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44933 | MCGUIRE - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45004 | MCGUIRE - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45075 | MCGUIRE - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45146 | MCGUIRE - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45239 | MCGUIRE - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45285 | MCGUIRE - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45355 | MCGUIRE - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45402 | MCGUIRE - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45451 | MCGUIRE - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45481 | MCGUIRE - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45511 | MCGUIRE - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45641 | MCGUIRE - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45838 | MCGUIRE - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45957 | MCGUIRE - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46028 | MCGUIRE - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46100 | MCGUIRE - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46172 | MCGUIRE - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46244 | MCGUIRE - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46316 | MCGUIRE - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46387 | MCGUIRE - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46456 | MCGUIRE - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

TECHNICAL ASSIGNMENT CONTROL SYSTEM
CONDENSED MANAGEMENT REPORT

R-1200620-001

DATA AS OF - 05/31/81

(CONTINUATION)

MCGUIRE 1

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

| | | |
|-------|--|--------|
| TAC # | TAC DESCRIPTION | TARGET |
| 03722 | ACTIVE ACTIONS MCGUIRE 1 & 2--CONSTRUCTION DEFICIENCY REPORT CD-396-370-76/2 WIRE FAILURES IN CRDM MISSILE SHIELD HOLDDOWN TENDONS | |
| 06648 | MCGUIRE UNIT 1 - TECH SPEC PREPARATION | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: MILLSTONE 1

PLANT LOCATION: 5 MI SW OF NEW LONDON, CONN
 DOCKET NUMBER: 050-00245
 ARCH/ENGINEER: EBASCO
 IE INSPECTOR: T. SHEDLOSKY

LICENSED POWER: 2011 MWT
 DESIGN POWER: 0660 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: J. SHEA
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 10949 | MILLSTONE 1 - BWR OFF GAS SYSTEMS H2 IGNITIONS | | | 10/01/80 |
| 07088 | MILLSTONE 1 - SITE EMERGENCY PLAN | | 1 | 10/07/80 |
| 10192 | MILLSTONE 1 - CONTAINMENT PURGE | | 1 | 10/25/80 |
| 10286 | MILLSTONE 1 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 10/25/80 |
| 08412 | MILLSTONE 1 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | 11/17/80 |
| 42210 | MILLSTONE 1 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | 01/09/81 |
| 11786 | MILLSTONE 1 - FOLLOW UP ON BULLETINS 79-07,04 AND 02 ADL PIPE | | 3 | 01/23/81 |
| 08221 | MILLSTONE 1 - STRESS CORROSION CRACKING - BWR RCSPB - GENERIC | | 1 | 01/30/81 |
| 12290 | MILLSTONE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE MILLSTONE 1 | | 3 | 02/19/81 |
| 12918 | MILLSTONE 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | | 02/25/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 11135 | MILLSTONE 1 - BWR POST LOCA H2 CONTROL | | 3 | |
| 11705 | MILLSTONE 1 - THREE MILE ISLAND FOLLOWUP | | 1 | |
| 12137 | MILLSTONE 1 - EMERGENCY PLAN REVIEW | | 1 | |
| 12762 | MILLSTONE 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 43050 | MILLSTONE 1 - TECHNICAL SPECIFICATIONS FOR OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 07426 | MILLSTONE 1 - CONTAINMENT LEAK RATE CONTINUOUS MONITORING | | 2 | |
| 10033 | MILLSTONE 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 10704 | MILLSTONE 1 - CONTROL ROD FAILURE TO INSERT | | 3 | |
| 11210 | MILLSTONE 1 - INSERVICE TESTING (IST) | | 1 | |
| 10412 | MILLSTONE 1 - CONTINGENCY PLAN REVIEW | | 1 | |
| 10137 | MILLSTONE 1 - CONTAINMENT LEAK TESTING - APPENDIX J (GENERIC) | | 2 | |
| 08060 | MILLSTONE 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 10910 | MILLSTONE 1 - RPS POWER SUPPLY | | 1 | |
| 10348 | MILLSTONE 1 - LOSS OF 125-V BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 11116 | MILLSTONE 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 10410 | MILLSTONE 1 - VITAL AREA ANALYSIS | | 1 | |
| 10408 | MILLSTONE 1 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 08415 | MILLSTONE 1 - TS SURVEILLANCE FOR MECHANICAL SNUBBERS | | 2 | |
| 08417 | MILLSTONE 1 - TS SURVEILLANCE HYDRAULIC SNUBBERS | | 3 | |
| 07941 | MILLSTONE 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 08115 | MILLSTONE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 12421 | MILLSTONE 1 - LESSONS LEARNED IMPLEMENTATION | | 1 | |
| 42684 | MILLSTONE 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42755 | MILLSTONE 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43628 | LACBWR - FIRE PROT. ITEM 3.1.4(2) FIXED SUPPRESSION EXEMPTION 3.1.27/3.2.2/3.2.3/3.2.8/ COMBINED WATER SYSTEM | | 1 | |
| 43925 | MILLSTONE UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43721 | MILLSTONE 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42523 | MILLSTONE 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MILLSTONE 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|---|-----------------|----------------------|
| | | | | <u>TARGET</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44033 | MILLSTONE 1 - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44103 | MILLSTONE 1 - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44174 | MILLSTONE 1 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SPO TRAINING | | |
| 44245 | MILLSTONE 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44315 | MILLSTONE 1 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44384 | MILLSTONE 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44453 | MILLSTONE 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44524 | MILLSTONE 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44594 | MILLSTONE 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44792 | MILLSTONE 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44863 | MILLSTONE 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44934 | MILLSTONE 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45005 | MILLSTONE 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45076 | MILLSTONE 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45147 | MILLSTONE 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45356 | MILLSTONE 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45642 | MILLSTONE 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45839 | MILLSTONE 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45958 | MILLSTONE 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46029 | MILLSTONE 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46101 | MILLSTONE 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46173 | MILLSTONE 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46245 | MILLSTONE 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46317 | MILLSTONE 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46388 | MILLSTONE 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46457 | MILLSTONE 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| | | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| | | | | <u>COMPLETE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42823 | MILLSTONE 1 - T.S. CHANGES TO PERMIT LESS THAN 3 CPS ON SRM'S UNDER CERTAIN CONDITIONS | | | 10/09/80 |
| 07363 | MILLSTONE 1 - ETS CHANGES | | 1 | 10/10/80 |
| 42439 | MILLSTONE 1 & 2 - ORGANIZATIONAL CHANGES (APPENDICES A&B) | | | 02/13/81 |
| 13131 | MILLSTONE 1 - EXTEND T.S. MAPLNGR LIMITS FROM 30,000 TO 40,000 MWDC | | 1 | 03/11/81 |
| 42822 | MILLSTONE 1 - RELOAD 7 T.S. CHANGES | | | 03/11/81 |
| 43226 | MILLSTONE 1 - STNDBY LIQ CONTROL SYSTEM T.S. CHG REDUCE MIN DELTA K SUBCRITICAL FM 3-2.6% & 80 CONC FM 600-660 PPM | | | 03/11/81 |
| 11935 | MILLSTONE 1 - ETS SECTIONS 3 AND 4 CHANGES | | 1 | 04/06/81 |
| 43537 | MILLSTONE 1 - ROD WORTH MINIMIZER TEST | | | 04/16/81 |
| 43538 | MILLSTONE 1 - NEW MULTIPLIER FOR APRM ROD BLOCK MONITOR | | | 04/16/81 |
| 43542 | MILLSTONE 1 - ECCS & SLC EQUIPMENT SURVEILLANCE TESTING | | | 04/16/81 |
| 43564 | MILLSTONE 1 - HOUSEKEEPING T.S. CHANGES | | | 04/16/81 |
| | | | | <u>TARGET</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12784 | MILLSTONE 1 - ISO COND STEAMLINE RESTRAINT | | 1 | |
| 41014 | MILLSTONE 1 - IREP | | | |
| 43245 | MILLSTONE 1 - QUALITY ASSURANCE | | | |
| 43536 | MILLSTONE 1 - REACTOR PROTECTION SYSTEM RESPONSE TIME | | 2 | |
| 43539 | MILLSTONE 1 - PERSONNEL AIR-LOCK DOOR LEAK TEST | | 2 | |
| 43540 | MILLSTONE 1 - CONTROL ROD ACCUMULATOR | | 2 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MILLSTONE 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|----------------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | (CONTINUATION) | | <u>TARGET</u> |
| 43541 | MILLSTONE 1 - TEST & CALIB. FREQ. FOR CORE COOLING INSTRUMENTS | | 2 | |
| 43789 | MILLSTONE 1 - LOW PRESSURE TURBINE FAILURE | | | |
| 42612 | MILLSTONE 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |
| 42913 | MILLSTONE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | | |
| <u>ANTICIPATED ACTIONS</u> | | | | <u>INITIATION</u> |
| 46497 | MILLSTONE-1 SALP EVAL (CYC-2) 7/1 TO 6/30/81 | | | 06/04/81 |
| 11221 | MILLSTONE 1 - CONVERSION POL TO FTOL | | 1 | 06/30/81 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: MILLSTONE 2

PLANT LOCATION: 5 MI SW OF NEW LONDON, CONN
 DOCKET NUMBER: 050-00336
 ARCH/ENGINEER: BECH
 IE INSPECTOR: J. STREETER

LICENSED POWER: 2700 MWT
 DESIGN POWER: 0870 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: E. CONNER
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10214 | MILLSTONE 2 - CONTAINMENT PURGE | | 1 | <u>COMPLETE</u> 10/06/80 |
| 12505 | MILLSTONE 2 - CYCLE 4 RELOAD ANALYSIS | | 1 | 10/06/80 |
| 07575 | MILLSTONE 2 - SITE EMERGENCY PLAN | | 1 | 10/07/80 |
| 11117 | MILLSTONE 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 10/21/80 |
| 10637 | MILLSTONE 2 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/14/80 |
| 43029 | MILLSTONE 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 11/19/80 |
| 11391 | MILLSTONE 2 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | 02/05/81 |
| 11733 | MILLSTONE 2 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 04/07/81 |
| 42112 | MILLSTONE 2 - DECAY HEAT REMOVAL CAPABILITY TECH SEPCS CAPABILITY TECH SPECS | | 2 | 05/19/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08061 | MILLSTONE 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | <u>TARGET</u> |
| 08116 | MILLSTONE 2 - APPENDIX 1 TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08241 | MILLSTONE 2 - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS-GENERIC | | 2 | |
| 0P320 | MILLSTONE 2 - INSERVICE INSPECTION | | 1 | |
| 08949 | MILLSTONE 2 - REVIEW OF ASYMMETRIC LOCA LOADS | | 2 | |
| 08952 | MILLSTONE 2 - TS SURVEILLANCE FOR MECHANICAL SNUBBERS | | 2 | |
| 08957 | MILLSTONE 2 - STEAM GENERATOR TUBE DENTING AND SUPPORT PLATE MODIFICATIONS-CE | | 3 | |
| 08969 | MILLSTONE 2 - TS SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 10376 | MILLSTONE 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10468 | MILLSTONE 2 - BARON SALABILITY DURING LONG TERM COOLING FOLLOWING LOCA | | 3 | |
| 10638 | MILLSTONE 2 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10657 | MILLSTONE 2 - VITAL AREA ANALYSIS | | 1 | |
| 11290 | MILLSTONE 2 - INSERVICE TESTING | | 1 | |
| 12318 | MILLSTONE 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE | | 3 | |
| 12672 | MILLSTONE 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 12983 | MILLSTONE 2 - EMERGENCY PLANNING & REVISIONS | | 1 | |
| 42685 | MILLSTONE 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42756 | MILLSTONE 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43656 | MILLSTONE 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 08947 | MILLSTONE 2 - PWR PUMS & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 07220 | MILLSTONE 2 - CE GENERIC CEA GUIDE TUBE WEAR | | 3 | |
| 07089 | MILLSTONE 2 - C E GENERIC - RPS CHANNEL INOPERABILITY | | 3 | |
| 07318 | MILLSTONE 2 - CEA POSITION INDICATOR CHANNEL | | 3 | |
| 07545 | MILLSTONE 2 - 15 CE GENERIC RTD RESPONSE TIME | | 3 | |
| 43926 | MILLSTONE UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42894 | MILLSTONE 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43142 | MILLSTONE 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42080 | MILLSTONE 2 - ANALYSIS OF TURBINE DISCS IN G.E. TURBINES | | 2 | |
| 42588 | MILLSTONE 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42493 | MILLSTONE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MILLSTONE 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|-----------------------|------------------------|--|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44035 | MILLSTONE 2 | - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44104 | MILLSTONE 2 | - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44175 | MILLSTONE 2 | - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44246 | MILLSTONE 2 | - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44316 | MILLSTONE 2 | - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44385 | MILLSTONE 2 | - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | |
| 44454 | MILLSTONE 2 | - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44525 | MILLSTONE 2 | - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44595 | MILLSTONE 2 | - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44665 | MILLSTONE 2 | - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44705 | MILLSTONE 2 | - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44746 | MILLSTONE 2 | - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44793 | MILLSTONE 2 | - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44864 | MILLSTONE 2 | - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44935 | MILLSTONE 2 | - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | |
| 45006 | MILLSTONE 2 | - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45077 | MILLSTONE 2 | - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45148 | MILLSTONE 2 | - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45264 | MILLSTONE 2 | - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | |
| 45310 | MILLSTONE 2 | - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | |
| 45357 | MILLSTONE 2 | - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45431 | MILLSTONE 2 | - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | |
| 45643 | MILLSTONE 2 | - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45840 | MILLSTONE 2 | - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45959 | MILLSTONE 2 | - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46030 | MILLSTONE 2 | - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46102 | MILLSTONE 2 | - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46174 | MILLSTONE 2 | - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | |
| 46246 | MILLSTONE 2 | - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46318 | MILLSTONE 2 | - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46389 | MILLSTONE 2 | - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46458 | MILLSTONE 2 | - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|------------------------|---|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11348 | MILLSTONE 2 | - CONVERSION TO WESTINGHOUSE FUEL | 1 | COMPLETE 10/06/80 |
| 11561 | MILLSTONE 2 | - ESF COMPONENT LEAKAGE OUTSIDE CONTAINMENT | 1 | 10/06/80 |
| 42625 | MILLSTONE 2 | - DEFINITION OF OPERABLE | 1 | 11/19/80 |
| 42846 | MILLSTONE 2 | - REACTOR COOLING SYSTEM VENTS (AP II.B.1) | 1 | 12/12/80 |
| 43248 | MILLSTONE 2 | - S.L.M GENERATOR TUBE INSPECTION METHOD | 2 | 01/08/81 |
| 42410 | MILLSTONE 2 | - AUTO INITIATION OF AUX FEEDWATER | 1 | 01/14/81 |
| 43559 | MILLSTONE 2 | - AFW'S ENDURANCE TEST AND AUTOMATION OF STEAM TURBINE PUMP | 1 | 03/19/81 |
| 10857 | MILLSTONE 2 | - REMOVAL OF CONFLICTING STS AND ETS REQUIREMENTS | 1 | 04/06/81 |
| 11936 | MILLSTONE 2 | - ETS SECTION 3 AND 4 CHANGES | 1 | 04/06/81 |
| 06480 | MILLSTONE 2 | - ESAS LOGIC TEST (27/76) | 2 | 04/09/81 |
| 43842 | MILLSTONE 2 | - PORV ACTION STATEMENT CORRECTION | | 05/06/81 |

ACTIVE ACTIONS

| | | | | |
|-------|-------------|---|---|--------|
| 11327 | MILLSTONE 2 | - HYDRAULIC SNUBBER TS ACTION STATEMENT | 2 | TARGET |
|-------|-------------|---|---|--------|

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(CONTINUATION)

MILLSTONE 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | <u>(CONTINUATION)</u> | | |
| 11347 | MILLSTONE 2 - UPGRADE STS | | 1 | <u>TARGET</u> |
| 11763 | MILLSTONE 2 - REMOVAL OF RADWASTE EVAPORATOR | | 1 | |
| 43575 | MILLSTONE 2 - AFFECTS OF DC BUS LOSS ON AC EMERGENCY POWER | | 1 | |
| 11522 | MILLSTONE 2 - APPROPRIATENESS OF EBFS DESIGN | | 2 | |
| 43543 | MILLSTONE 2 - AFFECTS OF LOSS OF 125V DC ON AC SYSTEMS | | 1 | |
| 43380 | MILLSTONE 2 - REVIEW OF W BASIC SAFETY REPORT FOR M-2 | | 2 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: MONTICELLO

PLANT LOCATION: 30 MI NW OF MINNEAPOLIS, MINN
 DOCKET NUMBER: 050-00263
 ARCH/ENGINEER: BECH
 IE INSPECTOR: N. CHOULES

LICENSED POWER: 1670 MWT
 DESIGN POWER: 0545 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: K. ECCLESTON
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. SHEPPARD

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10301 | MONTICELLO - CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATION | | 3 | <u>COMPLETE</u> 11/14/80 |
| 42223 | MONTICELLO - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | 01/09/81 |
| 1003 | MONTICELLO - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH DEGRADED GRID VOLTAGE | | 2 | 03/27/81 |
| 10670 | MONTICELLO - CONTINGENCY PLAN REVIEW | | 1 | 03/27/81 |
| 10675 | MONTICELLO - GUARD TRAINING PLAN REVIEW | | 1 | 03/27/81 |
| 12337 | MONTICELLO - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12306 | MONTICELLO - HELB AND CONSEQUENTIAL SYSTEM FAILURE MONTICELLO | | 3 | <u>TARGET</u> |
| 12858 | MONTICELLO - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 41055 | MONTICELLO - CONTROL RODS FAIL TO FULLY INSERT | | 3 | |
| 42254 | MONTICELLO - REVIEW CORPORATE CAPABILITIES | | 3 | |
| 43020 | MONTICELLO - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 11004 | MONTICELLO - MECHANICAL SNUBBERS | | 2 | |
| 07942 | MONTICELLO - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 08944 | MONTICELLO - RPS POWER SUPPLY | | 1 | |
| 07253 | MONTICELLO - REVISED INSERVICE INSPECTION REQUIREMENTS | | 1 | |
| 08306 | MONTICELLO - 10 CFR 50.55A(G) INSERVICE INSPECTION/TESTING - GENERIC | | 1 | |
| 08117 | MONTICELLO - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 42686 | MONTICELLO - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42757 | MONTICELLO - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 08062 | MONTICELLO - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08551 | MONTICELLO - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | |
| 10665 | MONTICELLO - VITAL AREA ANALYSIS | | 1 | |
| 43720 | MONTICELLO - SAF CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | 3 | |
| 10363 | MONTICELLO - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 2 | |
| 42883 | MONTICELLO - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 3 | |
| 06190 | MONTICELLO - PROPOSED LICENSE AMENDMENT FOR SINGLE LOOP OPERATION | | 1 | |
| 11281 | MONTICELLO - INSERVICE TESTING | | 1 | |
| 08222 | MONTICELLO - STRESS CORROSION CRACKING-BWR RCS PB - GENERIC | | 1 | |
| 11134 | MONTICELLO - APP J-CONTAINMENT LEAK TESTING | | 2 | |
| 11093 | MONTICELLO - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 42577 | MONTICELLO - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42475 | MONTICELLO - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TMI ACTIONS</u> | | | | |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44036 | MONTICELLO - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44105 | MONTICELLO - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44176 | MONTICELLO - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44247 | MONTICELLO - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44317 | MONTICELLO - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44386 | MONTICELLO - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MONTICELLO

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TAC ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|-------------------------------------|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44455 | MONTICELLO - NUREG-0737 II.B.3.2, | POST ACCIDENT SAMPLING MODIFICATIONS | | <u>TARGET</u> |
| 44526 | MONTICELLO - NUREG-0737 II.B.4.1, | TRAINING FOR MITIGATING CORE | | |
| 44596 | MONTICELLO - NUREG-0737 II.D.1.2, | RELIEF AND SAFETY VALVE TESTING | | |
| 44794 | MONTICELLO - NUREG-0737 II.E.4.1.2, | DEDICATED HYDROGEN PENETRATIONS | | |
| 44865 | MONTICELLO - NUREG-0737 II.E.4.2, | CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44936 | MONTICELLO - NUREG-0737 II.F.1.1, | NOBLE GAS MONITOR | | |
| 45007 | MONTICELLO - NUREG-0737 II.F.1.2, | IODINE/ PARTICULATE SAMPLING | | |
| 45078 | MONTICELLO - NUREG-0737 II.F.1.3, | CONTAINMENT HIGH RANGE MONITOR | | |
| 45149 | MONTICELLO - NUREG-0737 II.F.2.3, | INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45358 | MONTICELLO - NUREG-0737 II.K.3.3, | REPORT ON RV/SV FAILURES | | |
| 45546 | MONTICELLO - NUREG-0737 II.K.3.13, | HPCI AND RCIC INITIATION LEVELS | | |
| 45574 | MONTICELLO - NUREG-0737 II.K.3.15, | ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45598 | MONTICELLO - NUREG-0737 II.K.3.16, | CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45644 | MONTICELLO - NUREG-0737 II.K.3.17, | ECC SYSTEM OUTAGES | | |
| 45694 | MONTICELLO - NUREG-0737 II.K.3.18, | ADS ACTUATION STUDY | | |
| 45718 | MONTICELLO - NUREG-0737 II.K.3.19, | INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45742 | MONTICELLO - NUREG-0737 II.K.3.21, | RESTART OF CSS AND LPCI | | |
| 45766 | MONTICELLO - NUREG-0737 II.K.3.25, | POWER ON PUMP SEALS | | |
| 45790 | MONTICELLO - NUREG-0737 II.K.3.27, | COMMON REFERENCE LEVEL | | |
| 45841 | MONTICELLO - NUREG-0737 II.K.3.30, | SB LOCA OUTLINE | | |
| 45890 | MONTICELLO - NUREG-0737 II.K.3.44, | TRANSIENTS WITH SINGLE FAILURES | | |
| 45914 | MONTICELLO - NUREG-0737 II.K.3.45, | MANUAL DEPRESSURIZATION | | |
| 45960 | MONTICELLO - NUREG-0737 III.A.1.2, | TECHNICAL SUPPORT CENTER | | |
| 46031 | MONTICELLO - NUREG-0737 III.A.1.2, | OPERATIONAL SUPPORT CENTER | | |
| 46103 | MONTICELLO - NUREG-0737 III.A.1.2, | EMERGENCY OPERATIONS FACILITY | | |
| 46175 | MONTICELLO - NUREG-0737 III.A.1.2, | NUCLEAR DATA DATA | | |
| 46247 | MONTICELLO - NUREG-0737 III.A.1.2, | EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46319 | MONTICELLO - NUREG-0737 III.A.2.2, | METEOROLOGICAL DATA UPGRADE | | |
| 46390 | MONTICELLO - NUREG-0737 III.D.3.3, | INPLANT RADIATION MONITORING | | |
| 46459 | MONTICELLO - NUREG-0737 III.D.3.4, | CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 13261 | MONTICELLO - NRC STAFF GENERIC REPORT ON BWKS | | 1 | <u>COMPLETE</u> 10/24/80 |
| 07252 | MONTICELLO - FULL TERM LICENSE CONVERSION EFFORT | | 2 | 01/09/81 |
| 41056 | MONTICELLO - CLARIFICATION OF "OPERABILITY" | | 1 | 01/22/81 |
| 41065 | MONTICELLO - REQUEST RE: LOSS OF VOLTAGE PROTECTION AND MISC. APP. A TECH SPECS | | 1 | 03/27/81 |
| 43296 | MONTICELLO - EXTEND TABLE OF MAPLHGR LIMITS FOR 8DB250 FUEL | | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 41053 | MONTICELLO - DIESEL GENERATOR RELIABILITY | | 1 | <u>TARGET</u> |
| 41061 | MONTICELLO - DC POWER SUPPLY FAILURE EFFECT ECCS | | 1 | |
| 41064 | MONTICELLO - TURBINE CRACKING (GE) | | 1 | |
| 42256 | MONTICELLO - LESSONS LEARNED TECH SPECS | | 1 | |
| 42257 | MONTICELLO - SCRAM DISCHARGE VOL TECH SPECS | | 1 | |
| 42419 | MONTICELLO - ADDITIONAL TMI RELATED REQUIREMENTS | | 1 | |
| 42360 | MONTICELLO - REQUEST FOR EXEMPTION RE: 10 CFR 50 APP. G | | 1 | |
| 43445 | MONTICELLO - RELOAD 8, CYCLE 9 | | 1 | |
| 43531 | MONTICELLO - REQUEST FOR EXEMPTION FROM REQUIREMENTS OF SECTION IV.J OF 10 CFR 50 APPENDIX R | | 2 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MONTICELLO

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|-----------------|---|----------|-------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 43770 | MONTICELLO - | REQUEST FOR EXEMPTION FROM SCHEDULAR REQUIREMENTS OF 10 CFR 50-4T (C) | 1 | |
| 43496 | MONTICELLO - | SNUBBER SURVEILLANCE TECH SPECS | 1 | |
| 46509 | MONTICELLO - | T/S CHGS ASSOCIATED WITH MARK I CONTAINMENT LONG TERM PROGRAM | 1 | |
| 07587 | MONTICELLO - | ENVIRONMENTAL TECH SPEC WORK | 1 | |
| 43496 | MONTICELLO - | PERIODIC UPDATING OF FINAL SAFETY ANALYSIS REPORT-INITIAL SUBMITTAL | 1 | |
| 43497 | MONTICELLO - | 10 CFR 50.48 AND 10 CFR 50 APPENDIX R FIRE PROTECTION REQUIREMENTS | 1 | |
| 46510 | MONTICELLO - | ISI/IST PROGRAM-SECOND TEN YEAR INTERVAL | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: NINE MILE POINT 1

PLANT LOCATION: 8 MI NE OF OSWEGO, NY
 DOCKET NUMBER: 050-00220
 ARCH/ENGINEER: NIAGARA
 IE INSPECTOR: S. HUDSON

LICENSED POWER: 1850 MWT
 DESIGN POWER: 0620 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: P. POLK
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42758 | NINE MILE POINT 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | <u>COMPLETE</u> 12/31/80 |
| 11078 | NINE MILE POINT 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/13/81 |
| 10417 | NINE MILE POINT 1 - CONTINGENCY PLAN REVIEW | | 1 | 02/27/81 |
| 10418 | NINE MILE POINT 1 - GUARD TRAINING PLAN REVIEW | | 1 | 02/27/81 |
| 10087 | NINE MILE POINT 1 - ENHANCED FISSION PRODUCT R EASE FOR HIGH BURNUP LWR FUEL | | 8 | 03/10/81 |
| 42221 | NINE MILE POINT 1 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | 03/27/81 |
| 12167 | NINE MILE POINT 1 - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 07899 | NINE MILE POINT 1 - DIESEL GENERATOR LOCKOUT | | 3 | 04/03/81 |
| 06077 | NINE MILE POINT 1 - REVISED REQUIREMENTS FOR 1ST GENERIC REV | | 1 | 04/17/81 |
| 07943 | NINE MILE POINT 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | 04/17/81 |
| 08719 | NINE MILE POINT 1 - MECHANICAL SNUBBER | | 2 | 05/01/81 |
| 08725 | NINE MILE POINT 1 - HYDRAULIC SNUBBERS | | 3 | 05/01/81 |
| 13155 | NINE MILE POINT 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | 05/01/81 |
| 08223 | NINE MILE POINT 1 - STRESS CORROSION CRACKING-BWR RCSPB GENERIC | | 1 | 05/26/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 11739 | NINE MILE POINT 1 - THREE MILE ISLAND FOLLOWUP WORK | | 1 | <u>TARGET</u> |
| 12307 | NINE MILE POINT 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE NINE MILE POINT | | 3 | |
| 43021 | NINE MILE POINT 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 07442 | NINE MILE POINT 1 - SFP MODIFICATION | | 1 | |
| 08063 | NINE MILE POINT 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 07956 | NINE MILE POINT 1 - NON JET PUMP SPRAY PERFORMANCE | | 3 | |
| 08943 | NINE MILE POINT 1 - RPS POWER SUPPLY | | 1 | |
| 08118 | NINE MILE POINT 1 - APPENDIX I TECH SPECS IMPLEMENTATION REVIEW | | 3 | |
| 10364 | NINE MILE POINT 1 - LOSS OF 125-V BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08335 | NINE MILE POINT 1 - 10 CFR 50.55A(G) INSERVICE INSPECTION GENERIC | | 1 | |
| 03029 | NINE MILE POINT 1 - REQUESTED EXEMPTIONS FROM 10CFR50 APPENDIX J | | 2 | |
| 06916 | NINE MILE POINT 1 - CONVERSION TESTS | | 3 | |
| 42884 | NINE MILE POINT 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42687 | NINE MILE POINT 1 - IE BULLETIN 79-27, LOSS OF NON-CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43924 | NINE MILE POINT UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43718 | NINE MILE POINT 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | 1 | |
| 42476 | NINE MILE POINT 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 41032 | NINE MILE-REACTOR VESSEL WATER LEVEL INSTRUMENTATION | | 1 | <u>COMPLETE</u> 04/17/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44037 | NINE MILE POINT 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44106 | NINE MILE POINT 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44177 | NINE MILE POINT 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NINE MILE POINT 1

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|------------------------|--|-----------------|-------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44248 | NINE MILE POINT 1 | - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44318 | NINE MILE POINT 1 | - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44387 | NINE MILE POINT 1 | - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | |
| 44456 | NINE MILE POINT 1 | - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44527 | NINE MILE POINT 1 | - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44597 | NINE MILE POINT 1 | - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44795 | NINE MILE POINT 1 | - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44866 | NINE MILE POINT 1 | - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44937 | NINE MILE POINT 1 | - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | |
| 45008 | NINE MILE POINT 1 | - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45079 | NINE MILE POINT 1 | - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45359 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45547 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45557 | NINE MILE POINT 1 | - NUREG-0737, II.K.3.14, ISOLATION OF ISOLATION CONDENSERS ON HIGH RADIATION | | |
| 45575 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45599 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45645 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45695 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY | | |
| 45719 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45743 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45767 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.25, POWER ON PUMP SEALS | | |
| 45791 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45801 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.29, PERFORMANCE OF ISOLATION CONDENSER | | |
| 45842 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45891 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45915 | NINE MILE POINT 1 | - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45961 | NINE MILE POINT 1 | - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46032 | NINE MILE POINT 1 | - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46104 | NINE MILE POINT 1 | - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46176 | NINE MILE POINT 1 | - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | |
| 46248 | NINE MILE POINT 1 | - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46320 | NINE MILE POINT 1 | - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46391 | NINE MILE POINT 1 | - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46460 | NINE MILE POINT 1 | - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>PLANT SPECIFIC</u> | | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 12259 | NINE MILE POINT 1 | - N-2 LOOP OPERATION | 1 | 12/12/80 |
| 43119 | NINE MILE POINT 1 | - SECONDARY CONTAINMENT INTEGRITY | 1 | 12/12/80 |
| 42292 | NINE MILE POINT 1 | - MAXIMUM CONDENSER DELTA T | 1 | 01/12/81 |
| 11905 | NINE MILE POINT 1 | - ADMIN. PROCEDURES (SRO REFUELING REQ.) | 1 | 02/27/81 |
| 42423 | NINE MILE POINT 1 | - 50.59 RELOAD | 1 | 02/27/81 |
| 10391 | NINE MILE POINT 1 | - RADWASTE REDUCTION SYSTEM (EVALUATE DESIGN PER 10 CFR 20.305) | 1 | 04/03/81 |
| 43413 | NINE MILE POINT 1 | - POTENTIAL HEARING ON SHOW CAUSE ORDER OF 11/26/80 | 1 | 04/17/81 |
| 43529 | NINE MILE POINT 1 | - CORE SPRAY SYSTEM MODIFICATION | 1 | 05/01/81 |
| 41083 | NINE MILE POINT 1 | - DIESEL GENERATOR RELIABILITY | 1 | 05/01/81 |
| 42293 | NINE MILE POINT 1 | - SECURITY FORCE HUMAN ERROR | 1 | 05/01/81 |
| 42295 | NINE MILE POINT 1 | - HEALTH EFFECT OF LL IONIZING RADIATION | 1 | 05/01/81 |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NINE MILE POINT 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 41084 | NINE MILE POINT 1 - TECHNICAL SPECIFICATION OPERABILITY DEFINITION | | 1 | <u>TARGET</u> |
| 41085 | NINE MILE POINT 1 - CONTROL RODS FAIL TO FULLY INSERT FULLY INSERT | | 1 | |
| 41086 | NINE MILE POINT 1 - STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 1 | |
| 41087 | NINE MILE POINT 1 - REVIEW OF CORPORATE CAPABILITIES | | 1 | |
| 42425 | NINE MILE POINT 1 - MANAGEMENT REORGANIZATION | | 2 | |
| 07431 | NINE MILE POINT 1 - SNUBBER INSPECTIONS | | 1 | |
| 07443 | NINE MILE POINT 1 - ADDITION OF N2 ISOLATION VALVES | | 1 | |
| 42294 | NINE MILE POINT 1 - REACTOR BUILDING CRACK | | 1 | |
| 42424 | NINE MILE POINT 1 - SNUBBER ADDITIONS AND DELETIONS | | 2 | |
| 43706 | NINE MILE POINT 1 - SCRAM DISCHARGE VOLUME MODIFICATION | | 1 | |
| 42578 | NINE MILE POINT 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 43528 | NINE MILE POINT 1 - TECHNICAL SPECIFICATION | | 2 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: NORTH ANNA 1

PLANT LOCATION: 40 MI NW OF RICHMOND, VA
 DOCKET NUMBER: 050-00338
 ARCH/ENGINEER: S&W
 IE INSPECTOR: T. WEBSER

LICENSED POWER: 2775 MWT
 DESIGN POWER: 1907 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: L. ENGLE
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | AL DATE |
|--------------------------|--|-------------|----------|----------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11574 | NORTH ANNA 1 - CONTINGENCY PLAN REVIEW | | 1 | 10/03/80 |
| 10161 | NORTH ANNA 1 - CONTAINMENT PURGE | | 1 | 11/21/80 |
| 11116 | NORTH ANNA 1 - B-24 VENTING AND PURGING REVIEW OF NORTH ANNA UNITS | | 1 | 11/21/80 |
| 43030 | NORTH ANNA 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 12/10/80 |
| 11575 | NORTH ANNA 1 - GUARD TRAINING PLAN | | 1 | 03/20/81 |
| 12929 | NORTH ANNA 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 11022 | NORTH ANNA 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | 05/28/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 43666 | NORTH ANNA 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | TARGET |
| 12270 | NORTH ANNA 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE NORTH ANNA 1 | | 3 | |
| 12245 | NORTH ANNA 1 - ISI PROGRAM | | 1 | |
| 42113 | NORTH ANNA 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS CAPABILITY TECH SPECS | | 2 | |
| 42688 | NORTH ANNA 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 12605 | NORTH ANNA 1 - ANALYSIS OF TURBINE DISC CRACKS | | 3 | |
| 42759 | NORTH ANNA 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 2 | |
| 43941 | NORTH ANNA 1 - STATION BLACKOUT PROCEDURES & TRAINING | | 2 | |
| 13109 | NORTH ANNA 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 42895 | NORTH ANNA 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43143 | NORTH ANNA 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 1 | |
| 42497 | NORTH ANNA 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>IMI ACTIONS</u> | | | | |
| <u>ACTIVE ACTIONS</u> | | | | |
| 43191 | NORTH ANNA 1 - LT REQUIREMENTS FOR AUX FEED INITIATION & FLOW INDICATION | | 1 | TARGET |
| 44038 | NORTH ANNA 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44107 | NORTH ANNA 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44178 | NORTH ANNA 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44249 | NORTH ANNA 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44319 | NORTH ANNA 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44388 | NORTH ANNA 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44457 | NORTH ANNA 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44528 | NORTH ANNA 1 - NUREG-0737 II.B.3.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44598 | NORTH ANNA 1 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44643 | NORTH ANNA 1 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44722 | NORTH ANNA 1 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44796 | NORTH ANNA 1 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44867 | NORTH ANNA 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44938 | NORTH ANNA 1 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45009 | NORTH ANNA 1 - NUREG-0737 II.F.1.2, IODINE PARTICULATE SAMPLING | | | |
| 45080 | NORTH ANNA 1 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45150 | NORTH ANNA 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45240 | NORTH ANNA 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 1

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|-----------------|---|----------|-------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 45286 | NORTH ANNA 1 | - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | |
| 45360 | NORTH ANNA 1 | - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45403 | NORTH ANNA 1 | - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | |
| 45452 | NORTH ANNA 1 | - NUREG-0737 II.K.3.9, PID CONTROLLER | | |
| 45482 | NORTH ANNA 1 | - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45512 | NORTH ANNA 1 | - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45646 | NORTH ANNA 1 | - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45843 | NORTH ANNA 1 | - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45962 | NORTH ANNA 1 | - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46033 | NORTH ANNA 1 | - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46105 | NORTH ANNA 1 | - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46177 | NORTH ANNA 1 | - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | |
| 46249 | NORTH ANNA 1 | - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46321 | NORTH ANNA 1 | - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46392 | NORTH ANNA 1 | - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46461 | NORTH ANNA 1 | - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE COMPLETE |
|--------------------------|--------------------------------------|--|----------|---------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42552 | NORTH ANNA 1 | - REVISED LICENSE CONDITION 2.D.(3)J | | 11/19/80 |
| 43177 | NORTH ANNA 1 | - SITE VISIT (11-13-80) NRC & VEPCO UPDATE ON ACTIVE ITEMS | | 11/19/80 |
| 12546 | NORTH ANNA 1 | - SER FOR STEAM GENERATOR REPORT & CORRECTIVE ACTIONS | | 11/25/80 |
| 42935 | NA-1 SG INSPECTION PART INSTALLATION | | | 12/08/80 |
| 43179 | NORTH ANNA 1 | - TECH SPEC CHANGE FOR REMAINDER CYCLE 2 | | 12/10/80 |
| 43178 | NORTH ANNA 1 | - AMEND APPENDIX B TECH SPECS | | 12/31/80 |
| 42384 | NORTH ANNA 1 | - LER 80-061/OIT; 7/17/80-AFW PUMP LOW FLOW | | 01/07/81 |
| 43459 | NORTH ANNA 1 | - SITE VISIT (2/19/81) RE: ASLAP/REPLACEMENT AT LPROTOR #2 | | 02/26/81 |
| 43458 | NORTH ANNA 1&2 | - TECH SPEC AMEND'T; CHARGING PUMP CROSS CONNECT MODS | 1 | 03/18/81 |
| 43525 | NORTH ANNA 1 | - GREEN TICKET #10159, INFO REQ. SPENT FUEL STORAGE | 2 | 03/19/81 |
| 43566 | NORTH ANNA 1 | - LICENSE AMENDMENT; SNUBBER RELOCATION PER ARS | | 03/24/81 |
| 42178 | NORTH ANNA 1 | - MULTIPLE STRUCTURE ARS CONCERN ARS CONCERN | 1 | 04/17/81 |
| 43624 | NORTH ANNA 1&2 | - T.S. CHANGE: FUEL ENRICHMENT TO 4.1% | | 04/29/81 |
| 42364 | NORTH ANNA 1 | - TECH SPEC CHANGE - SUB ATMOS CSSAE & QA | 2 | 05/14/81 |
| 43864 | NORTH ANNA 1 | - T/S TABLE 3.7-4. SNUBBER REMOVAL PER LETTER 5/13 & 5/15/81 | | 05/15/81 |
| 42830 | NORTH ANNA 1&2 | - T.S. CHANGE; ADMINISTRATIVE, SERIAL NO 731 | 2 | 05/22/81 |
| 42359 | NORTH ANNA 1 | - CONSISTENCY CHECK ON LICENSE NPF-4 (AMD'TS 1 THRU 19) | 1 | 05/28/81 |

| TAC # | TAC DESCRIPTION | PRIORITY | CRITICAL DATE TARGET |
|-----------------------|---|----------|-------------------------|
| <u>ACTIVE ACTIONS</u> | | | |
| 42551 | NA-1 GREEN TICKET CONTROL NO. 09495 | | |
| 42628 | NORTH ANNA 1 - TMI-2 RELATED REQUIREMENTS | 1 | |
| 43855 | NORTH ANNA 1 - TS CHANGE: MIN. WATER LEVEL FOR REFUELING OPS | 1 | |
| 05085 | NORTH ANNA 1 - RELIEF FROM CERTAIN REQUIREMENTS OF ASME SEC FOR INSERVICE TESTING OF CERTAIN PUMPS AND VALVES | | |
| 43180 | NORTH ANNA 1 - TECH SPEC CHANGES FOR CONFORMANCES WITH CATEGORY A | 1 | |
| 05159 | NORTH ANNA 1&2 - RECONVENING OF APPEALS BOARD HEARINGS | 1 | |
| 12660 | NORTH ANNA 1 - REPORT TO ACRS ON SEISMIC DESIGN MARGINS FOR SYSTEMS REQUIRED FOR SAFE SHUTDOWN | 2 | |
| 42589 | NORTH ANNA 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | 1 | |
| 46507 | NORTH ANNA 1 - APPENDIX R; ALTERNATIVE SAFE SHUTDOWN | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: NORTH ANNA 2

PLANT LOCATION: 40 MI NW OF RICHMOND, VA
 DOCKET NUMBER: 050-00339
 ARCH/ENGINEER: S&W
 IE INSPECTOR:

LICENSED POWER: 2775 MWT
 DESIGN POWER: 0907 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: L. ENGLE
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12150 | NORTH ANNA 2 - UPGRADED EMERGENCY PLAN | | 1 | <u>COMPLETE</u> 04/12/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 43673 | NORTH ANNA 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | <u>TARGET</u> |
| 43948 | NORTH ANNA 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42540 | NORTH ANNA 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44039 | NORTH ANNA 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44108 | NORTH ANNA 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44179 | NORTH ANNA 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44250 | NORTH ANNA 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44320 | NORTH ANNA 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44389 | NORTH ANNA 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44458 | NORTH ANNA 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44529 | NORTH ANNA 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44599 | NORTH ANNA 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44644 | NORTH ANNA 2 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44685 | NORTH ANNA 2 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |
| 44723 | NORTH ANNA 2 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44797 | NORTH ANNA 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44868 | NORTH ANNA 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44939 | NORTH ANNA 2 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45010 | NORTH ANNA 2 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45081 | NORTH ANNA 2 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45151 | NORTH ANNA 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45241 | NORTH ANNA 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45287 | NORTH ANNA 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45361 | NORTH ANNA 2 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45404 | NORTH ANNA 2 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45453 | NORTH ANNA 2 - NUREG-0737 II.K.3.9, PID CONTROLLER | | | |
| 45483 | NORTH ANNA 2 - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | | |
| 45513 | NORTH ANNA 2 - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | | |
| 45647 | NORTH ANNA 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45844 | NORTH ANNA 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45963 | NORTH ANNA 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46034 | NORTH ANNA 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46106 | NORTH ANNA 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46173 | NORTH ANNA 2 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46250 | NORTH ANNA 2 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46322 | NORTH ANNA 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46393 | NORTH ANNA 2 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46462 | NORTH ANNA 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 2

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|--------------------------|--|----------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 43057 | NORTH ANNA 2 - REVIEW PRIMARY & SECONDARY ELECTRICAL PENETRATION FOR POWER CONTROL & INSTR OF ELECT CIRCUITS | | | 11/10/80 |
| 43098 | NORTH ANNA 2 - REVIEW VEPCO'S REQUEST TO REVISE TECH. SPEC 4.4.6.2.2 | | | 11/17/80 |
| 43162 | NORTH ANNA 2 - REVIEW RESULTS OF UNIT 2 TURBINE DRIVEN AUX. FEEDWATER PUMP ENDURANCE TEST & PROVIDE EVALUATION | | | 12/09/80 |
| 43214 | NORTH ANNA 2 - VEPCO LTR OF 11/20/80 REQUESTING AN EXTENSION TO THE IMPLEMENT. DATE RELATED TO BULLETIN 79-2 | | | 12/29/80 |
| 09191 | NORTH ANNA 2 - REV CHANGES RE NORTH ANNA UNIT 2 STARTUP PHYSICS TESTING PROGRAM | | 1 | 02/02/81 |
| 43447 | NORTH ANNA 2 - LICENSE AMENDMENT: CONDITION 2.C.(13) | | | 02/10/81 |
| 13227 | NORTH ANNA 2 - TECH SPEC MAINTENANCE & REVISION | | | 02/28/81 |
| 43486 | NORTH ANNA 2 - LICENSING CONDITION 2.C.(14) | | | 03/06/81 |
| 42966 | NORTH ANNA 2 - DETERMINE WHETHER DEFICIENCY EXISTS IN OPERABILITY OF LHSI PUMPS - AND PROVIDE SER | | 1 | 03/11/81 |
| 43599 | NORTH ANNA 2 - LICENSING CONDITION 2.C.(16) | | | 03/31/81 |
| 43764 | NORTH ANNA 2 - 30 DAY EXEMPTION TO T.S. 4.8.2.3.2.D & 4.8.2.4.2 | | | 04/29/81 |
| 43856 | NA-2 - TS CHANGE - CHAN'L FUNCT'L TEST & SJAEEI VALVES | | 2 | 05/14/81 |
| 43763 | NORTH ANNA 2 - T.S. CHANGE, ADMINISTRATIVE, SERIAL NO. 731 | | 2 | 05/22/81 |
| 42936 | NORTH ANNA 2 - PIPING REANALYSIS - CONDITION 2.C.(13) OF NPF-7 | | 1 | 05/29/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 42934 | NORTH ANNA 2 - INSERVICE INSPECTION PROGRAM | | 2 | |
| 43865 | NORTH ANNA 2 - T/S TABLE 3.7-4, SNUBBER REMOVAL PER LETTERS 5/13 & 5/15/81 | | | |
| 46508 | NORTH ANNA 2 - APPENDIX R; ALTERNATIVE SAFE SHUTDOWN | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: OCONEE 1

PLANT LOCATION: 30 MI W OF GREENVILLE, SC
 DOCK IT NUMBER: 050-00269
 ARC /ENGINEER: DUKE/BECH
 IE INSPECTOR: F. JARRE

LICENSED POWER: 2568 MWT
 DESIGN POWER: 0887 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: P. WAGNER
 BRANCH CHIEF: R. REID
 LIC. ASSISTANT: R. INGRAM

| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42978 | OCONEE 1 - INADVERTENT SAFETY SYSTEM ACTUATION DURING SURV. TESTING (B-52) | | 2 | <u>COMPLETE</u> 10/10/80 |
| 10209 | OCONEE 1 - VENTING AND PURGING CONTAINMENT | | 1 | 11/05/80 |
| 08285 | OCONEE 1 - SUPPLEMENTAL T.S. TO OCONEE 1-ISI PROGRAM INSPECTION | | 1 | 11/07/80 |
| 11247 | OCONEE 1 - PWR PRESSURE-TEMPERATURE LIMITS TECH SPECS | | 1 | 11/13/80 |
| 11798 | OCONEE 1 - FEEDWATER LINE CRACKS | | 2 | 12/09/80 |
| 43039 | OCONEE 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 12/10/80 |
| 11750 | OCONEE 1 - REV OF TS CHANGES FOR REVISED RELIEF VALVE SETTING & RC PR. TRIP | | 1 | 01/28/81 |
| 11805 | OCONEE 1 - REVIEW OF EMERG FEEDWATER T.S. CHANGES REQUIRED BY MAY, 7 79 ORDER | | 1 | 01/28/81 |
| 42123 | OCONEE 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 03/09/81 |
| 12116 | OCONEE 1 - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 10544 | OCONEE 1 - CONTINGENCY PLAN REVIEW | | 1 | 04/20/81 |
| 10550 | OCONEE 1 - GUARD TRAINING PLAN REVIEW | | 1 | 04/20/81 |
| 12884 | OCONEE 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 10120 | OCONEE 1 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 1 | 04/20/81 |
| 10380 | OCONEE 1 - LOSS OF 125-V BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | 04/29/81 |
| 42760 | OCONEE 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | 05/06/81 |
| 11088 | OCONEE 1 - FIRE PROTECTION SER SUPP | | 1 | 05/11/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 43643 | OCONEE 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | <u>TARGET</u> |
| 11271 | OCONEE 1-SUPPLEMENTED T.S. TO 1ST PROGRAM | | 2 | |
| 10133 | OCONEE 1 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 11847 | OCONEE 1 - THREE MILE ISLAND FOLLOW-UP WORK | | 1 | |
| 10130 | OCONEE 1 - REV OF ASYMMETRIC LOCA LOADS | | 2 | |
| 10846 | OCONEE 1 - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 42689 | OCONEE 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 10114 | OCONEE 1 - CONTAINMENT LEAK TESTING APP J | | 2 | |
| 08119 | OCONEE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 12323 | OCONEE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE OCONEE 1 | | 3 | |
| 43259 | OCONEE 1 - B&W ACCIDENT INDUCED NEUTRON FLUX ERRORS (MULTI-PLANT ITEM B-64) | | 3 | |
| 42082 | OCONEE 1&2&3 - ANALYSIS OF TURBINE DISCS IN G.E. TURBINES IN G.E. TURBINES | | 2 | |
| 43908 | OCONEE UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 06887 | OCONEE 1 - REACTOR VESSEL OVERPRESSURE PROTECTION | | 2 | |
| 42904 | OCONEE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 12740 | OCONEE 1 - ADEQUACY STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 11274 | OCONEE 1 - SNUBBER TS UPGRADE-HYD & MECH (B-17 & B-22) | | 2 | |
| 42598 | OCONEE 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 08065 | OCONEE 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42509 | OCONEE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| 43610 | OCONEE 1 - FIRE PROTECTION - APP "R" EXEMPTIONS & REVIEWS | | 1 | |
| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44040 | OCONEE 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 1

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------|---|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | (CONTINUATION) | | |
| 44109 | OCONEE 1 - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44180 | OCONEE 1 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44251 | OCONEE 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44321 | OCONEE 1 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44390 | OCONEE 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44459 | OCONEE 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44530 | OCONEE 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44600 | OCONEE 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44670 | OCONEE 1 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44752 | OCONEE 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44798 | OCONEE 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44869 | OCONEE 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44940 | OCONEE 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45011 | OCONEE 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45082 | OCONEE 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45152 | OCONEE 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45185 | OCONEE 1 - NUREG-0737 | II.K.2.10, SAFETY GRADE ARTS | | |
| 45192 | OCONEE 1 - NUREG-0737 | II.K.2.11, OPERATOR TRAINING AND DRILLING | | |
| 45199 | OCONEE 1 - NUREG-0737 | II.K.2.13, THERMAL-MECHANICAL REPORT | | |
| 45206 | OCONEE 1 - NUREG-0737 | II.K.2.14, LIFT FREQUENCY OF PORV'S AND SV'S | | |
| 45213 | OCONEE 1 - NUREG-0737 | II.K.2.16, RCP SEAL DAMAGE | | |
| 45219 | OCONEE 1 - NUREG-0737 | II.K.2.17, POTENTIAL FOR VOIDING IN RCS | | |
| 45225 | OCONEE 1 - NUREG-0737 | II.K.2.20, SYSTEM RESPONSE TO SB LOCA | | |
| 45270 | OCONEE 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45316 | OCONEE 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45362 | OCONEE 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45437 | OCONEE 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45648 | OCONEE 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45845 | OCONEE 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45964 | OCONEE 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46035 | OCONEE 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46107 | OCONEE 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46179 | OCONEE 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46251 | OCONEE 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46323 | OCONEE 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46394 | OCONEE 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46463 | OCONEE 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| 42401 | OCONEE 1 - LESSONS LEARNED - CAT B-ITEM 2.1.7 A REVIEW SAFETY GRADE FW AUTO-INITIATION SYSTEM | | 1 | 09/26/80 |
| 43429 | OCONEE 1 - ITEM II K.2.13 THERMAL-MECH EFFECTS OF HPI ON VESSEL | | 1 | 03/31/81 |
| 42794 | OCONEE 1 - CONFIRMATORY ORDER - ICS RELIABILITY ANALYSIS | | 1 | 12/01/81 |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42337 | OCONEE 1 - PRESSURE TEMP LIMIT CHANGE HEATUP-COOLDOWN CURVES-UNITS 2 & 3 | | | 10/07/80 |
| 13071 | OCONEE 1 - REACTOR COOLANT SYSTEM LEAK TEST SPEC MODIFICATION | | | 11/07/80 |
| 42629 | OCONEE 1 - SAFETY GRADE ANTICIPATORY REACTOR TRIP | | | 12/04/80 |
| 42315 | OCONEE 1 - TECH SPEC CHANGE DEFINING THE TERM "OPERABLE" | | | 12/10/80 |
| 41031 | OCONEE 1 - NUCL STA. EXEMPTION REQUEST TO 10CFR PART 20 - CONTAMINATED OIL BURNING | | | 12/17/80 |
| 43275 | OCONEE 1 - CORRECTIONS TO OCONEE 1-CYCLE 6 POWER DISTRIBUTION CURVES | | | 12/18/80 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 1

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|---|---|----------------|----------|-------------------|
| <u>COMPLETED ACTIONS</u> (CONTINUATION) | | | | <u>COMPLETE</u> |
| 13951 | OCONEE 1 - FUEL STORAGE POOL EXPANSION | | 1 | 12/24/80 |
| 43309 | OCONEE 1 - ISI RELIEF REQUEST FOR 24 INCH MAIN FEEDWATER CHECK VALVES | | | 01/16/81 |
| 42819 | OCONEE 1 - REQUEST TO LICENSEE TO UPGRADE REPORTING REQ IN TECH. SPECS TO R.G.1.16 REV.4 | | | 01/28/81 |
| 42958 | OCONEE 1 - NUREG 0578 CATEGORY A TECHNICAL SPECIFICATIONS | | | 01/28/81 |
| 43289 | OCONEE 1 - REQUEST TO EXTEND SURV. REQUIREMENTS IN TECH. SPECS FROM 12 TO 18 MONTHS. | | 1 | 01/28/81 |
| 43395 | OCONEE 1 - ITEM II K.2.15 SLUG FLOW EFFECTS IN OTSG | | | 02/03/81 |
| 42814 | OCONEE 1 - OCONEE 3 CYCLE 6 RELOAD | | | 02/10/81 |
| 43471 | OCONEE 1 - II K.2.19 BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW | | | 02/27/81 |
| 42268 | OCONEE 1 - FA HOLDDOWN SPRINGS | | 1 | 03/12/81 |
| 43058 | OCONEE 1 - REACTIVITY ANOMOLY AT 97.4 EFPD OF OPERATION DURING CYCLE 6 | | 1 | 03/23/81 |
| 42342 | OCONEE 1 - STEAM GENERATOR TUBE LEAKAGE TECH. SPEC. CHANGE REQUEST | | 1 | 03/30/81 |
| 43304 | OCONEE 1 - INCINERATION OF LOW LEVEL CONTAMINATED OIL | | 1 | 03/31/81 |
| 11702 | OCONEE 1 - NUCL STA. FILTER TS REVISIONS | | 1 | 04/01/81 |
| 43500 | OCONEE 1 - SURVEILLANCE REQUIREMENTS EXTENDED FROM ANNUAL TO 18 MONTHS (SECOND REQUEST) | | 1 | 04/01/81 |
| 43579 | OCONEE 1 - ANTICIPATORY TRIP TECH SPECS | | 2 | 04/01/81 |
| 43183 | OCONEE 1 - VALVES BETWEEN HIGH PRESSURE AND LOW PRESSURE SYSTEMS-TECH. SPEC. SURV. CHANGE REQUEST | | 1 | 04/20/81 |
| 43571 | OCONEE 1 - SEISMIC REEVALUATION | | 3 | 05/22/81 |
| 43532 | OCONEE 1 - NUREG 0737 REVIEW | | | 05/29/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12202 | OCONEE 1 - REVIEW SEISMIC PIPING AVAILABLE USING OVERLAP METHOD | | 1 | |
| 43316 | OCONEE 1 - REVIEW OF RELOAD DESIGN METHODOLOGY TECH. REPORT | | 1 | |
| 43986 | OCONEE 1 - 10 YEAR ISI OF RPV | | 2 | |
| 43790 | OCONEE 1 - EFW TS CHANGE OF 4/17/81 | | | |
| 43983 | OCONEE 1 - OCONEE 1 CYCLE 7 RELOAD | | 1 | |
| 42259 | OCONEE 1 - STANDBY SHUTDOWN FACILITY REVIEW | | 1 | |
| 43809 | OCONEE 1 - RADWASTE TREATMENT FACILITY | | 1 | |
| <u>ANTICIPATED ACTIONS</u> | | | | <u>INITIATION</u> |
| 43994 | OCONEE 1 - ISI RELIEF REQUEST OF 6/1/81 | | | 06/01/81 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: OCONEE 2

PLANT LOCATION: 30 MI W OF GREENVILLE, SC
 DOCKET NUMBER: 050-00270
 ARCH/ENGINEER: DUKE/BECH
 IE INSPECTOR: F. JAPE

LICENSED POWER: 2568 MWT
 DESIGN POWER: 0887 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: P. WAGNER
 BRANCH CHIEF: R. REID
 LIC. ASSISTANT: R. INGRAM

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42979 | OCONEE 2 - INADVERTENT SAFETY SYSTEM ACTUATION DURING SURV. TESTING (B-52) | | 2 | 10/10/80 |
| 10208 | OCONEE 2 - VENTING AND PURGING CONTAINMENT | | 1 | 11/05/80 |
| 08304 | OCONEE 2 - SUPPLEMENTAL T.S. TO OCONEE 2-1ST PROGRAM INSPECTION | | 1 | 11/07/80 |
| 11248 | OCONEE 2 - PWR PRESSURE-TEMPERATURE LIMITS TECH SPECS | | 1 | 11/13/80 |
| 11799 | OCONEE 2 - FEEDWATER LINE CRACKS | | 2 | 12/09/80 |
| 43040 | OCONEE 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 12/10/80 |
| 11751 | OCONEE 2 - REVIEW OF T.S. CHANGES FOR REVISED RELIEF VALVE SETTING AND RC HI PR. TRIP | | 1 | 01/28/81 |
| 11806 | OCONEE 2 - REVIEW OF EMERG. FEEDWATER T.S. CHANGES REQUIRED BY MAY 7, 79 ORDER | | 1 | 01/28/81 |
| 42124 | OCONEE 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 03/09/81 |
| 12117 | OCONEE 2 - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 10545 | OCONEE 2 - CONTINGENCY PLAN REVIEW | | 1 | 04/20/81 |
| 10551 | OCONEE 2 - GUARD TRAINING PLAN REVIEW | | 1 | 04/20/81 |
| 12885 | OCONEE 2 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 10121 | OCONEE 2 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | 04/29/81 |
| 10381 | OCONEE 2 - LOSS OF 125-V BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | 05/06/81 |
| 42761 | OCONEE 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 2 | 05/11/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 43644 | OCONEE 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 11272 | OCONEE 2 - SUPPLEMENTAL T.S. TO OCONEE 2 1ST PROGRAM | | 1 | |
| 10134 | OCONEE 2 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 11848 | OCONEE 2 - TMI FOLLOW WORK | | 1 | |
| 11089 | OCONEE 2 - FIRE PROTECTION SER SUPP | | 1 | |
| 10131 | OCONEE 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 10847 | OCONEE 2 - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 42690 | OCONEE 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 10115 | OCONEE 2 - CONTAINMENT LEAK TESTING APP J | | 2 | |
| 12324 | OCONEE 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE OCONEE 2 | | 3 | |
| 43260 | OCONEE 2 - B&W ACCIDENT INDUCED NEUTRON FLUX ERRORS (MULTI-PLANT ITEM B-64) | | 3 | |
| 43909 | OCONEE UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 08339 | OCONEE 2 - REACTOR VESSEL OVERPRESSURE PROTECTION | | 2 | |
| 42905 | OCONEE 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 12741 | OCONEE 2 - ADEQUACY STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 11275 | OCONEE 2 - SNUBBERS TS UPGRADE-HYD & MECH (B-17 & B-22) | | 2 | |
| 42599 | OCONEE 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 08066 | OCONEE 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08120 | OCONEE 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 42510 | OCONEE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| 43611 | OCONEE 2 - FIRE PROTECTION - APP. "R" EXEMPTIONS & REVIEWS | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 44041 | OCONEE 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44110 | OCONEE 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | <u>(CONTINUATION)</u> | | <u>TARGET</u> |
| 44181 | OCONEE 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44252 | OCONEE 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44322 | OCONEE 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44391 | OCONEE 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44460 | OCONEE 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44531 | OCONEE 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44601 | OCONEE 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44671 | OCONEE 2 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44753 | OCONEE 2 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44799 | OCONEE 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44870 | OCONEE 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44941 | OCONEE 2 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45012 | OCONEE 2 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45083 | OCONEE 2 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45153 | OCONEE 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45186 | OCONEE 2 - NUREG-0737 II.K.2.10, SAFETY GRADE ARTS | | | |
| 45193 | OCONEE 2 - NUREG-0737 II.K.2.11, OPERATOR TRAINING AND DRILLING | | | |
| 45200 | OCONEE 2 - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT | | | |
| 45207 | OCONEE 2 - NUREG-0737 II.K.2.14, LIFT FREQUENCY OF PORV'S AND SV'S | | | |
| 45214 | OCONEE 2 - NUREG-0737 II.K.2.16, RCP SEAL DAMAGE | | | |
| 45220 | OCONEE 2 - NUREG-0737 II.K.2.17, POTENTIAL FOR VOIDING IN RCS | | | |
| 45226 | OCONEE 2 - NUREG-0737 II.K.2.20, SYSTEM RESPONSE TO SB LOCA | | | |
| 45271 | OCONEE 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45317 | OCONEE 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 4536 | OCONEE 2 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45438 | OCONEE 2 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45649 | OCONEE 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45846 | OCONEE 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45965 | OCONEE 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46036 | OCONEE 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46108 | OCONEE 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46180 | OCONEE 2 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46252 | OCONEE 2 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46324 | OCONEE 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46395 | OCONEE 2 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46464 | OCONEE 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |
| 42402 | OCONEE 2 - LESSONS LEARNED - CAT. B - ITEM 2.1.7 A REVIEW SAFETY GRADE FW AUTO-INITIATION SYSTEM | | 1 | |
| 43430 | OCONEE 2 - ITEM II K.2.13 THERMAL-MECH EFFECTS OF HPI ON VESSEL | | 1 | |
| 42795 | OCONEE 2 - CONFIRMATORY ORDER - ICS RELIABILITY ANALYSIS | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42338 | OCONEE 2 - PRESSURE TEMP LIMIT CHANGE HEATUP-COOLDOWN CURVES-UNITS 2 & 3 | | | 10/07/80 |
| 13072 | OCONEE 2 - REACTOR COOLANT SYSTEM LEAK TEST SPEC. MODIFICATION | | 1 | 11/07/80 |
| 42630 | OCONEE 2 - SAFETY GRADE ANTICIPATORY REACTOR TRIP | | | 12/04/80 |
| 42316 | OCONEE 2 - TECH SPEC CHANGE DEFINING THE TERM "OPERABLE" | | | 12/10/80 |
| 43276 | OCONEE 2 - CORRECTIONS TO OCONEE 1-CYCLE 6 POWER DISTRIBUTION CURVES | | | 12/18/80 |
| 13052 | OCONEE 2 - FUEL STORAGE POOL EXPANSION | | 1 | 12/24/80 |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|---|---|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> (CONTINUATION) | | | | <u>COMPLETE</u> |
| 43310 | OCONEE 2 - ISI RELIEF REQUEST FOR 24 INCH MAIN FEEDWATER CHECK VALVES | | | 01/16/81 |
| 42820 | OCONEE 2 - REQUEST TO LICENSEE TO UPGRADE REPORTING REQ. IN TECH SPECS TO R.G. 1.15 REV. 4 | | | 01/28/81 |
| 42959 | OCONEE 2 - NUREG 0578 CATEGORY A TECHNICAL SPECIFICATIONS | | | 01/28/81 |
| 43290 | OCONEE 2 - REQUEST TO EXTEND SURV. REQUIREMENTS IN TECH. SPECS FROM 12 TO 18 MONTHS. | | 1 | 01/28/81 |
| 43396 | OCONEE 2 - ITEM II K 2.15 SLUG FLOW EFFECTS IN OTSG | | | 02/03/81 |
| 42815 | OCONEE 2 - OCONEE 3-CYCLE 6 RELOAD | | | 02/10/81 |
| 43472 | OCONEE 2 - II K.2.19 BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW | | | 02/27/81 |
| 42269 | OCONEE 2 - FA HOLDDOWN SPRINGS | | 1 | 03/12/81 |
| 42343 | OCONEE 2 - STEAM GENERATOR TUBE LEAKAGE TECH. SPEC. CHANGE REQUEST | | 1 | 03/30/81 |
| 43305 | OCONEE 2 - INCINERATION OF LOW LEVEL CONTAMINATED OIL | | 1 | 03/31/81 |
| 11703 | OCONEE 2 - NUCL STA FILTER TS REVISIONS | | 1 | 04/01/81 |
| 43501 | OCONEE 2 - SURVEILLANCE REQUIREMENTS EXTENDED FROM ANNUAL TO 18 MONTHS (SECOND REQUEST) | | 1 | 04/01/81 |
| 43580 | OCONEE 2 - ANTICIPATORY TRIP TECH SPECS | | 2 | 04/01/81 |
| 43184 | OCONEE 2 - VALVES BETWEEN HIGH PRESSURE AND LOW PRESSURE SYSTEMS-TECH. SPEC. SURV. CHANGE REQUEST | | 1 | 04/20/81 |
| 43572 | OCONEE 2 - SEISMIC REEVALUATION | | 3 | 05/22/81 |
| 43533 | OCONEE 2 - NUREG 0737 REVIEW | | 1 | 05/29/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12203 | OCONEE 2 - REVIEW SEISMIC PIPING ANALYSIS USING OVERLAP METHOD | | 1 | |
| 43317 | OCONEE 2 - REVIEW OF RELOAD DESIGN METHODOLOGY TECH. REPORT | | 1 | |
| 43791 | OCONEE 2 - EFW TS CHANGE OF 4/17/81 | | 2 | |
| 43984 | OCONEE 2 - OCONEE 1 CYCLE 7 RELOAD | | | |
| 42260 | OCONEE 2 - STANDBY SHUTDOWN FACILITY REVIEW | | 1 | |
| 43810 | OCONEE 2 - RADWASTE TREATMENT FACILITY | | 1 | |
| <u>ANTICIPATED ACTIONS</u> | | | | <u>INITIATION</u> |
| 43995 | OCONEE 2 - ISI RELIEF REQUEST OF 6/1/81 | | | 06/01/81 |

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CONDENSED MANAGEMENT REPORT

FACILITY: OCONEE 3

PLANT LOCATION: 30 MI W OF GREENVILLE, SC
 DOCKET NUMBER: 050-00287
 ARCH/ENGINEER: DUKE/BECH
 IE INSPECTOR: F. JAPE

LICENSED POWER: 2568 MWT
 DESIGN POWER: 0887 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: P. WAGNER
 BRANCH CHIEF: R. REID
 LIC. ASSISTANT: R. INGRAM

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42980 | OCONEE 3 - INADVERTENT SAFETY SYSTEM ACTUATION DURING SURV. TESTING (B-52) | | 2 | <u>COMPLETE</u> 10/10/80 |
| 10207 | OCONEE 3 - VENTING AND PURGING CONTAINMENT | | 1 | 11/05/80 |
| 08307 | OCONEE 3 - SUPPLEMENTAL T.S. TO OCONEE 3-IST PROGRAM INSPECTION | | 1 | 11/07/80 |
| 08705 | OCONEE 3 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 11800 | OCONEE 3 - FEEDWATER LINE CRACKS | | 2 | 12/09/80 |
| 43041 | OCONEE 3 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 12/10/80 |
| 11752 | OCONEE 3 - REV OF TS CHANGES FOR REVISED RELIEF VALVE SETTING & RC HI PR TRIP | | 1 | 01/28/81 |
| 11807 | OCONEE 3 - REVIEW OF EMERG. FEEDWATER T.S. CHANGES REQUIRED BY MAY 7, 79 ORDER | | 1 | 01/28/81 |
| 42125 | OCONEE 3 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 03/09/81 |
| 12118 | OCONEE 3 - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 10552 | OCONEE 3 - GUARD TRAINING PLAN REVIEW | | 1 | 04/20/81 |
| 12886 | OCONEE 3 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 10122 | OCONEE 3 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | 04/29/81 |
| 10382 | OCONEE 3 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | 05/06/81 |
| 42762 | OCONEE 3 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 2 | 05/11/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 43645 | OCONEE 3 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | <u>TAP I</u> |
| 11273 | OCONEE 3 - SUPPLEMENTAL T.S. TO OCONEE 3 - IST PROGRAM | | 1 | |
| 10135 | OCONEE 3 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 11849 | OCONEE 3 - TMI FOLLOW-UP WORK | | 1 | |
| 11090 | OCONEE 3 - FIRE PROTECTION SER SUPP | | 1 | |
| 10132 | OCONEE 3 - REVIEW OF ASYMMETRIC LOCA LOADS | | 2 | |
| 10848 | OCONEE 3 - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 42691 | OCONEE 3 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 10116 | OCONEE 3 - CONTAINMENT LEAK TESTING APPENDIX J APPENDIX J | | 2 | |
| 12325 | OCONEE 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE OCONEE 3 | | 3 | |
| 43261 | OCONEE 3 - B&W ACCIDENT INDUCED NEUTRON FLUX ERRORS (MULTI-PLANT ITEM B-64) | | 3 | |
| 43910 | OCONEE UNIT 3 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 08340 | OCONEE 3 - REACTOR VESSEL OVERPRESSURE PROTECTION | | 2 | |
| 42906 | OCONEE 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 12742 | OCONEE 3 - ADEQUACY STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 11276 | OCONEE 3 - SNUBBER TS UPGRADE-HYD & MECH (B-17 & B-22) | | 2 | |
| 42600 | OCONEE 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 08067 | OCONEE 3 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08121 | OCONEE 3 - APPENDIX I-ALARA | | 3 | |
| 42511 | OCONEE 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| 43612 | OCONEE 3 - FIRE PROTECTION - APP. "R" EXEMPTIONS & REVIEWS | | 1 | |
| <u>TMI ACTIONS</u> | | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44042 | OCONEE 3 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44111 | OCONEE 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44182 | OCONEE 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |

(CONTINUATION)

000NEE 3

PRIORITY TARGET

IMI ACTIONS

TAC # TAC DESCRIPTION

ACTIVE ACTIONS (CONTINUATION)

| TAC # | TAC DESCRIPTION | PRIORITY | CRITICAL DATE |
|-------|--|----------|---------------|
| 44253 | I.C.1.2.A. INADEQUATE CORE COOLING GUIDELINES | 1 | 10/07/80 |
| 44323 | I.C.1.3A. ABNORMAL TRANSIENT OF REACTOR GUIDELINES | 1 | 11/07/80 |
| 44392 | I.B.1.1. RCS HIGH POINT VENTS | 1 | 12/04/80 |
| 44461 | I.B.3.2. POST ACCIDENT SAMPLING MODIFICATIONS | 1 | 12/10/80 |
| 44532 | I.B.4.1. TRAINING FOR MITIGATING CONDITIONS | 1 | 12/18/80 |
| 44602 | I.D.1.2. RELIEF AND SAFETY VALVE TESTS | 1 | 01/16/81 |
| 44672 | I.E.1.1. APW SYSTEM EVALUATION | 1 | 01/28/81 |
| 44754 | I.E.1.2.1. APW SAFETY GRADE FLOW | 1 | |
| 44800 | I.E.4.1.2. DEDICATED HYDROGEN PENETRATION | 1 | |
| 44871 | I.E.4.2. CONTAINMENT ISOLATION DEFICIENCIES | 1 | |
| 44942 | I.F.1.1. NOBLE GAS MONITOR | 1 | |
| 45013 | I.F.1.2. IODINE/PARTICULATE SAMPLING | 1 | |
| 45084 | I.F.1.3. CONTAINMENT HIGH RANGE MONITOR | 1 | |
| 45154 | I.F.2.3. INADEQUATE CORE COOLING INSTRUMENTATION | 1 | |
| 45187 | I.K.2.10. SAFETY GRADE ARTS | 1 | |
| 45194 | I.K.2.11. OPERATOR TRAINING AND DRILLING | 1 | |
| 45201 | I.K.2.12. THERMAL-MECHANICAL REPORT | 1 | |
| 45208 | I.K.2.14. LIFT FREQUENCY OF PORV'S AND SV'S | 1 | |
| 45213 | I.K.2.15. RCP SEAL DAMAGE | 1 | |
| 45221 | I.K.2.17. POTENTIAL FOR VOIDING IN RCS | 1 | |
| 45227 | I.K.2.20. SYSTEM RESPONSE TO SB LOCA | 1 | |
| 45232 | I.K.3.1. AUTO PORV ISOLATION | 1 | |
| 45338 | I.K.3.2. REPORT ON PORV FAILURES | 1 | |
| 45354 | I.K.3.3. REPORT ON RV/SV FAILURES | 1 | |
| 45439 | I.K.3.5. AUTO TRIP OF RCFS | 1 | |
| 45457 | I.K.3.7. ECC SYSTEM OUTAGES | 1 | |
| 45566 | I.K.3.10. SB LOCA OUTLINE | 1 | |
| 45697 | I.I.A.1.2. TECHNICAL SUPPORT CENTER | 1 | |
| 45709 | I.I.A.1.2. OPERATIONAL SUPPORT CENTER | 1 | |
| 45751 | I.I.A.1.2. EMERGENCY OPERATIONS FACILITY | 1 | |
| 45753 | I.I.A.1.2. NUCLEAR DATA DATA | 1 | |
| 45796 | I.I.A.1.2. EMERGENCY PLAN UPGRADE TO MEET RULE | 1 | |
| 45835 | I.I.A.2.2. METEOROLOGICAL DATA UPGRADE | 1 | |
| 45839 | I.I.D.3.3. INPLANT RADIATION MONITORING | 1 | |
| 45846 | I.I.D.3.4. CONTROL ROOM HABITABILITY | 1 | |
| 45843 | I.I.D.3.4. THERMAL-MECH EFFECTS OF HPI ON VESSEL | 1 | |
| 45843 | LESSONS LEARNED - CAT. B - ITEM 2.1.7 A REVIEW SAFETY GRADE FW AUTO-INITIATION | 1 | |
| 45843 | SYSTEM | 1 | |
| 45843 | CONFIRMATORY ORDER - ICS RELIABILITY ANALYSIS (AP II.K.2.9) | 1 | |
| 45843 | PLANT SPECIFIC | 1 | |
| TAC # | TAC DESCRIPTION | PRIORITY | CRITICAL DATE |
| 42339 | COMPLETED ACTIONS | | |
| 42339 | 000NEE 3 - PRESSURE TEMP LIMIT CHANGE HEATUP-COOLDOWN CURVES-UNITS 2 & 3 | 1 | 10/07/80 |
| 42631 | 000NEE 3 - REACTOR COOLANT SYSTEM LEAK TEST SPEC MODIFICATION | 1 | 11/07/80 |
| 42631 | 000NEE 3 - SAFETY GRADE ANTICIPATORY REACTOR TRIP | 1 | 12/04/80 |
| 43277 | 000NEE 3 - TECH SPEC CHANGE DEFINING THE TERM "OPERABLE" | 1 | 12/10/80 |
| 43311 | 000NEE 3 - CORRECTIONS TO 000NEE 1-CYCLE 6 POWER DISTRIBUTION CURVES | 1 | 12/18/80 |
| 42821 | 000NEE 3 - ISI RELIEF REQUEST FOR 24 INCH MAIN FEEDWATER CHECK VALVES | 1 | 01/16/81 |
| 42821 | 000NEE 3 - REQUEST TO LICENSEE TO UPGRADE REPORTING REQ IN TECH SPECS TO R.G. 1.15 REV.4 | 1 | 01/28/81 |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 3

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|---|---|----------------|----------|-------------------|
| <u>COMPLETED ACTIONS</u> (CONTINUATION) | | | | <u>COMPLETE</u> |
| 42960 | OCONEE 3 - NUREG 0578 CATEGORY A TECHNICAL SPECIFICATIONS | | | 01/28/81 |
| 43291 | OCONEE 3 - REQUEST TO EXTEND SURV. REQUIREMENTS IN TECH. SPECS FROM 12 TO 18 MONTHS. | | 1 | 01/28/81 |
| 43397 | OCONEE 3 - ITEM II K.2.15 SLUG FLOW EFFECTS IN OTSG | | | 02/03/81 |
| 42816 | OCONEE 3 - CYCLE 6 RELOAD | | | 02/10/81 |
| 43473 | OCONEE 3 - II K.2.19 BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW | | | 02/27/81 |
| 42270 | OCONEE 3 - FA HOLDDOWN SPRINGS | | 1 | 03/12/81 |
| 42344 | OCONEE 3 - STEAM GENERATOR TUBE LEAKAGE TECH. SPEC. CHANGE REQUEST | | 1 | 03/30/81 |
| 43306 | OCONEE 3 - INCINERATION OF LOW LEVEL CONTAMINATED DIL | | 1 | 03/31/81 |
| 11704 | OCONEE 3 - NUCL. STA FILTER TS REVISIONS | | 1 | 04/01/81 |
| 43502 | OCONEE 3 - SURVEILLANCE REQUIREMENTS EXTENDED FROM ANNUAL TO 18 MONTHS (SECOND REQUEST) | | 1 | 04/01/81 |
| 43581 | OCONEE 3 - ANTICIPATORY TRIP TECH SPECS | | 2 | 04/01/81 |
| 10546 | OCONEE 3 - CONTINGENCY PLAN REVIEW | | 1 | 04/20/81 |
| 43185 | OCONEE 3 - VALVES BETWEEN HIGH PRESSURE AND LOW PRESSURE SYSTEMS-TECH. SPEC. SURV. CHANGE REQUEST | | 1 | 04/20/81 |
| 43573 | OCONEE 3 - SEISMIC REEVALUATION | | 3 | 05/22/81 |
| 43534 | OCONEE 3 - NUREG 0737 REVIEW | | 1 | 05/29/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 12204 | OCONEE 3 - REVIEW REISMIC PIPING ANALYSIS USING OVERLAP METHOD | | 1 | |
| 43318 | OCONEE 3 - REVIEW OF RELOAD DESIGN METHODOLOGY TECH. REPORT | | 1 | |
| 43792 | OCONEE 3 - EFW TS CHANGE OF 4/17/81 | | 2 | |
| 43985 | OCONEE 3 - OCONEE 1 CYCLE 7 RELOAD | | | |
| 42261 | OCONEE 3 - STANDBY SHUTDOWN FACILITY REVIEW | | 1 | |
| 43811 | OCONEE 3 - RADWASTE TREATMENT FACILITY | | 1 | |
| <u>ANTICIPATED ACTIONS</u> | | | | <u>INITIATION</u> |
| 43996 | OCONEE 3 - ISI RELIEF REQUEST | | | 06/01/81 |

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CONDENSED MANAGEMENT REPORT

FACILITY: OYSTER CREEK 1

PLANT LOCATION: 9 MI S OF TOMS RIVER, NJ
 DOCKET NUMBER: 050-00219
 ARCH/ENGINEER: B&R
 IE INSPECTOR: L. BRIGGS

LICENSED POWER: 1930 MWT
 DESIGN POWER: 0650 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: W. PAULSON
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 08068 | OYSTER CREEK - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | <u>COMPLETE</u> 12/22/80 |
| 07944 | OYSTER CREEK - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | 01/13/81 |
| 10287 | OYSTER CREEK - CONTAINMENT LEAKAGE DUE TO A VALVE SEAT DETERIORATION | | 3 | 01/16/81 |
| 07683 | OYSTER CREEK - GUARD TRAINING PLAN REVIEW | | 1 | 02/11/81 |
| 11104 | OYSTER CREEK - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/13/81 |
| 12291 | OYSTER CREEK - HELB AND CONSEQUENTIAL SYSTEM FAILURE OYSTER CREEK | | 3 | 02/18/81 |
| 12422 | OYSTER CREEK - LESSONS LEARNED IMPLEMENTATION | | 1 | 03/27/81 |
| 12919 | OYSTER CREEK-PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 07147 | OYSTER CREEK - RPS POWER SUPPLY | | 1 | <u>TARGET</u> |
| 08440 | OYSTER CREEK - HYDRAULIC SNUBBERS | | 3 | |
| 12766 | OYSTER CREEK - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 43051 | OYSTER CREEK - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 06440 | OYSTER CREEK - PHYSICAL SEPARATION OF 125 VOLT DC ELECT. CABLES IN BATTERY ROOM. FINAL DESGN EARLY 78. | | 3 | |
| 08100 | OYSTER CREEK - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08678 | OYSTER CREEK - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 07961 | OYSTER CREEK - NON JET PUMP SPRAY PERFORMANCE | | 3 | |
| 10349 | OYSTER CREEK - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08334 | OYSTER CREEK - 10 CFR 50.55A(G)- INSERVICE INSPECTION-GENERIC | | 1 | |
| 10038 | OYSTER CREEK - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 08439 | OYSTER CREEK - MECHANICAL SNUBBERS | | 2 | |
| 10500 | OYSTER CREEK - VITAL AREA ANALYSIS | | 1 | |
| 42215 | OYSTER CREEK - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 11270 | OYSTER CREEK - INSERVICE TESTING (IST) | | 1 | |
| 42692 | OYSTER CREEK - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 12145 | OYSTER CREEK - EMERGENCY PLAN REVIEW | | 1 | |
| 42763 | OYSTER CREEK - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43919 | OYSTER CREEK UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43717 | OYSTER CREEK - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42914 | OYSTER CREEK - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42524 | OYSTER CREEK - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44043 | OYSTER CREEK - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 94112 | OYSTER CREEK - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44183 | OYSTER CREEK - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44254 | OYSTER CREEK - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44324 | OYSTER CREEK - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44393 | OYSTER CREEK - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44462 | OYSTER CREEK - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |

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(CONTINUATION)

OYSTER CREEK 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|---------------------------|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS (CONTINUATION)</u> | | | | |
| 44533 | OYSTER CREEK - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | <u>TARGET</u> |
| 44603 | OYSTER CREEK - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44801 | OYSTER CREEK - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44872 | OYSTER CREEK - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44943 | OYSTER CREEK - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45014 | OYSTER CREEK - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45085 | OYSTER CREEK - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45155 | OYSTER CREEK - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45365 | OYSTER CREEK - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45548 | OYSTER CREEK - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45558 | OYSTER CREEK - NUREG-0737 | II.K.3.14, ISOLATION OF ISOLATION CONDENSERS ON HIGH RADIATION | | |
| 45576 | OYSTER CREEK - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45600 | OYSTER CREEK - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45651 | OYSTER CREEK - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45696 | OYSTER CREEK - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45720 | OYSTER CREEK - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45744 | OYSTER CREEK - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45768 | OYSTER CREEK - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45792 | OYSTER CREEK - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45802 | OYSTER CREEK - NUREG-0737 | II.K.3.29, PERFORMANCE OF ISOLATION CONDENSER | | |
| 45848 | OYSTER CREEK - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45892 | OYSTER CREEK - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45916 | OYSTER CREEK - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45967 | OYSTER CREEK - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46038 | OYSTER CREEK - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46110 | OYSTER CREEK - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46182 | OYSTER CREEK - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46254 | OYSTER CREEK - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46326 | OYSTER CREEK - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46397 | OYSTER CREEK - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46466 | OYSTER CREEK - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42251 | OYSTER CREEK - REV OF NECESSITY FOR AIR DIST TESTS OF SGTs FILTERS & NEED FOR AUTO VENT, ISOL IN NEW RADWASTE BLDG | | | <u>COMPLETE</u> 10/24/80 |
| 42845 | OYSTER CREEK - ASLAB ORDER (ALAB-612) REGARDING GENERIC UNRESOLVED SAFETY ISSUES | | 1 | 10/30/80 |
| 07520 | OYSTER CREEK - CHARCOAL FILTER TS BASIS CHANGE | | 1 | 02/11/81 |
| 43325 | OYSTER CREEK - DEPOSITION ON OYSTER CREEK CYCLE 5 RELOAD | | 1 | 02/12/81 |
| 43084 | OYSTER CREEK - TECH. SPEC. CHANGE MODIFYING METHYL IODINE REMOVAL EFFICIENCY TEST & SGTs AIR FILTER FLOW DIST'N. | | 2 | 02/13/81 |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 00987 | OYSTER CREEK - RADWASTE SYSTEM MODIFICATION | | 1 | <u>TARGET</u> |
| 07776 | OYSTER CREEK - OPERATIONAL QUALITY ASSURANCE PLAN | | 1 | |
| 08652 | OYSTER CREEK - LIQUID/SOLID RADWASTE BUILDING SETTLEMENT | | 1 | |
| 42388 | OYSTER CREEK - T.S. CHANGE FOR RECENTLY INSTALLED FIRE PROTECTION EQUIPMENT (LIC. RESUBMITTING) | | 1 | |
| 43489 | OYSTER CREEK - REACTOR TRIP WITH "SPURIOUS" TRIPLE LOW WATER LEVEL SIGNAL | | 1 | |
| 43370 | OYSTER CREEK - TECH. SPEC. CHANGE REQUEST TO SUSPEND CERTAIN ENVIRONMENTAL MONITORING REQUIREMENTS FOR 2 YEARS | | 2 | |

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TECHNICAL ASSIGNMENT CONTROL SYSTEM

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OYSTER CREEK 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------|---|-----------------------|-----------------|----------------------|
| | <u>ACTIVE ACTIONS</u> | <u>(CONTINUATION)</u> | | <u>TARGET</u> |
| 11268 | OYSTER CREEK - FULL TERM OPERATING LICENSE | | 1 | |
| 42646 | OYSTER CREEK - GENERAL ELECTRIC RELOAD FUEL APPLICATION | | 1 | |
| 43588 | OYSTER CREEK - NEW CORE SPRAY SPARGER DESIGN AND PIPING MODIFICATIONS | | 1 | |
| 42613 | OYSTER CREEK - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: PALISADES

PLANT LOCATION: 5 MI S OF SOUTH HAVEN, MI
 DOCKET NUMBER: 050-00255
 ARCH/ENGINEER: BECH
 IE INSPECTOR: B. JORGENSEN

LICENSED POWER: 2530 MW
 DESIGN POWER: 0805 MWE
 NSS VENDOR: COMB

PROJECT MANAGER: T. WAMBACH
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 08627 | PALISADES - RELOAD | | 1 | <u>COMPLETE</u> 10/01/80 |
| 10413 | PALISADES - CONTINGENCY PLAN REVIEW | | 1 | 10/01/80 |
| 12700 | PALISADES - ECCS CLAD SWELLING & RUPTURE | | 2 | 10/01/80 |
| 11982 | PALISADES - AUXILIARY FEEDWATER TECH SPECS | | 1 | 10/20/80 |
| 11101 | PALISADES - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/10/81 |
| 12235 | PALISADES - INSTALLATION SCHEDULE FOR ALTERNATE SHUTDOWN PANAL | | 1 | 02/10/81 |
| 12292 | PALISADES - HELD AND CONSEQUENTIAL SYSTEM FAILURE PALISADES | | 3 | 02/18/81 |
| 10409 | PALISADES - GUARD TRAINING PLAN REVIEW | | 1 | 02/25/81 |
| 10190 | PALISADES - CONTAINMENT PURGE | | 1 | 03/02/81 |
| 10288 | PALISADES - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 03/02/81 |
| 12629 | PALISADES - EMERGENCY PLANNING & REVISIONS | | 1 | 04/01/81 |
| 12912 | PALISADES - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 11732 | PALISADES - TMI FOLLOW UP WORK | | 1 | <u>TARGET</u> |
| 12780 | PALISADES - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 42130 | PALISADES - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43052 | PALISADES - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43642 | PALISADES - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 11972 | PALISADES - STEAM GENERATOR REPLACEMENT HEARING | | 3 | |
| 01573 | PALISADES - FLOODING OF INTAKE STRUCTURE FROM MODERATE ENERGY PALISADES-FLOOD OF EQUIPMENT IMPORTANT TO SAFETY | | 2 | |
| 06779 | PALISADES - FILTER TECH SPECS | | 3 | |
| 01081 | PALISADES - FUEL CASK DROP ANALYSIS | | 3 | |
| 08622 | PALISADES - DEGRADED GRID VOLTAGE | | 2 | |
| 11214 | PALISADES - INSERVICE TESTING (ST) | | 1 | |
| 08148 | PALISADES - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08804 | PALISADES - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08031 | PALISADES - PWR PUMP & S/G SUPPORT - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 08625 | PALISADES - REVIEW MATERIAL SURVEILLANCE RESULTS | | 3 | |
| 10244 | PALISADES - INADVERTENT SAFETY INJECTION DURING COOLDOWN (PWR'S) | | 2 | |
| 10350 | PALISADES - LOSS OF 125-V BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08033 | PALISADES - BORON SOLUBILITY DURING LONG TERM COOLING FOLLOWING LOCA | | 3 | |
| 08621 | PALISADES - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08041 | PALISADES - STEAM GENERATOR FW FLO INSTABILITY | | 2 | |
| 10411 | PALISADES - VITAL AREA ANALYSIS | | 1 | |
| 08626 | PALISADES - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 08805 | PALISADES - TECH SPECS FOR MECHANICAL SNUBBERS | | 2 | |
| 12550 | PALISADES - QUALITY ASSURANCE REQUIREMENTS REGARDING DIESEL GENERATOR FUEL OIL | | 3 | |
| 07681 | PALISADES - RPS CHANNEL INOPERABILITY | | 3 | |
| 10943 | PALISADES - CEA POSITION INDICATOR CHANNELS | | 3 | |
| 10944 | PALISADES - CE GENERIC RTD RESPONSE TIME | | 3 | |
| 10677 | PALISADES - STEAM GENERATOR REPLACEMENT | | 3 | |
| 12586 | PALISADES - #1 ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42693 | PALISADES - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |

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(CONTINUATION)

PALISADES

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 42764 | PALISADES - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | TARGET |
| 43906 | PALISADES - STATION BLACKOUT PROCEDURES & TRAINING | | 2 | |
| 42915 | PALISADES - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43144 | PALISADES - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 1 | |
| 42517 | PALISADES - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|------------------------|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44044 | PALISADES - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44113 | PALISADES - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44184 | PALISADES - NUREG-0737 | I.A.2.1.1, UPGRADING OF RO AND SRO TRAINING | | |
| 44255 | PALISADES - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44325 | PALISADES - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44394 | PALISADES - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44463 | PALISADES - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44534 | PALISADES - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44604 | PALISADES - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44666 | PALISADES - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44706 | PALISADES - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44747 | PALISADES - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44802 | PALISADES - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44873 | PALISADES - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44944 | PALISADES - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45015 | PALISADES - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45086 | PALISADES - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45156 | PALISADES - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45265 | PALISADES - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45311 | PALISADES - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45366 | PALISADES - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45432 | PALISADES - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45552 | PALISADES - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45849 | PALISADES - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45968 | PALISADES - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46039 | PALISADES - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46111 | PALISADES - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46183 | PALISADES - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46255 | PALISADES - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46327 | PALISADES - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46398 | PALISADES - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46467 | PALISADES - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12786 | PALISADES - FIRE PROTECTION EQUIPMENT T.S. | | 1 | COMPLETE 10/01/80 |
| 42310 | PALISADES - T.S. CHANGE LIQUID EFFLUENTS AND ENVIRONMENTAL MONITORING | | 1 | 01/22/81 |
| 12442 | PALISADES - OPEN CONTAINMENT EXHAUST BYPASS LINE | | 1 | 02/27/81 |

ACTIVE ACTIONSTARGET

(CONTINUATION)

PALISADES

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY CRITICAL DATE

| TAC # | TAC DESCRIPTION | PRIORITY | CRITICAL DATE |
|----------------|--|----------|---------------|
| ACTIVE ACTIONS | (CONTINUATION) | | TARGET |
| 11909 | PALISADES - REVIEW COMMITTEE AND CORE ALTERATION SPECS | 2 | |
| 11934 | PALISADES - ADMIN. CONTROLS C.C.G.E. IN CPC DRG | 2 | |
| 42043 | PALISADES - ADM CHGS TO TECH SPEC DESCRIBED IN LETTER OF REQ DATED 5/14/80 | 2 | |
| 42320 | PALISADES - RELIEF REQUES FOR INSERVICE PUMP TESTING PROGRAM | 1 | |
| 43269 | PALISADES - ISI UPDATE REC XI 77578 ADDENDA DATED 11/12/80 | 1 | |
| 06366 | PALISADES - USE OF CE ABC UNPRESSURIZED FUEL | 1 | |
| 07077 | PALISADES - REQUIRED ACTION FOLLOWING EQUIPMENT FAILURE | 1 | |
| 07081 | PALISADES - CLARIFICATION OF SI PUMP TESTING REQUIREMENTS | 1 | |
| 07730 | PALISADES - HYDROGEN RECOMBINER TESTING REQUIREMENTS | 3 | |
| 07735 | PALISADES - SI TANK LEVEL SEC DISCREPANCY | 1 | |
| 07736 | PALISADES - REVISED TECH SPEC FOR HPCI PUMP | 2 | |
| 12478 | PALISADES - ADMINISTRATIVE CONTROLS CLARIFICATIONS | 2 | |
| 11644 | PALISADES - SEISMIC DESIGN ANALYSIS CONCERNS | 2 | |
| 11218 | PALISADES - FULL TERM OPERATING LICENSE | 2 | |
| 42614 | PALISADES - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (8-24) | 1 | |

ANTICIPATED ACTIONS

43970 PALISADES TECH SPEC CHG STEM GENERATOR SURV

INITIATION
06/01/81

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: PEACH BOTTOM 2

PLANT LOCATION: 19 MI S OF LANCASTER, PA
 DOCKET NUMBER: 050-00277
 ARCH/ENGINEER: BECH
 IE INSPECTOR: C COWGILL

LICENSED POWER: 3293 MWT
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: M. FAIRTILE
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42248 | PEACH BOTTOM 2 - L2 CAT A T. SPECS (#74) | | 1 | <u>COMPLETE</u> 10/28/80 |
| 10173 | PEACH BOTTOM 2 - CONTAINMENT PURGE (LTR) | | 1 | 11/05/80 |
| 10442 | PEACH BOTTOM 2 - CONTINGENCY PLAN REVIEW | | 1 | 04/13/81 |
| 11084 | PEACH BOTTOM 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 05/27/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12156 | PEACH BOTTOM 2 - UPGRADED EMERGENCY PLAN | | 1 | <u>TARGET</u> |
| 13107 | PEACH BOTTOM 2 - ADEQUACY OF DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 07010 | PEACH BOTTOM 2 - ISI | | 1 | |
| 10040 | PEACH BOTTOM 2 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 08935 | PEACH BOTTOM 2 - RPS POWER SUPPLY | | 1 | |
| 11258 | PEACH BOTTOM 2 - IST | | 1 | |
| 10366 | PEACH BOTTOM 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08044 | PEACH BOTTOM 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 10447 | PEACH BOTTOM 2 - VITAL AREA ANALYSIS | | 1 | |
| 10451 | PEACH BOTTOM 2 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 07945 | PEACH BOTTOM 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 42217 | PEACH BOTTOM 2 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 42765 | PEACH BOTTOM 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 08501 | PEACH BOTTOM 2 - SURV. OF MECHANICAL SNUBBERS | | 2 | |
| 43931 | PEACH BOTTOM 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43728 | PEACH BOTTOM 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 08503 | PEACH BOTTOM 2 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 42922 | PEACH BOTTOM 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42503 | PEACH BOTTOM 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42157 | PEACH BOTTOM 2 - L2 B - RCS VENT | | 1 | <u>COMPLETE</u> 04/12/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44045 | PEACH BOTTOM 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44114 | PEACH BOTTOM 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44185 | PEACH BOTTOM 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44256 | PEACH BOTTOM 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44326 | PEACH BOTTOM 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44464 | PEACH BOTTOM 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44535 | PEACH BOTTOM 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44605 | PEACH BOTTOM 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44803 | PEACH BOTTOM 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44874 | PEACH BOTTOM 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44945 | PEACH BOTTOM 2 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45016 | PEACH BOTTOM 2 - NUREG-0737 II.F.1.2, IODINE/PARTICULATE SAMPLING | | | |
| 45087 | PEACH BOTTOM 2 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PEACH BOTTOM 2

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
|--------------------------|--|---|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 45157 | PEACH BOTTOM 2 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45367 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45549 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45577 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45601 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45653 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45697 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45721 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45745 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45769 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45793 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45850 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45893 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45917 | PEACH BOTTOM 2 - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45969 | PEACH BOTTOM 2 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46040 | PEACH BOTTOM 2 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46112 | PEACH BOTTOM 2 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46184 | PEACH BOTTOM 2 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46256 | PEACH BOTTOM 2 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46328 | PEACH BOTTOM 2 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46399 | PEACH BOTTOM 2 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46468 | PEACH BOTTOM 2 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| 42694 | PEACH BOTTOM 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | | 03/20/81 |
| <u>COMPLETED ACTIONS</u> | | | | |
| 07072 | PEACH BOTTOM 2 - APRM ROD BLOCK AND SCRAM | | | COMPLETE |
| 42361 | PEACH BOTTOM 2 - II.E.4.1 DEDICATED PENETRATIONS | | | 10/16/80 |
| 42197 | PEACH BOTTOM 2 - SEISMIC INSTRUMENTATION (#75) | | | 10/29/80 |
| 42974 | PEACH BOTTOM 2 - REVISE RPS RESPONSE TIME (#76) | | | 11/19/80 |
| 43074 | PEACH BOTTOM 2 - LONG TERM STA (I.A.1.1) | | | 12/10/80 |
| 42391 | PEACH BOTTOM 2 - II.K.3.46 MICHELSON CONCERNS | | | 12/17/80 |
| 43302 | PEACH BOTTOM 2 - SINGLE LOOP OPN | | | 04/12/81 |
| 42976 | PEACH BOTTOM 2 - REVISE MAPLHGR | | | 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 13281 | PEACH BOTTOM 2 - DC POWER SUPPLY/ECCS PERFORMANCE | | | |
| 41021 | PEACH BOTTOM 2 - LP TURBINE CRACKING | | | |
| 41040 | PEACH BOTTOM 2 - ADM CHANGES | | | |
| 42064 | PEACH BOTTOM 2 - H.P. ORGN | | 2 | |
| 43750 | PEACH BOTTOM 2 - VACUUM BREAKERS BETWEEN TORUS AND REACTOR BLD'G. T.S. CHANGE | | 1 | |
| 43709 | PEACH BOTTOM 2 - REQUEST FOR MODIFICATION OF SCRAM DISCHARGE ORDERS | | 1 | |
| 43956 | PEACH BOTTOM 2 - PREPARATION OF ORDERS CONFIRMING SUBMITTAL DATES THRU 6/30/81 (NUREG 0737) | | 1 | |
| 42605 | PEACH BOTTOM 2 - LONG TERM REV CONTAINMENT PURGE AND VENT (B-24) | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: PEACH BOTTOM 3

PLANT LOCATION: 19 MI S OF LANCASTER, PA
 DOCKET NUMBER: 050-00278
 ARCH/ENGINEER: BECH
 IE INSPECTOR: C. COWGILL

LICENSED POWER: 3293 MWT
 DESIGN POWER: 1065 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: M. FAIRTILE
 BRANCH CHIEF: J. STOLZ
 LIC. ASSISTANT: R. INGRAM

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|---|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42247 | PEACH BOTTOM 3 - L2 CAT A T. SPECS (#73) | | 1 | <u>COMPLETE</u> 10/28/80 |
| 10172 | PEACH BOTTOM 3 - CONTAINMENT PURGE (LTR) | | 1 | 11/05/80 |
| 08936 | PEACH BOTTOM 3 - RPS POWER SUPPLY | | 1 | 01/21/81 |
| 10443 | PEACH BOTTOM 3 - CONTINGENCY PLAN REVIEW | | 1 | 04/13/81 |
| 11085 | PEACH BOTTOM 3 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 05/27/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12157 | PEACH BOTTOM 3 - UPGRADED EMERGENCY PLAN | | 1 | |
| 08504 | PEACH BOTTOM 3 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 10039 | PEACH BOTTOM 3 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 07289 | PEACH BOTTOM 3 - ISI (50.55A) | | 1 | |
| 11257 | PEACH BOTTOM 3 - IST | | 1 | |
| 10367 | PEACH BOTTOM 3 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08122 | PEACH BOTTOM 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 10446 | PEACH BOTTOM 3 - VITAL AREA ANALYSIS | | 1 | |
| 10453 | PEACH BOTTOM 3 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 07946 | PEACH BOTTOM 3 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 42216 | PEACH BOTTOM 3 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 42695 | PEACH BOTTOM 3 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42766 | PEACH BOTTOM 3 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 08500 | PEACH BOTTOM 3 - SURV. OF MECHANICAL SHUBBERS | | 2 | |
| 43932 | PEACH BOTTOM 3 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43732 | PEACH BOTTOM 3 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42923 | PEACH BOTTOM 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42504 | PEACH BOTTOM 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u> | | | | |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42158 | PEACH BOTTOM 3 - L2 B - RCS VENT | | 1 | <u>COMPLETE</u> 04/12/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44046 | PEACH BOTTOM 3 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44115 | PEACH BOTTOM 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44186 | PEACH BOTTOM 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44257 | PEACH BOTTOM 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44327 | PEACH BOTTOM 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44465 | PEACH BOTTOM 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44536 | PEACH BOTTOM 3 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44606 | PEACH BOTTOM 3 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44804 | PEACH BOTTOM 3 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44875 | PEACH BOTTOM 3 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44946 | PEACH BOTTOM 3 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45017 | PEACH BOTTOM 3 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PEACH BOTTOM 3

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|------------------------|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| (CONTINUATION) | | | | |
| 45088 | PEACH BOTTOM 3 | - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45158 | PEACH BOTTOM 3 | - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45368 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45550 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45578 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45602 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45654 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45698 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY | | |
| 45722 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45746 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45770 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.25, POWER ON PUMP SEALS | | |
| 45794 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45851 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45894 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45918 | PEACH BOTTOM 3 | - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45970 | PEACH BOTTOM 3 | - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46041 | PEACH BOTTOM 3 | - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46113 | PEACH BOTTOM 3 | - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46185 | PEACH BOTTOM 3 | - NUREG-0737 III.A.1.2, NUCLEAR DATA | | |
| 46257 | PEACH BOTTOM 3 | - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46329 | PEACH BOTTOM 3 | - NUREG-0737 III.A.1.2, METEOROLOGICAL DATA UPGRADE | | |
| 46400 | PEACH BOTTOM 3 | - NUREG-0737 III.B.3.5, INPLANT RADIATION MONITORING | | |
| 46469 | PEACH BOTTOM 3 | - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>PLANT SPECIFIC</u> | | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 07821 | PEACH BOTTOM 3 | - APRM ROD BLOCK AND SCRAM | 1 | 10/16/80 |
| 42362 | PEACH BOTTOM 3 | - II.E.4.1 DEDICATED PENETRATIONS | | 10/29/80 |
| 42196 | PEACH BOTTOM 3 | - SEISMIC INSTRUMENTATION (#74) | | 11/19/80 |
| 42975 | PEACH BOTTOM 3 | - REVISE RPS RESPONSE TIME (#75) | | 12/10/80 |
| 43075 | PEACH BOTTOM 3 | - LONG TERM STA (I.A.1.1) | | 12/17/80 |
| 42392 | PEACH BOTTOM 3 | - II.K.3.46 MICHELSON CONCERNS | | 04/12/81 |
| 43303 | PEACH BOTTOM 3 | - SINGLE LOOP OPN | 1 | 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 13108 | PEACH BOTTOM 3 | - ADEQUACY OF DISTRIBUTION SYSTEMS VOLTAGES | 1 | |
| 13282 | PEACH BOTTOM 3 | - DC POWER SUPPLY/ECCS PERFORMANCE | 1 | |
| 41022 | PEACH BOTTOM 3 | - LP TURBINE CRACKING | 1 | |
| 41041 | PEACH BOTTOM 3 | - ADM CHANGES | 1 | |
| 42065 | PEACH BOTTOM 3 | - H.P. ORGN | 2 | |
| 42977 | PEACH BOTTOM 3 | - REVISE MAPLHGR | 2 | |
| 43710 | PEACH BOTTOM 3 | - REQUEST FOR MODIFICATION OF SCRAM DISCHARGE ORDERS | 1 | |
| 43957 | PEACH BOTTOM 3 | - PREPARATION OF ORDERS CONFIRMING SUBMITTAL DATES THRU 6/30/81 (NUREG 0737) | 1 | |
| 43697 | PEACH BOTTOM 3 | - RELOAD NO. 4 REVIEW (CYCLE 5) | 1 | |
| 42606 | PEACH BOTTOM 3 | - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | 1 | |
| 46499 | PEACH BOTTOM 3 | - REPLACEMENT OF CARBON STEEL PIPE IN CRD FLOW STABILIZER LOOP | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: PILGRIM 1

PLANT LOCATION: 4 MI SE OF PLYMOUTH, MASS
 DOCKET NUMBER: 050-00293
 ARCH/ENGINEER: BECH
 IE INSPECTOR: J. JOHNSON

LICENSED POWER: 1998 MWT
 DESIGN POWER: 0655 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: M. WILLIAMS
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|---------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10696 | PILGRIM 1 - CONTINGENCY PLAN REVIEW | | 1 | 10/02/80 |
| 11077 | PILGRIM 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 12/15/80 |
| 11351 | PILGRIM 1 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE SURVEILLANCE | | 3 | 01/28/81 |
| 10791 | PILGRIM 1 - NUCLEAR POWER STATION CONVERSION TO STANDARD TECHNICAL SPECIFICATIONS | | 3 | 02/03/81 |
| 10700 | PILGRIM 1 - VITAL AREA ANALYSIS | | 1 | 03/02/81 |
| 07901 | PILGRIM 1 - DIESEL GENERATOR LOCKOUT | | 3 | 03/16/81 |
| 10692 | PILGRIM 1 - GUARD TRAINING PLAN REVIEW | | 1 | 03/24/81 |
| 11628 | PILGRIM 1 - PROPOSED TS CHGS TO COMPLY W 10 CFR 50 (50.55A (G)(5)(II)) | | 1 | 03/31/81 |
| 12141 | PILGRIM 1 - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 08836 | PILGRIM 1 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | 04/28/81 |
| 42209 | PILGRIM 1 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | 05/05/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08325 | PILGRIM 1 - 10 CFR 50.55A (G)-ISI - GENERIC/WELDS/PIPING | | 1 | TARGET |
| 10365 | PILGRIM 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 43614 | PILGRIM 1 - FIRE PROTECTION - APPENDIX R, EXEMPTION REQUEST | | 1 | |
| 43847 | PILGRIM 1 - BWR SCRAM DISCHARGE SYSTEM TECHNICAL SPECIFICATION CHANGES | | 1 | |
| 43848 | PILGRIM 1 - BWR SCRAM DISCHARGE SYSTEM-LONG TERM FIX TO SDV-IV COUPLING | | 1 | |
| 08830 | PILGRIM 1 - TS SURVEILLANCE FOR MECHANICAL SNUBBERS | | 2 | |
| 08831 | PILGRIM 1 - TS SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 10831 | PILGRIM 1 - CONTAINMENT ATMOSPHERIC DILUTION SYSTEM | | 3 | |
| 08123 | PILGRIM 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 12185 | PILGRIM 1 HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE | | 2 | |
| 10817 | PILGRIM 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 11034 | PILGRIM 1 - 50.55A(G) 1ST PUMPS/VALVES | | 1 | |
| 42696 | PILGRIM 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42767 | PILGRIM 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 41073 | PILGRIM 1 - CONTROL RODS FAIL TO FULLY INSERT | | 3 | |
| 43890 | PILGRIM UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43726 | PILGRIM 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM. | | | |
| 10825 | PILGRIM 1 - BWR SINGLE LOOP OPERATION | | 3 | |
| 41076 | PILGRIM 1 - CLARIFY OPERABLE AS IT APPLIES TO SINGLE FAILURE | | 2 | |
| 42885 | PILGRIM 1 - MASONRY WALL DESIGN. RESPONSE TO I BULLETIN 80-11 | | 2 | |
| 07948 | PILGRIM 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 42579 | PILGRIM 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 08937 | PILGRIM 1 - RPS POWER SUPPLY | | 1 | |
| 08841 | PILGRIM 1 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | |
| 42477 | PILGRIM 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| TAC # | TAC DESCRIPTION | IMI ACTIONS | PRIORITY | CRITICAL DATE |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44047 | PILGRIM 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | TARGET |
| 44116 | PILGRIM 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44187 | PILGRIM 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PILGRIM 1

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|---|---|-----------------|-------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44258 | PILGRIM 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44328 | PILGRIM 1 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44395 | PILGRIM 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44466 | PILGRIM 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44537 | PILGRIM 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44607 | PILGRIM 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44805 | PILGRIM 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44876 | PILGRIM 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44947 | PILGRIM 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45018 | PILGRIM 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45089 | PILGRIM 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45159 | PILGRIM 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45369 | PILGRIM 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45551 | PILGRIM 1 - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45579 | PILGRIM 1 - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45603 | PILGRIM 1 - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45655 | PILGRIM 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45699 | PILGRIM 1 - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45723 | PILGRIM 1 - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45747 | PILGRIM 1 - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45771 | PILGRIM 1 - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45795 | PILGRIM 1 - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45852 | PILGRIM 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45895 | PILGRIM 1 - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45919 | PILGRIM 1 - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45971 | PILGRIM 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46042 | PILGRIM 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46114 | PILGRIM 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46186 | PILGRIM 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46258 | PILGRIM 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46330 | PILGRIM 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46401 | PILGRIM 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46470 | PILGRIM 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>PLANT SPECIFIC</u> | | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42926 | PILGRIM 1 - TECH SPEC CHANGE CONCERNING DEFINITION OF HIGH RADIATION AREA | | 2 | COMPLETE 12/10/80 |
| 42965 | PILGRIM 1 - ORGANIZATIONAL CHANGE OF SEPT 1, 1980 | | 2 | 02/02/81 |
| 41082 | PILGRIM 1 - ATWS/ARI DESIGN: REVIEW DIFFERENCES WRT MONTICELLO | | 1 | 02/19/81 |
| 07665 | PILGRIM 1 - SERVICE WATER INTAKE STRUCTURE BREAKWATER (BW) | | 1 | 03/30/81 |
| 43230 | PILGRIM 1 - CAT A TECH. SPEC CHANGES | | 1 | 04/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 41079 | PILGRIM 1 - DIESEL GENERATOR RELIABILITY | | 1 | TARGET |
| 42945 | PILGRIM 1 - EFFECT OF A DC POWER SUPPLY FAILURE ON ECCS PERFORMANCE | | 1 | |
| 43804 | PILGRIM 1 - APPENDIX J-TECHNICAL SPECIFICATION CHANGE | | | |
| 43414 | PILGRIM 1 - PERIODIC UPDATING OF FSAR - 10 CFR 50.71(E) | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: POINT BEACH 1

PLANT LOCATION: 15 MI N OF MANITOWOC, WISC
 DOCKET NUMBER: 050-00266
 ARCH/ENGINEER: BECH
 IE INSPECTOR: J. SMITH

LICENSED POWER: 1518 MWT
 DESIGN POWER: 0497 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: T. COLBURN
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11872 | POINT BEACH 1 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | <u>COMPLETE</u> 10/21/80 |
| 08662 | POINT BEACH 1 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 11404 | POINT BEACH 1 - HEAVY LOAD HANDLING OVER SPENT FUEL | | 2 | 01/01/81 |
| 11079 | POINT BEACH 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/13/81 |
| 10143 | POINT BEACH 1 - MECHANICAL SNUBBERS | | 2 | 03/13/81 |
| 07708 | POINT BEACH 1 - GENERIC - DEGRADED GRID VOLTAGE | | 2 | 03/31/81 |
| 12632 | POINT BEACH 1 - EMERGENCY PLAN | | 1 | 04/01/81 |
| 12930 | POINT BEACH 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 10557 | POINT BEACH 1 - CONTINGENCY PLAN REVIEW | | 1 | 04/30/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 06828 | POINT BEACH 1 - UPI ECCS INJECTION | | 3 | <u>TARGET</u> |
| 07712 | POINT BEACH 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 10331 | POINT BEACH 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS ANNUNCIATOR SYSTEM | | 3 | |
| 10553 | POINT BEACH 1 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 11300 | POINT BEACH 1 - HYDRAULIC SNUBBERS-UPGRADE TECH SPECS | | 3 | |
| 11692 | POINT BEACH 1 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | |
| 12271 | POINT BEACH 1 - ENVIRON. QUAL. OF CONTROL SYSTEMS POINT BEACH 1 | | 3 | |
| 43669 | POINT BEACH 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 11243 | POINT BEACH 1 - INSERVICE TESTING(IST) | | 1 | |
| 07727 | POINT BEACH 1 - GENERIC - HEAVY LOADS NEAR SPENT FUEL (A36) | | 2 | |
| 10560 | POINT BEACH 1 - VITAL AREA ANALYSIS | | 1 | |
| 07710 | POINT BEACH 1 - ASYMMETRIC LOCA LOADS (GENERIC) | | 2 | |
| 06832 | POINT BEACH 1 - INSERVICE INSPECTION (ISI) REQUESTS OF 5-20-77 | | 1 | |
| 12584 | POINT BEACH 1 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42697 | POINT BEACH 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42768 | POINT BEACH 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43031 | POINT BEACH 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 42114 | POINT BEACH 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43944 | POINT BEACH UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | 2 | |
| 07711 | POINT BEACH 1 - GENERIC - APPENDIX J | | 2 | |
| 12955 | POINT BEACH 1 - ADEQUACY OF STATION ELECTRIC DISTR SYS VOLTAGES | | 2 | |
| 42896 | POINT BEACH 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43145 | POINT BEACH 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42590 | POINT BEACH 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42498 | POINT BEACH 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44048 | POINT BEACH 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44117 | POINT BEACH 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44188 | POINT BEACH 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

POINT BEACH 1

| TAC # | TAC DESCRIPTION | IMI ACTIONS | PRIORITY | CRITICAL DATE |
|-----------------------|----------------------------|---|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | (CONTINUATION) | | <u>TARGET</u> |
| 44259 | POINT BEACH 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44329 | POINT BEACH 1 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44396 | POINT BEACH 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44467 | POINT BEACH 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44538 | POINT BEACH 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44608 | POINT BEACH 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44645 | POINT BEACH 1 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44686 | POINT BEACH 1 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44724 | POINT BEACH 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44806 | POINT BEACH 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44877 | POINT BEACH 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44948 | POINT BEACH 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45019 | POINT BEACH 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45090 | POINT BEACH 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45160 | POINT BEACH 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45242 | POINT BEACH 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45280 | POINT BEACH 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45370 | POINT BEACH 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45405 | POINT BEACH 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45454 | POINT BEACH 1 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45484 | POINT BEACH 1 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45514 | POINT BEACH 1 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45656 | POINT BEACH 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45853 | POINT BEACH 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45972 | POINT BEACH 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46043 | POINT BEACH 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46115 | POINT BEACH 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46187 | POINT BEACH 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46259 | POINT BEACH 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46331 | POINT BEACH 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46402 | POINT BEACH 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46471 | POINT BEACH 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|--------------------------|---|----------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 43761 | POINT BEACH 1 - SPRAY ADDITIVE VALVE TESTING | | | 12/17/80 |
| 43401 | POINT BEACH 1 - WISCONSIN PSC REQUEST FOR INFORMATION DATED 1/16/81 | | | 02/11/81 |
| 42854 | POINT BEACH 1 - REVIEW SG TUBE INSPECTION RESULTS - NOV 1980 | | | 02/13/81 |
| 43088 | POINT BEACH 1 - GENERAL SURVEILLANCE | | | 02/28/81 |
| 43487 | POINT BEACH 1 - RESPOND TO WSPC LTR OF 2-23-81 | | | 03/04/81 |
| 43514 | POINT BEACH 1 - BORON DILUTION | | 1 | 04/29/81 |
| 12663 | POINT BEACH 1 - F DELTA H/ROD ALIGNMENT T.S. | | 2 | 05/04/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 42637 | POINT BEACH 1 - CONTAINMENT SPRAY ACTUATION LOGIC | | 1 | |
| 07757 | POINT BEACH 1 - REVIEW NON-RAD ENV. SURV. PGM 5-YEAR SUMMARY REPORT | | | |
| 42249 | POINT BEACH 1 - INSTRUMENT POWER SUPPLY MODIFICATION | | 2 | |
| 43742 | POINT BEACH 1 - ADMINISTRATIVE CHANGES TO TECHNICAL SPECIFICATIONS | | 2 | |
| 43744 | POINT BEACH 1 - PRESSURE - TEMPERATURE OPERATING CURVE REVISION | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

POINT BEACH 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | (CONTINUATION) | | | <u>TARGET</u> |
| 43492 | POINT BEACH 1 - REVIEW S/G TUBE INSPECTION RESULTS JULY 1981 | | 1 | |
| 43600 | POINT BEACH 1 - FIRE PROTECTION APPENDIX R | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: POINT BEACH 2

PLANT LOCATION: 15 MI N OF MANITOWOC, WISC
 DOCKET NUMBER: 050-06301
 ARCH/ENGINEER: BECH
 IE INSPECTOR: J. SMITH

LICENSED POWER: 1518 MWT
 DESIGN POWER: 0497 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: T. COLBURN
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11873 | POINT BEACH 2 - REVUEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE 1, TING | | 2 | <u>COMPLETE</u> 10/21/80 |
| 08663 | POINT BEACH 2 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 11080 | POINT BEACH 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/11/81 |
| 10142 | POINT BEACH 1- TECH SPEC SURVEILLANCY REQ FOR MECHANICAL SNUBBERS | | 2 | 03/14/81 |
| 10043 | POINT BEACH 2 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | 03/31/81 |
| 10139 | POINT BEACH 2 - SG AND RCP SUPPORTS - POTENTIAL FOR LAMELLAR TEARING | | 2 | 03/31/81 |
| 12633 | POINT BEACH 2 - EMERGENCY PLAN | | 1 | 04/01/81 |
| 12931 | POINT BEACH 2 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 10556 | POINT BEACH 2 - CONTINGENCY PLAN REVIEW | | 1 | 04/30/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08150 | POINT BEACH 2 - GENERIC - APPENDIX I | | 3 | <u>TARGET</u> |
| 08349 | POINT BEACH 2 - UPI ECCS INJECTION | | 3 | |
| 10332 | POINT BEACH 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10554 | POINT BEACH 2 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 11321 | POINT BEACH 2 - HYDRAULIC SNUBBERS - UPGRADE TECH SPECS | | 3 | |
| 11693 | POINT BEACH 2 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | |
| 12272 | POINT BEACH 2 - ENVIRON. QUAL. OF CONTROL SYSTEMS POINT BEACH 2 | | 3 | |
| 12956 | POINT BEACH 2 - ADEQUACY OF STATION ELECTRIC DISTR SYS VOLTAGES | | 2 | |
| 43670 | POINT BEACH 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 08322 | POINT BEACH 2 - INSERVICE INSPECTION (ISI) | | 1 | |
| 08074 | POINT BEACH 2 - GENERIC - HEAVY LOADS NEAR SPENT FUEL (A36) | | 2 | |
| 10559 | POINT BEACH 2 - VITAL AREA ANALYSIS | | 1 | |
| 11244 | POINT BEACH 2 - INSERVICE TESTING (IST) | | 1 | |
| 08156 | POINT BEACH 2 - GENERIC - ASYMMETRIC LOCA LOADS | | 2 | |
| 12585 | POINT BEACH 2 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42698 | POINT BEACH 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42769 | POINT BEACH 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42115 | POINT BEACH 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43032 | POINT BEACH 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43945 | POINT BEACH UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 08404 | POINT BEACH 2 - GENERIC - APPENDIX J | | 2 | |
| 42897 | POINT BEACH 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43146 | POINT BEACH 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42591 | POINT BEACH 2 - LONG TERM REVIEW CONTAINMENT PURGE AND PURGE AND VENT (B-24) | | 1 | |
| 42499 | POINT BEACH 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44049 | POINT BEACH 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44118 | POINT BEACH 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44189 | POINT BEACH 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

POINT BEACH 2

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
|--------------------------|------------------------|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | (CONTINUATION) | | <u>TARGET</u> |
| 44260 | POINT BEACH 2 | - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44330 | POINT BEACH 2 | - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44397 | POINT BEACH 2 | - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | |
| 44468 | POINT BEACH 2 | - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44539 | POINT BEACH 2 | - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44609 | POINT BEACH 2 | - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44646 | POINT BEACH 2 | - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44687 | POINT BEACH 2 | - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44725 | POINT BEACH 2 | - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44807 | POINT BEACH 2 | - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44878 | POINT BEACH 2 | - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44949 | POINT BEACH 2 | - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | |
| 45020 | POINT BEACH 2 | - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45091 | POINT BEACH 2 | - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45161 | POINT BEACH 2 | - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45243 | POINT BEACH 2 | - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | |
| 45289 | POINT BEACH 2 | - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | |
| 45371 | POINT BEACH 2 | - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45406 | POINT BEACH 2 | - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | |
| 45455 | POINT BEACH 2 | - NUREG-0737 II.K.3.9, PID CONTROLLER | | |
| 45485 | POINT BEACH 2 | - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45515 | POINT BEACH 2 | - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45657 | POINT BEACH 2 | - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45854 | POINT BEACH 2 | - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45973 | POINT BEACH 2 | - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46044 | POINT BEACH 2 | - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46116 | POINT BEACH 2 | - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46188 | POINT BEACH 2 | - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | |
| 46260 | POINT BEACH 2 | - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46332 | POINT BEACH 2 | - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46403 | POINT BEACH 2 | - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46472 | POINT BEACH 2 | - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 43762 | POINT BEACH 2 | - SPRAY ADDITIVE VALVE TESTING | | 12/17/80 |
| 11405 | POINT BEACH 2 | - HEAVY LOAD HANDLING OVER SPENT FUEL | | 01/01/81 |
| 43089 | POINT BEACH 2 | - GENERAL SURVEILLANCE | | 02/28/81 |
| 43607 | POINT BEACH 2 | - WAIVER OF MONTHLY TURBINE STOP VALVE TEST | | 04/03/81 |
| 43515 | POINT BEACH 2 | - BORON DILUTION | 1 | 04/29/81 |
| 12664 | POINT BEACH 2 | - F DELTA H/ROD ALIGNMENT T.S. | 2 | 05/04/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 08149 | POINT BEACH 2 | - ATWS (GENERIC) | | |
| 42063 | POINT BEACH 2 | - REVIEW MATERIALS SURVEILLANCE CAPSULE R TEST REPORT | 1 | |
| 08173 | POINT BEACH 2 | - REVIEW NON-RAD ENV. SURV. PROGRAM 5-YEAR SUMMARY REPORT | | |
| 42638 | POINT BEACH 2 | - CONTAINMENT SPRAY ACTUATION LOGIC | 1 | |
| 42250 | POINT BEACH 2 | - INSTRUMENT POWER SUPPLY MODIFICATION | 2 | |
| 43743 | POINT BEACH 2 | - ADMINISTRATIVE CHANGES TO TECHNICAL SPECIFICATIONS | 2 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

POINT BEACH 2

IAC # IAC DESCRIPTION

PLANT SPECIFIC

PRIORITY CRITICAL DATE

ACTIVE ACTIONS (CONTINUATION)

43745 POINT BEACH 2 - PRESSURE - TEMPERATURE OPERATING CURVE REVISION 1

TARGET

1

43601 POINT BEACH 2 - FIRE PROTECTION APPENDIX R

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: PRAIRIE ISLAND 1

PLANT LOCATION: 28 MI SE OF MINNEAPOLIS, MINN
 DOCKET NUMBER: 050-00282
 ARCH/ENGINEER: FPI
 IE INSPECTOR: C. FEIERABEND

LICENSED POWER: 1650 MWT
 DESIGN POWER: 0530 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. MARTIN
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11874 | PRAIRIE ISLAND 1 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | <u>COMPLETE</u> 10/23/80 |
| 08665 | PRAIRIE ISLAND 1 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 06336 | PRAIRIE ISLAND 1 - INSERVICE INSPECTION REQUIREMENTS (10 CFR 50.55A(G)) | | 1 | 11/14/80 |
| 08394 | PRAIRIE IS. 1 - GENERIC - PWR MODERATOR DILUTION | | 3 | 11/26/80 |
| 12273 | PRAIRIE ISLAND 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE PRAIRIE ISLAND 1 | | 3 | 12/01/80 |
| 10398 | PRAIRIE ISLAND 1 - CONTINGENCY PLAN REVIEW | | 1 | 02/25/81 |
| 10402 | PRAIRIE ISLAND 1 - GUARD TRAINING PLAN REVIEW | | 1 | 02/25/81 |
| 11387 | PRAIRIE ISLAND 1 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | 02/27/81 |
| 42116 | PRAIRIE ISLAND 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 04/01/81 |
| 12932 | PRAIRIE ISLAND 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 42770 | PRAIRIE ISLAND 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | 04/30/81 |
| 08752 | PRAIRIE IS. 1 - CONTROL ROD GUIDE TUBE WEAR | | 3 | 05/04/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 06250 | PRAIRIE ISLAND 1 - CONTAINMENT LEAK TESTING APPENDIX J APPENDIX J | | 2 | <u>TARGET</u> |
| 06853 | PRAIRIE ISLAND 1 - ECCS EVALUATION | | 1 | |
| 07614 | PRAIRIE ISLAND 1 - SECONDARY WATER CHEMICAL MONITORING REQUIREMENTS | | 2 | |
| 08151 | PRAIRIE ISLAND 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08351 | PRAIRIE ISLAND 1 - WESTINGHOUSE UPPER PLENUM ECCS INJECTION - LONG RANGE | | 3 | |
| 08734 | PRAIRIE ISLAND 1 - POTENTIAL EQUIPMENT FAILURE ASSOC. WITH DEG. GRID VOLT. | | 2 | |
| 08750 | PRAIRIE ISLAND 1 - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT | | 2 | |
| 08754 | PRAIRIE ISLAND 1 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 08756 | PRAIRIE ISLAND 1 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS | | 2 | |
| 10333 | PRAIRIE ISLAND 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 11094 | PRAIRIE ISLAND 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 11429 | PRAIRIE ISLAND 1 - 1ST PUMPS & VALVES | | 1 | |
| 11694 | PRAIRIE ISLAND 1 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | |
| 12348 | PRAIRIE ISLAND 1 - EMERGENCY PLANNING REVIEW 1 | | 1 | |
| 12966 | PRAIRIE ISLAND 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 13126 | PRAIRIE ISLAND 1 - EVALUATION REPORT FOR BULLETIN 79-064 RESPONSE | | 1 | |
| 43033 | PRAIRIE ISLAND 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43657 | PRAIRIE ISLAND 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 10400 | PRAIRIE ISLAND 1 - VITAL AREA ANALYSIS | | 1 | |
| 13038 | PRAIRIE ISLAND 1 - CONTROL ROD MISALIGNMENT TECH SPECS | | 2 | |
| 08731 | PRAIRIE ISLAND 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 12591 | PRAIRIE ISLAND 1 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 08075 | PRAIRIE ISLAND 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08748 | PRAIRIE ISLAND 1 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 42699 | PRAIRIE ISLAND 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43927 | PRAIRIE ISLAND UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42898 | PRAIRIE ISLAND 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43147 | PRAIRIE ISLAND 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42592 | PRAIRIE ISLAND 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PRAIRIE ISLAND 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|---|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 42500 | PRAIRIE ISLAND 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | <u>TARGET</u> |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 42541 | PRAIRIE ISLAND 1 - LESSONS LEARNED 2.1.7.A AND 2.1.7.6 AUTO INITIATION OF AFW | | 1 | <u>TARGET</u> |
| 44050 | PRAIRIE ISLAND 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44119 | PRAIRIE ISLAND 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44190 | PRAIRIE ISLAND 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44261 | PRAIRIE ISLAND 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44331 | PRAIRIE ISLAND 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44398 | PRAIRIE ISLAND 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44469 | PRAIRIE ISLAND 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44540 | PRAIRIE ISLAND 1 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44610 | PRAIRIE ISLAND 1 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44647 | PRAIRIE ISLAND 1 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44726 | PRAIRIE ISLAND 1 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44808 | PRAIRIE ISLAND 1 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44879 | PRAIRIE ISLAND 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44950 | PRAIRIE ISLAND 1 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45021 | PRAIRIE ISLAND 1 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45092 | PRAIRIE ISLAND 1 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45162 | PRAIRIE ISLAND 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45244 | PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45290 | PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45372 | PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45407 | PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45456 | PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.9, PID CONTROLLER | | | |
| 45486 | PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | | |
| 45516 | PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | | |
| 45658 | PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45855 | PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45974 | PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46045 | PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46117 | PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46189 | PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46261 | PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46333 | PRAIRIE ISLAND 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46404 | PRAIRIE ISLAND 1 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46473 | PRAIRIE ISLAND 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42341 | PRAIRIE ISLAND 1 - MASONRY WALLS (NRC 80-11) | | | <u>COMPLETE</u> 10/24/80 |
| 42036 | PRAIRIE ISLAND 1 - SI ACTUATION AND POWER DIST. LIMITS | | 2 | 12/17/80 |
| 12637 | PRAIRIE ISLAND 1 - AUX FEEDWATER SYSTEM TECH SPECS | | 1 | 01/16/81 |
| 12818 | PRAIRIE ISLAND 1 - SPENT FUEL POOL EXPANSION 2 | | 2 | 05/13/81 |
| 42352 | PRAIRIE ISLAND 1 - FIVE ADDITIONAL TMI-2 ITEMS | | 1 | 05/15/81 |

CONDENSED MANAGEMENT REPORT

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PRAIRIE ISLAND 1

(CONTINUATION)

| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------|--|-----------------|----------------------|
| | <u>PLANT SPECIFIC</u> | | <u>TARGET</u> |
| | ACTIVE ACTIONS | | |
| 06785 | PRAIRIE ISLAND 1 - MISCELLANEOUS CHANGE REQUEST 8-27-76 | 2 | |
| 06790 | PRAIRIE ISLAND 1 - LOCAL LEAKING ACCEPTANCE CRITERIA | 2 | |
| 07143 | PRAIRIE ISLAND 1 - LEAK RATE TEST CHANGE | 2 | |
| 12506 | PRAIRIE ISLAND 1 - PRESSURE VESSEL NOZZLE UNDERCLAD CRACKING POTENTIAL | 1 | |
| 42645 | PRAIRIE ISLAND 1 - FUEL SURVEILLANCE | 1 | |
| 12881 | PRAIRIE ISLAND 1 - REACTOR AND SECONDARY COOLANT ACTIVITY | 1 | |
| 43921 | PRAIRIE ISLAND 1 - APPENDIX B TECHNICAL SPECIFICATION | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: PRAIRIE ISLAND 2

PLANT LOCATION: 28 MI SE OF MINNEAPOLIS, MINN
 DOCKET NUMBER: 050-00306
 ARCH/ENGINEER: FPI
 IE INSPECTOR: C. FEIERABEND

LICENSED POWER: 1650 MWT
 DESIGN POWER: 0530 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. MARTIN
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10270 | PRAIRIE ISLAND 2 - CONTAINMENT LEAKAGE DUE TO SEAL TION | | 3 | <u>COMPLETE</u> 10/02/80 |
| 12684 | PRAIRIE ISLAND 2 - ECCS CLAD SWELLING AND RUPTURE | | 2 | 10/02/80 |
| 11875 | PRAIRIE ISLAND 2 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 10/23/80 |
| 08680 | PRAIRIE ISLAND 2 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 08395 | PRAIRIE IS. 2 - GENERIC - PWR MODERATOR DILUTION | | 3 | 11/26/80 |
| 12274 | PRAIRIE ISLAND 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE PRAIRIE ISLAND 2 | | 3 | 12/01/80 |
| 10399 | PRAIRIE ISLAND 2 - CONTINGENCY PLAN REVIEW | | 1 | 02/25/81 |
| 10403 | PRAIRIE ISLAND 2 - GUARD TRAINING PLAN REVIEW | | 1 | 02/25/81 |
| 08747 | PRAIRIE ISLAND 2 - PWR AUX. FEEDWATER PUMPS | | 1 | 03/05/81 |
| 08309 | PRAIRIE ISLAND 2 - 10 CFR 50.55A(G) - ISI ND TESTING - GENERIC | | 1 | 03/31/81 |
| 42117 | PRAIRIE ISLAND 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 04/01/81 |
| 12933 | PRAIRIE ISLAND 2 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 08753 | PRAIRIE IS. 2 - CONTROL ROD GUIDE TUBE WEAR | | 3 | 05/04/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08152 | PRAIRIE ISLAND 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | <u>TARGET</u> |
| 08471 | PRAIRIE ISLAND 2 - WESTINGHOUSE UPPER PLENUM ECCS INJECTION - LONG RANGE | | 3 | |
| 08729 | PRAIRIE ISLAND 2 - CONTAINMENT LEAK TESTING - APP J (GENERIC) | | 2 | |
| 08733 | PRAIRIE ISLAND 2 - SECONDARY WATER CHEM. MONITORING REQUIREMENTS | | 2 | |
| 08735 | PRAIRIE ISLAND 2 - POTENTIAL EQUIPMENT FAILURE ASSOC. WITH DEG. GRID. VOLT. | | 2 | |
| 08746 | PRAIRIE ISLAND 2 - ECCS EVALUATION | | 1 | |
| 08751 | PRAIRIE ISLAND 2 - FUEL HANDLING ACC INSIDE CONTAINMENT | | 2 | |
| 08755 | PRAIRIE ISLAND 2 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 08757 | PRAIRIE ISLAND 2 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS | | 2 | |
| 10334 | PRAIRIE ISLAND 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 11095 | PRAIRIE ISLAND 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 11330 | PRAIRIE ISLAND 2 - PURGING CONTAINMENT AT POWER AND EFFECT ON LOCA | | 1 | |
| 11695 | PRAIRIE ISLAND 2 - THREE MILE FOLLOW UP WORK | | 1 | |
| 12349 | PRAIRIE ISLAND 2 - EMERGENCY PLANNING REVIEW 2 | | 1 | |
| 12967 | PRAIRIE ISLAND 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 13127 | PRAIRIE ISLAND 2 - EVALUATION REPORT FOR BULLETIN 79-064 RESPONSE | | 1 | |
| 42622 | PRAIRIE ISLAND 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 43658 | PRAIRIE ISLAND 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 10401 | PRAIRIE ISLAND 2 - VITAL AREA ANALYSIS | | 1 | |
| 11223 | PRAIRIE ISLAND 2 - IST PUMPS AND VALVES | | 1 | |
| 13039 | PRAIRIE ISLAND 2 - CONTROL ROD MISALIGNMENT TECH SPECS | | 2 | |
| 08732 | PRAIRIE ISLAND 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 12592 | PRAIRIE ISLAND 2 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 08076 | PRAIRIE ISLAND 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08749 | PRAIRIE ISLAND 2 - PWR PUMPS & S/G SUPPORTS-LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 42700 | PRAIRIE ISLAND 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42771 | PRAIRIE ISLAND 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43928 | PRAIRIE ISLAND UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PRAIRIE ISLAND 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 42899 | PRAIRIE ISLAND 2 - MASONRY WALL DESIGN, RESPONSE TO IE BULLETIN 80-11 | | 2 | <u>TARGET</u> |
| 43148 | PRAIRIE ISLAND 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44051 | PRAIRIE ISLAND 2 - NUREG-0737 I.A.1.1 5'A LONG TERM REQUIREMENTS | | | <u>TARGET</u> |

| | | | | |
|-------|---|--|--|--|
| 44120 | PRAIRIE ISLAND 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44191 | PRAIRIE ISLAND 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44262 | PRAIRIE ISLAND 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44332 | PRAIRIE ISLAND 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44399 | PRAIRIE ISLAND 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44470 | PRAIRIE ISLAND 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44541 | PRAIRIE ISLAND 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44611 | PRAIRIE ISLAND 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44648 | PRAIRIE ISLAND 2 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44688 | PRAIRIE ISLAND 2 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |
| 44727 | PRAIRIE ISLAND 2 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44809 | PRAIRIE ISLAND 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44880 | PRAIRIE ISLAND 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44951 | PRAIRIE ISLAND 2 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45022 | PRAIRIE ISLAND 2 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45093 | PRAIRIE ISLAND 2 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45163 | PRAIRIE ISLAND 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45245 | PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45291 | PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45373 | PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45408 | PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45457 | PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.9, PID CONTROLLER | | | |
| 45487 | PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | | |
| 45517 | PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | | |
| 45659 | PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45856 | PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45975 | PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46046 | PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46118 | PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46190 | PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.3, NUCLEAR DATA DATA | | | |
| 46262 | PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46334 | PRAIRIE ISLAND 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46405 | PRAIRIE ISLAND 2 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46474 | PRAIRIE ISLAND 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12638 | PRAIRIE ISLAND 2 - AUX FEEDWATER SYSTEM TECH SPECS | | 1 | <u>COMPLETE</u> |
| 42355 | PRAIRIE ISLAND 2 - FIVE ADDITIONAL TMI-2 ITEMS ITEMS | | 1 | 01/16/81 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08736 | PRAIRIE ISLAND 2 - LOCAL LEAKAGE ACCEPTANCE CRITERIA | | 2 | <u>TARGET</u> |

TECHNICAL ASSIGNMENT CONTROL SYSTEM
CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PRAIRIE ISLAND 2

IAC # IAC DESCRIPTION

PLANT SPECIFIC

ACTIVE ACTIONS (CONTINUATION)

| | |
|-------|--|
| 08737 | PRAIRIE ISLAND 2 - MISCELLANEOUS CHANGE REQUEST 8-27-76 |
| 08738 | PRAIRIE ISLAND 2 - LEAK RATE TEST CHANGE |
| 12500 | PRAIRIE ISLAND 2 - PRESSURE VESSEL NOZZLE UNDERCLAD CRACKING POTENTIAL |
| 12657 | PRAIRIE ISLAND 2 - STEAM GENERATOR INSPECTION OF JAN 1980 OF JAN 1980 |
| 12482 | PRAIRIE ISLAND 2 - REACTOR AND SECONDARY COOLANT ACTIVITY COOLANT ACTIVITY |
| 12819 | PRAIRIE ISLAND 2 - SWEPT FUEL POOL EXPANSION 2 |

PRIORITY CRITICAL DATE

| | |
|---|--------|
| 2 | TARGET |
| 2 | |
| 1 | |
| 1 | |
| 1 | |
| 2 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: QUAD CITIES 1

PLANT LOCATION: 20 MI NE OF MOLINE, ILL
 DOCKET NUMBER: 050-00254
 ARCH/ENGINEER: S&L
 IE INSPECTOR: N. CHRISOITIMOS

LICENSED POWER: 2511 MWT
 DESIGN POWER: 0789 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. BEVAN
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12124 | QUAD CITIES 1 - EMERGENCY PLAN REVIEW | | 1 | <u>COMPLETE</u> 10/02/80 |
| 08554 | QUAD CITIES 1 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | 11/13/80 |
| 10666 | QUAD CITIES 1 - CONTINGENCY PLAN REVIEW | | 1 | 11/17/80 |
| 08933 | QUAD CITIES 1 - RPS POWER SUPPLY | | 1 | 12/11/80 |
| 07902 | QUAD CITIES 1 - DIESEL GENERATOR LOCKOUT | | 3 | 12/15/80 |
| 10671 | QUAD CITIES 1 - GUARD TRAINING PLAN REVIEW | | 1 | 12/15/80 |
| 08077 | QUAD CITIES 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | 12/22/80 |
| 08541 | QUAD CITIES 1 - COMMONWEALTH COMBUSTIBLE GAS CONTROL | | 3 | 01/15/81 |
| 11722 | QUAD CITIES 1 - TMI 2 FOLLOW-UP | | 1 | 02/01/81 |
| 11150 | QUAD CITIES 1 - MODIFIED AMENDED SECURITY PLAN | | 1 | 02/10/81 |
| 11069 | QUAD CITIES 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/12/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12310 | QUAD CITIES 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE QUAD CITIES 1 | | 3 | <u>TARGET</u> |
| 13230 | QUAD CITIES 1 - REVIEW OF CORPORATE CAPABILITIES | | 3 | |
| 13242 | QUAD CITIES 1 - GENERIC CLARIFICATION OF "OPERABILITY" | | 2 | |
| 07947 | QUAD CITIES 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 08124 | QUAD CITIES 1 - APP I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08666 | QUAD CITIES 1 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 42224 | QUAD CITIES 1 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 42701 | QUAD CITIES 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 42772 | QUAD CITIES 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43897 | QUAD-CITIES UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43723 | QUAD CITIES 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42886 | QUAD CITIES 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 08228 | QUAD CITIES 1 - STRESS CORROSION CRACKING - BWR RC5PB - GENERIC | | 1 | |
| 08329 | QUAD CITIES 1 - 10 CFR 50.55A(G) - INSERVICE INSPECTION - GENERIC | | 1 | |
| 08549 | QUAD CITIES 1 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS | | 2 | |
| 10047 | QUAD CITIES 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH DEGRADED GRID VOLTAGE | | 2 | |
| 10368 | QUAD CITIES 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10661 | QUAD CITIES 1 - VITAL AREA ANALYSIS | | 1 | |
| 11036 | QUAD CITIES 1 - BWR SINGLE LOOP OPERATION | | 3 | |
| 11280 | QUAD CITIES 1 - INSERVICE TESTING | | 1 | |
| 42478 | QUAD CITIES 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44121 | QUAD CITIES 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | <u>TARGET</u> |
| 44192 | QUAD CITIES 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRD TRAINING | | | |
| 44263 | QUAD CITIES 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44333 | QUAD CITIES 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44400 | QUAD CITIES 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44471 | QUAD CITIES 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

QUAD CITIES 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|--------------------------------------|---|---|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS (CONTINUATION)</u> | | | | |
| 44542 | QUAD CITIES 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44613 | QUAD CITIES 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44810 | QUAD CITIES 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44881 | QUAD CITIES 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44952 | QUAD CITIES 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45023 | QUAD CITIES 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45094 | QUAD CITIES 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45164 | QUAD CITIES 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45374 | QUAD CITIES 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45552 | QUAD CITIES 1 - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45580 | QUAD CITIES 1 - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45604 | QUAD CITIES 1 - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45660 | QUAD CITIES 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45700 | QUAD CITIES 1 - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45724 | QUAD CITIES 1 - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45748 | QUAD CITIES 1 - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45772 | QUAD CITIES 1 - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45796 | QUAD CITIES 1 - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45857 | QUAD CITIES 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45896 | QUAD CITIES 1 - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45920 | QUAD CITIES 1 - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45976 | QUAD CITIES 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46047 | QUAD CITIES 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46119 | QUAD CITIES 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46191 | QUAD CITIES 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46263 | QUAD CITIES 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46335 | QUAD CITIES 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46406 | QUAD CITIES 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46475 | QUAD CITIES 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| 43222 | QUAD CITIES 1 - SHIFT TECHNICAL ADVISOR | | 1 | 12/10/80 |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42382 | QUAD CITIES 1 - THREE CPS SRM COUNT RATE | | | <u>COMPLETE</u> |
| 42840 | QUAD CITIES 1 - 50.59 RELOAD - FALL 1980 | | 2 | 11/01/80 |
| 13088 | QUAD CITIES 1 - EXTENDED EXPOSURE MAPLHGR CURVES | | 2 | 11/01/80 |
| 06355 | QUAD CITIES 1 - OPERATION WITH ADS VALVE OUT OF SERVICE | | 1 | 11/06/80 |
| 42367 | QUAD CITIES 1 - MODEL TECH SPECS FOR IMPLEMENTATION OF TMI-2 CAT. A LL ITEMS | | 2 | 12/20/80 |
| 07653 | QUAD CITIES 1 - CECO CORRECTION TO HEAT DETECTOR LOCATION | | | 02/06/81 |
| 11580 | QUAD CITIES 1 - CORRECT LISTING OF SMOKE DETECTORS & SPRINKLERS | | 2 | 03/16/81 |
| 10959 | QUAD CITIES 1 - SUMP FLOW MONITORING REQUIREMENTS | | 2 | 03/16/81 |
| 06538 | QUAD CITIES 1 - INCREASE ECCS - ADS PRESSURE INTERLOCK SETPOINTS | | 2 | 03/27/81 |
| 07388 | QUAD CITIES 1 - OTHER PROPOSED CHANGES IN INSTRUMENT SETPOINTS | | 2 | 04/17/81 |
| 08038 | QUAD CITIES 1 - ELIMINATE REQUIREMENTS TO BE > 50% POWER FOR MSIV TESTING | | 2 | 04/17/81 |
| 11595 | QUAD CITIES 1 - ELIMINATE REQMT CONTINUOUS MONITORING FOR PC INERTING WYWTEM MAKEUP | | 2 | 04/20/81 |
| 07770 | QUAD CITIES 1 - RWCU SYSTEM ISOLATION VALVE SURVEILLANCE REQUIREMENTS | | 2 | 04/20/81 |
| 12489 | QUAD CITIES 1 - LOAD LINES LIMIT ANALYSIS | | 2 | 04/24/81 |
| 43321 | QUAD CITIES 1 - EXTENDED BURNUP OF MIXED OXIDE FUEL ASSEMBLY | | 2 | 04/30/81 |
| | | | 1 | 05/13/81 |

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TECHNICAL ASSIGNMENT CONTROL SYSTEM
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(CONTINUATION)

QUAD CITIES 1

| IAC # | IAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|-----------------------|--|----------------|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 12808 | QUAD CITIES 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM; VOLTAGES | | 1 | <u>TARGET</u> |
| 42373 | QUAD CITIES 1 - COMMONWEALTH EDISON CO USE OF ECONOMIC GENERATION CONTROL SYSTEM | | 3 | |
| 42597 | QUAD CITIES 1 - CURRENT EVENTS-ACTIVITIES RESULTING FROM DAILY PLANT OPERATION | | 1 | |
| 43619 | QUAD CITIES 1 - UPDATING OF FSAR | | 1 | |
| 42644 | QUAD CITIES 1 - ORGANIZATIONAL CHANGES - STATION AND CORPORATE | | 1 | |
| 07409 | QUAD CITIES 1 - REDUCE TIME REQUIRED TO PCILRI TEST | | 1 | |
| 07656 | QUAD CITIES 1 - INTER-FACILITY TRANSFER STORAGE OF SPENT FUEL BETWEEN/AT DRESDEN & QUAD CITIES | | 2 | |
| 43759 | QUAD CITIES 1 - SPENT FUEL POOL MODIFICATION | | 2 | |
| 42580 | QUAD CITIES 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 2 | |

ANTICIPATED ACTIONS

43622 QUAD CITIES 1 - DDYN CODE

INITIATION
02/01/82

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: QUAD CITIES 2

PLANT LOCATION: 20 MI NE OF MOLINE, ILL
 DOCKET NUMBER: 050-00265
 ARCH/ENGINEER: S&L
 IE INSPECTOR: N. CHRISSTIMOS

LICENSED POWER: 2511 MW
 DESIGN POWER: 0789 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: R. BEVAN
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. NORRIS

| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|---|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 12125 | QUAD CITIES 2 - EMERGENCY PLAN REVIEW | | 1 | <u>COMPLETE</u> 10/02/80 |
| 08553 | QUAD CITIES 2 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | 11/13/80 |
| 10667 | QUAD CITIES 2 - CONTINGENCY PLAN REVIEW | | 1 | 11/17/80 |
| 08934 | QUAD CITIES 2 - PPS POWER SUPPLY | | 1 | 12/11/80 |
| 07903 | QUAD CITIES 2 - DIESEL GENERATOR LOCKOUT | | 3 | 12/15/80 |
| 10672 | QUAD CITIES 2 - GUARD TRAINING PLAN REVIEW | | 1 | 12/15/80 |
| 08078 | QUAD CITIES 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | 12/22/80 |
| 08542 | QUAD CITIES 2 - COMMONWEALTH COMBUSTIBLE GAS CONTROL | | 3 | 01/15/81 |
| 11723 | QUAD CITIES 2 - TMI 2 FOLLOW-UP | | 1 | 02/01/81 |
| 11070 | QUAD CITIES 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/12/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12311 | QUAD CITIES 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE QUAD CITIES 2 | | 3 | <u>TARGET</u> |
| 13229 | QUAD CITIES 2 - REVIEW OF CORPORATE CAPABILITIES | | 3 | |
| 13241 | QUAD CITIES 2 - GENERIC CLARIFICATION OF "OPERABILITY" | | 2 | |
| 07949 | QUAD CITIES 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 08125 | QUAD CITIES 2 - APP I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 10369 | QUAD CITIES 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 42225 | QUAD CITIES 2 - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 08333 | QUAD CITIES 2 - 10 CFR 50.55A(G)-INSERVICE INSPECTION GENERIC | | 1 | |
| 42702 | QUAD CITIES 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 08667 | QUAD CITIES 2 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 42773 | QUAD CITIES 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 08547 | QUAD CITIES 2 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS | | 2 | |
| 43898 | QUAD-CITIES UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 43725 | QUAD CITIES 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 10046 | QUAD CITIES 2 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH DEGRADED GRID VOLTAGE | | 2 | |
| 42887 | QUAD CITIES 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 10662 | QUAD CITIES 2 - VITAL AREA ANALYSIS | | 1 | |
| 11279 | QUAD CITIES 2 - INSERVICE TESTING | | 1 | |
| 42479 | QUAD CITIES 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>IAC # IAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u> | | | | |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44122 | QUAD CITIES 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | <u>TARGET</u> |
| 44193 | QUAD CITIES 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44264 | QUAD CITIES 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44334 | QUAD CITIES 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44401 | QUAD CITIES 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44472 | QUAD CITIES 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44543 | QUAD CITIES 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44614 | QUAD CITIES 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44811 | QUAD CITIES 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

QUAD CITIES 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|---|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44882 | QUAD CITIES 2 - NUREG-0737 II.E.4.2, | CONTAINMENT ISOLATION DEPENDABILITY | | <u>TARGET</u> |
| 44953 | QUAD CITIES 2 - NUREG-0737 II.F.1.1, | NOBLE GAS MONITOR | | |
| 45024 | QUAD CITIES 2 - NUREG-0737 II.F.1.2, | IODINE/ PARTICULATE SAMPLING | | |
| 45095 | QUAD CITIES 2 - NUREG-0737 II.F.1.3, | CONTAINMENT HIGH RANGE MONITOR | | |
| 45165 | QUAD CITIES 2 - NUREG-0737 II.F.2.3, | INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45375 | QUAD CITIES 2 - NUREG-0737 II.K.3.3, | REPORT ON RV/SV FAILURES | | |
| 45553 | QUAD CITIES 2 - NUREG-0737 II.K.3.13, | HPCI AND RCIC INITIATION LEVELS | | |
| 45581 | QUAD CITIES 2 - NUREG-0737 II.K.3.15, | ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45605 | QUAD CITIES 2 - NUREG-0737 II.K.3.16, | CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45661 | QUAD CITIES 2 - NUREG-0737 II.K.3.17, | ECC SYSTEM OUTAGES | | |
| 45701 | QUAD CITIES 2 - NUREG-0737 II.K.3.18, | ADS ACTUATION STUDY | | |
| 45725 | QUAD CITIES 2 - NUREG-0737 II.K.3.19, | INTERLOC' RECIRC PUMP MODIFICATION | | |
| 45749 | QUAD CITIES 2 - NUREG-0737 II.K.3.21, | RESTART OF CSS AND LPCI | | |
| 45773 | QUAD CITIES 2 - NUREG-0737 II.K.3.25, | POWER ON PUMP SEALS | | |
| 45797 | QUAD CITIES 2 - NUREG-0737 II.K.3.27, | COMMON REFERENCE LEVEL | | |
| 45858 | QUAD CITIES 2 - NUREG-0737 II.K.3.30, | SB LOCA OUTLINE | | |
| 45897 | QUAD CITIES 2 - NUREG-0737 II.K.3.44, | TRANSIENTS WITH SINGLE FAILURES | | |
| 45921 | QUAD CITIES 2 - NUREG-0737 II.K.3.45, | MANUAL DEPRESSURIZATION | | |
| 45977 | QUAD CITIES 2 - NUREG-0737 III.A.1.2, | TECHNICAL SUPPORT CENTER | | |
| 46048 | QUAD CITIES 2 - NUREG-0737 III.A.1.2, | OPERATIONAL SUPPORT CENTER | | |
| 46120 | QUAD CITIES 2 - NUREG-0737 III.A.1.2, | EMERGENCY OPERATIONS FACILITY | | |
| 46192 | QUAD CITIES 2 - NUREG-0737 III.A.1.2, | NUCLEAR DATA DATA | | |
| 46264 | QUAD CITIES 2 - NUREG-0737 III.A.1.2, | EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46336 | QUAD CITIES 2 - NUREG-0737 III.A.2.2, | METEOROLOGICAL DATA UPGRADE | | |
| 46407 | QUAD CITIES 2 - NUREG-0737 III.D.3.3, | INPLANT RADIATION MONITORING | | |
| 46476 | QUAD CITIES 2 - NUREG-0737 III.D.3.4, | CONTROL ROOM HABITABILITY | | |
| 43223 | QUAD CITIES 2 - | SHIFT TECHNICAL ADVISOR | 1 | 12/10/80 |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42383 | QUAD CITIES 2 - THREE CPS SRM COUNT RATE | | 2 | <u>COMPLETE</u> |
| 13089 | QUAD CITIES 2 - EXTENDED EXPOSURE MAPLHGR CURVES | | 1 | 11/01/80 |
| 08575 | QUAD CITIES 2 - REQUIRE POWER DISTRIBUTION CHECK ONLY IN RUN MODE | | 1 | 11/06/80 |
| 08567 | QUAD CITIES 2 - OPERATION WITH ONE ADS VALVE OUT OF SERVICE | | 2 | 12/05/80 |
| 08573 | QUAD CITIES 2 - CECO CORRECTIONS TO HEAT DETECTOR LOCATIONS | | 2 | 12/20/80 |
| 10726 | QUAD CITIES 2 - CORRECT LISTING OF OPERABLE HEAT DETECTORS | | 2 | 03/16/81 |
| 11579 | QUAD CITIES 2 - CORRECT LISTING OF SMOKE DETECTORS & SPRINKLERS | | 2 | 03/16/81 |
| 08555 | QUAD CITIES 2 - SUMP FLOW MONITORING REQUIREMENTS | | 2 | 03/16/81 |
| 08570 | QUAD CITIES 2 - INCREASE ECCS-ADS PRESSURE INTERLOCK SET POINTS | | 2 | 03/27/81 |
| 08571 | QUAD CITIES 2 - OTHER PROPOSED CHANGES IN INSTRUMENT SET POINTS | | 2 | 04/17/81 |
| 08576 | QUAD CITIES 2 - ELIMINATE REQUIREMENTS TO BE > 50% POWER FOR MSIV TESTING | | 2 | 04/17/81 |
| 11596 | QUAD CITIES 2 - ELIMINATE REQMT CONTINUOUS MONITORING FOR PC INERTING SYSTEM MAKEUP | | 2 | 04/20/81 |
| 08572 | QUAD CITIES 2 - RWCU SYSTEM ISOLATION VALVE SURVEILLANCE REQUIREMENTS | | 2 | 04/20/81 |
| 12490 | QUAD CITIES 2 - LOAD LINE LIMIT ANALYSIS | | 2 | 04/21/81 |
| | | | | 04/30/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12809 | QUAD CITIES 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | | <u>TARGET</u> |
| 42374 | QUAD CITIES 2 - COMMONWEALTH EDISON CO USE OF ECONOMIC GENERATION CONTROL SYSTEM | | 3 | |
| 42998 | QUAD CITIES 2 - CURRENT EVENTS-ACTIVITIES RESULTING FROM DAILY PLANT OPERATION | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: RANCHO SECO 1

PLANT LOCATION: 25 MI SE OF SACRAMENTO, CA
 DOCKET NUMBER: 050-00312
 ARCH/ENGINEER: BECH
 IE INSPECTOR: H. CANTER

LICENSED POWER: 2772 MWT
 DESIGN POWER: 0918 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: M. PADOVAN
 BRANCH CHIEF: R. REID
 LIC. ASSISTANT: R. INGRAM

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|--|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 08599 | RANCHO SECO - REACTOR VESSEL ATYPICAL WELD MATERIAL | | | <u>COMPLETE</u> 10/29/80 |
| 11801 | RANCHO SECO - FEEDWATER LINE CRACKS | | 2 | 12/19/80 |
| 10563 | RANCHO SECO - CONTINGENCY PLAN REVIEW | | 1 | 04/13/81 |
| 10569 | RANCHO SECO - GUARD TRAINING PLAN REVIEW | | 1 | 04/13/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12154 | RANCHO SECO - UPGRADED EMERGENCY PLAN | | 1 | <u>TARGET</u> |
| 12326 | RANCHO SECO - HELB AND CONSEQUENTIAL SYSTEM FAILURE RANCHO SECO | | 3 | |
| 12746 | RANCHO SECO - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 42126 | RANCHO SECO - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43042 | RANCHO SECO - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43663 | RANCHO SECO - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 06710 | RANCHO SECO - REACTOR COOLANT SYSTEM OVERPRESSURIZATION PROTECTION | | 2 | |
| 11130 | RANCHO SECO - APPENDIX J - CONTAINMENT LEAK TESTING | | 2 | |
| 08609 | RANCHO SECO - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | |
| 08612 | RANCHO SECO - FILTER TECH SPECS | | 3 | |
| 08079 | RANCHO SECO - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 10321 | RANCHO SECO - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | |
| 10383 | RANCHO SECO - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08589 | RANCHO SECO - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10206 | RANCHO SECO - CONTAINMENT PURGE | | 1 | |
| 11288 | RANCHO SECO - INSERVICE TESTING | | 1 | |
| 11287 | RANCHO SECO - TENDON SURVEILLANCE | | 2 | |
| 10816 | RANCHO SECO - 10 CFR 50.55A(G) - INSERVICE INSPECTION - GENERIC | | 1 | |
| 11076 | RANCHO SECO - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 08153 | RANCHO SECO - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08602 | RANCHO SECO - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08605 | RANCHO SECO - MECHANICAL SNUBBER SURVEILLANCE | | 2 | |
| 11128 | RANCHO SECO - TS FOR HYDRAULIC SNUBBERS | | 3 | |
| 10968 | RANCHO SECO - DEGRADED GRID VOLTAGE | | 2 | |
| 11927 | RANCHO SECO - PUBLIC HEARINGS ON COMMISSION ORDER OF MAY 7, 1979 | | 1 | |
| 08247 | RANCHO SECO - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | |
| 12604 | RANCHO SECO - ANALYSIS OF TURBINE DISC CRACKS | | 3 | |
| 10849 | RANCHO SECO - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 42703 | RANCHO SECO - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 42774 | RANCHO SECO - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43264 | RANCHO SECO - B&W ACCIDENT INDUCED NEUTRON FLUX ERRORS (MULTI-PLANT ITEM B-64) | | 3 | |
| 43938 | RANCHO SECO - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42918 | RANCHO SECO - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42601 | RANCHO SECO - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |
| 42507 | RANCHO SECO - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

RANCHO SECO 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|--------------------------|---|---|-----------------|---|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44052 | RANCHO SECO - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44123 | RANCHO SECO - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44194 | RANCHO SECO - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44265 | RANCHO SECO - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44335 | RANCHO SECO - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44402 | RANCHO SECO - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44473 | RANCHO SECO - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44544 | RANCHO SECO - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44615 | RANCHO SECO - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44673 | RANCHO SECO - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44709 | RANCHO SECO - NUREG-0737 | II.E.1.2 1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44755 | RANCHO SECO - NUREG-0737 | II.E.1.2 2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44812 | RANCHO SECO - NUREG-0737 | II.E.4.1 2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44883 | RANCHO SECO - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44954 | RANCHO SECO - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45025 | RANCHO SECO - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45096 | RANCHO SECO - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45166 | RANCHO SECO - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45188 | RANCHO SECO - NUREG-0737 | II.K.2.10, SAFETY GR/DE ARTS | | |
| 45195 | RANCHO SECO - NUREG-0737 | II.K.2.11, OPERATOR TRAINING AND DRILLING | | |
| 45202 | RANCHO SECO - NUREG-0737 | II.K.2.13, THERMAL-MECHANICAL REPORT | | |
| 45209 | RANCHO SECO - NUREG-0737 | II.K.2.14, LIFT FREQUENCY OF PORV'S AND SV'S | | |
| 45216 | RANCHO SECO - NUREG-0737 | II.K.2.16, RCP SEAL DAMAGE | | |
| 45222 | RANCHO SECO - NUREG-0737 | II.K.2.17, POTENTIAL FOR VOIDING IN RCS | | |
| 45228 | RANCHO SECO - NUREG-0737 | II.K.2.20, SYSTEM RESPONSE TO SB LOCA | | |
| 45273 | RANCHO SECO - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45319 | RANCHO SECO - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45376 | RANCHO SECO - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45440 | RANCHO SECO - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCP'S | | |
| 45662 | RANCHO SECO - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45859 | RANCHO SECO - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45978 | RANCHO SECO - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46040 | RANCHO SECO - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46121 | RANCHO SECO - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46193 | RANCHO SECO - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46265 | RANCHO SECO - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46337 | RANCHO SECO - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46408 | RANCHO SECO - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46477 | RANCHO SECO - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>PLANT SPECIFIC</u> | | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>COMPLETE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 43065 | RANCHO SECO - TLD MONITORING LOCATION CHANGE | | | 11/08/80 |
| 42782 | RANCHO SECO - TECH. SPEC. AMENDMENT 70 - FLUX TO FLOW RATIO CHANGE | | | 11/21/80 |
| 42989 | RANCHO SECO - EXPANDED TLD MONITORING REQUIREMENTS | | | 12/02/80 |
| 43393 | RANCHO SECO - ITEM II K.2.15 SLUG FLOW EFFECTS IN OTS. | | | 02/03/81 |
| 42366 | RANCHO SECO - PURGE LINE ISOLATION VALVES FAIL OPEN | | | 02/11/81 |
| 43066 | RANCHO SECO - EMERGENCY PLANNING - NRC'S ROLE AT EMERGENCY FACILITIES | | | 02/19/81 |
| 43081 | RANCHO SECO - EMERGENCY PLANNING - EOF HABITABILITY REQUIREMENTS | | | 02/20/81 |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

RANCHO SECD 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|---|--|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> (CONTINUATION) | | | | <u>COMPLETE</u> |
| 43476 | RANCHO SECD - II K.2.19 BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW | | | 02/27/81 |
| 42265 | RANCHO SECD - FA HOLDDOWN SPRINGS | | 1 | 03/11/81 |
| 42961 | RANCHO SECD - NUREG 0578 CATEGORY A TECHNICAL SPECIFICATIONS | | 1 | 03/27/81 |
| 43195 | RANCHO SECD - PROPOSED TECH. SPEC. AMEND. NO. 67 - ORGANIZATION CHART/SURVEILLANCE TESTING | | 2 | 03/27/81 |
| 43707 | RANCHO SECD - TECH. SPECS. FOR AUXILIARY FEEDWATER SYSTEM UPGRADE | | | 01/27/81 |
| 07548 | RANCHO SECD - COOLDOWN TRANSIENT | | | 04/14/81 |
| 12889 | RANCHO SECD - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 43369 | RANCHO SECD - CYCLE 5 RELOAD | | 1 | 04/23/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 13099 | RANCHO SECD - SEISMIC ASSESSMENT OF FOOTHILLS FAULT SYSTEM | | | |
| 42049 | RANCHO SECD - LCO FOR DG OPERABILITY | | 1 | |
| 42790 | RANCHO SECD - CONFIRMATORY ORDER - ICS RELIABILITY ANALYSIS | | | |
| 42050 | RANCHO SECD - TECH. SPEC. AMENDMENT 71 - INVERTOR OUTAGE DURATION | | 1 | |
| 42933 | RANCHO SECD - ATMOSPHERIC DUMP VALVE CIRCUITRY MODIFICATION | | 1 | |
| 41190 | RANCHO SECD - AUXILIARY FEEDWATER SYSTEM UPGRADE | | 1 | |
| 43285 | RANCHO SECD - MISCELLANEOUS WATER EVAPORATOR | | 2 | |
| 43320 | RANCHO SECD - FIRE PROTECTION - APPENDIX R | | | |
| 43460 | RANCHO SECD - IAEA SAFEGUARDS | | 1 | |
| 43606 | RANCHO SECD - SNUBBER INSERVICE SURVEILLANCE TECHNICAL SPECIFICATIONS | | | |
| 43686 | RANCHO SECD - FSAR RADIOLOGICAL DOSE RE-EVALUATION | | | |
| 43702 | RANCHO SECD - SECURITY PLAN AMENDMENTS 7 & 8 | | | |
| 43427 | RANCHO SECD - ITEM II K.2.13 THERMAL-MECH EFFECTS OF HPI ON VESSEL | | | 03/31/81 |
| <u>ANTICIPATED ACTIONS:</u> | | | | <u>INITIATION</u> |
| 43992 | RANCHO SECD - PROPOSED AMENDM NO. 78 TO T/S RADIOACTIVE EFFLUENT MONITOR | | | 06/05/81 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: ROBINSON 2

PLANT LOCATION: 5 MI NW OF HARTSVILLE, SC
 DOCKET NUMBER: 050-00261
 ARCH/ENGINEER: EBASCO
 IE INSPECTOR: A. HARDEN

LICENSED POWER: 2300 MWT
 DESIGN POWER: 0700 MWE
 NSJS VENDOR: WEST

PROJECT MANAGER: D. NEIGHBORS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|--|-------------|----------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10489 | ROBINSON 2 - CONTINGENCY PLAN REVIEW | | 1 | COMPLETE 10/06/80 |
| 08681 | ROBINSON 2 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 11876 | ROBINSON 2 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEMS ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 11/26/80 |
| 08248 | ROBINSON 2 - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | 12/13/80 |
| 07904 | ROBINSON 2 - DIESEL GENERATOR LOCKOUT | | 3 | 01/19/81 |
| 08511 | ROBINSON 2 - TECH SPEC SURVEILLANCE REQMTS. FOR MECHANICAL SNUBBERS | | 2 | 01/30/81 |
| 11710 | ROBINSON 2 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 01/30/81 |
| 08618 | ROBINSON 2 - EMERGENCY PLANNING | | 1 | 04/01/81 |
| 12934 | ROBINSON 2 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08080 | ROBINSON 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | <u>TARGET</u> |
| 12275 | ROBINSON 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE H.B. ROBINSON 2 | | 3 | |
| 12978 | ROBINSON 2 - ADEQUACY OF STATION ELECTRIC. DISTR SYS. VOLTAGES | | 2 | |
| 42099 | ROBINSON 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43636 | ROBINSON 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 01954 | ROBINSON 2 - REQUEST FOR EXEMPTION FROM REQUIREMENTS OF 10 CFR 50 APPENDIX J | | 2 | |
| 06989 | ROBINSON 2 - INSERVICE INSPECTION | | 1 | |
| 11437 | ROBINSON 2 - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 08126 | ROBINSON 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 11215 | ROBINSON 2 - INSERVICE TESTING - PUMPS AND VALVES | | 1 | |
| 08519 | ROBINSON 2 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10335 | ROBINSON 2 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08522 | ROBINSON 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 10490 | ROBINSON 2 - VITAL AREA ANALYSIS | | 1 | |
| 08508 | ROBINSON 2 - TECH SPEC SURVEILLANCE REQMTS. FOR HYDRAULIC SNUBBERS | | 3 | |
| 12755 | ROBINSON 2 - QUALITY ASSURANCE REQUIREMENTS REGARDING DIESEL GENERATOR FUEL OIL | | 3 | |
| 10488 | ROBINSON 2 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 12581 | ROBINSON 2 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42704 | ROBINSON 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER, REV. & EVAL. RESPONSES | | 2 | |
| 42775 | ROBINSON 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43893 | H. B. ROBINSON UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42865 | ROBINSON 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43149 | ROBINSON 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 08505 | ROBINSON 2 - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 42466 | ROBINSON 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| TAC # | TAC DESCRIPTION | IMI ACTIONS | PRIORITY | CRITICAL DATE |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44053 | ROBINSON 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44124 | ROBINSON 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44195 | ROBINSON 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |

TECHNICAL ASSISTANCE CONTROL SYSTEM
CONDENSED MANAGEMENT REPORT

R-1208620-001

DATA AS OF - 05/31/81

(CONTINUATION)

ROBINSON 2

PRIORITY CRITICAL DATE
TARGET

IM1 ACTIONS

| TAC # | TAC DESCRIPTION | (CONTINUATION) | IM1 ACTIONS | PRIORITY | CRITICAL DATE |
|-------|-----------------|----------------|--|----------|---------------|
| 44266 | ROBINSON 2 | I.C. 1.2.A. | INADEQUATE CORE COOLING GUIDELINES | | |
| 44336 | ROBINSON 2 | I.C. 1.3A. | ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44403 | ROBINSON 2 | I.B. 1. | RCS HIGH POINT VENTS | | |
| 44474 | ROBINSON 2 | I.B. 3.2. | POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44545 | ROBINSON 2 | I.B. 4.1. | TRAINING FOR MITIGATING CORE | | |
| 44616 | ROBINSON 2 | I.D. 1.2. | RELIEF AND SAFETY VALVE TESTING | | |
| 44649 | ROBINSON 2 | I.D. 1.1. | REFM SYSTEM EVALUATION | | |
| 44689 | ROBINSON 2 | I.E. 1.2.1. | APW SAFETY GRADE AUTO INITIATION | | |
| 44728 | ROBINSON 2 | I.E. 1.2.2. | APW SAFETY GRADE FLOW INDICATION | | |
| 44813 | ROBINSON 2 | I.E. 4.1.2. | DEDICATED HYDROGEN PENETRATIONS | | |
| 44886 | ROBINSON 2 | I.E. 4.2. | CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44950 | ROBINSON 2 | I.E. 1.1. | NOBLE GAS MONITOR | | |
| 45029 | ROBINSON 2 | I.E. 1.1. | IODINE/ PARTICULATE SAMPLING | | |
| 45097 | ROBINSON 2 | I.E. 1.3. | CONTAINMENT HIGH RANGE MONITOR | | |
| 45167 | ROBINSON 2 | I.F. 2.3. | CONTAINMENT CORE COOLING INSTRUMENTATION | | |
| 45246 | ROBINSON 2 | I.K. 3.1. | INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45292 | ROBINSON 2 | I.K. 3.2. | AUTO PORV ISOLATION | | |
| 45377 | ROBINSON 2 | I.K. 3.3. | REPORT ON PORV FAILURES | | |
| 45409 | ROBINSON 2 | I.K. 3.5. | REPORT ON RV/SV FAILURES | | |
| 45458 | ROBINSON 2 | I.K. 3.9. | AUTO TRIP OF ROPS | | |
| 45488 | ROBINSON 2 | I.K. 3.10. | TD CONTROLLER | | |
| 45518 | ROBINSON 2 | I.K. 3.12. | ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45563 | ROBINSON 2 | I.K. 3.17. | ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45860 | ROBINSON 2 | I.K. 3.30. | ECC SYSTEM OUTAGES | | |
| 45979 | ROBINSON 2 | I.A. 1.2. | SB LOCA OUTLINE | | |
| 46050 | ROBINSON 2 | I.A. 1.2. | TECHNICAL SUPPORT CENTER | | |
| 46122 | ROBINSON 2 | I.A. 1.2. | OPERATIONAL SUPPORT CENTER | | |
| 46194 | ROBINSON 2 | I.A. 1.2. | EMERGENCY OPERATIONS FACILITY | | |
| 46266 | ROBINSON 2 | I.A. 1.2. | NUCLEAR DATA DATA | | |
| 46338 | ROBINSON 2 | I.A. 2.2. | EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46409 | ROBINSON 2 | I.A. 2.3. | METEOROLOGICAL DATA UPGRADE | | |
| 46478 | ROBINSON 2 | I.D. 3.3. | IMPLANT RADIATION MONITORING | | |
| | | I.D. 3.4. | CONTROL ROOM HABITABILITY | | |

PRIORITY CRITICAL DATE

PLANT SPECIFIC

COMPLETE
10/22/80
12/10/80
01/16/81
05/07/81
05/14/81

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|-------|-----------------|---|----------|---------------|
| 43077 | ROBINSON 2 | DEGRADED GRID VOLTAGE-BYPASS TRIP FEATURE AT LESS THAN 2% POWER | 1 | 10/22/80 |
| 43251 | ROBINSON 2 | STEAM GENERATOR WATER LEVEL INSTRUMENTATION | 1 | 12/10/80 |
| 43247 | ROBINSON 2 | HEAT TRACING | 1 | 01/16/81 |
| 42855 | ROBINSON 2 | SEPTEMBER 1980 STEAM GENERATOR INSPECTION | 1 | 05/07/81 |
| 43825 | ROBINSON 2 | EVENT V ADMINISTRATIVE CHANGE | 1 | 05/14/81 |

PRIORITY CRITICAL DATE

PLANT SPECIFIC

TARGET
2
1
3
2

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|-------|-----------------|---|----------|---------------|
| 42415 | ROBINSON 2 | SPENT FUEL POOL STORAGE EXPANSION | 2 | TARGET |
| 06809 | ROBINSON 2 | ECCS VALVES LOCKOUT | 1 | TARGET |
| 13024 | ROBINSON 2 | ENVIRONMENTAL TECHNICAL SPECIFICATIONS | 3 | TARGET |
| 41037 | ROBINSON 2 | DEFINITE OPERABLE | 2 | TARGET |
| 42566 | ROBINSON 2 | LONG TERM VIEW CONTAINMENT PURGE AND VENT (B-24) | 2 | TARGET |
| 43234 | ROBINSON 2 | LESSONS LEARNED TECH SPEC | 2 | TARGET |
| 43381 | ROBINSON 2 | EXEMPTION FOR FIRE PROTECTION OIL COLLECTION SYSTEM | 2 | TARGET |

DATA AS OF - 05/31/81

(CONTINUATION)

ROBINSON 2

| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------|---|-----------------------|-----------------|----------------------|
| | <u>ACTIVE ACTIONS</u> | | | <u>TARGET</u> |
| 43312 | ROBINSON - (CONTINUATION) MINIMUM WATER LEVEL | | | |
| 43602 | ROBINSON - EXEMPTION TO APPENDIX R TO EXTEND DATE OF SUBMITTALS | | | |
| 43603 | ROBINSON - EXEMPTION TO APPENDIX R ON FIRE SUPPRESSION SYSTEM IN CONTROL ROOM | | | |
| 43604 | ROBINSON - EXEMPTION TO APPENDIX R FROM COLD SHUTDOWN WITHIN 72 HOURS | | | |
| 43605 | ROBINSON - EXEMPTION TO APPENDIX R ON CABLE PENETRATION TEMPERATURE | | | |
| 43775 | ROBINSON - QA PROGRAM REVIEW | | | |
| 43616 | ROBINSON 2 - MAY 1981 STEAM GENERATOR TUBE INSPECTION | | | |
| 43828 | ROBINSON 2 - ALTERNATE SHUTDOWN SYSTEM | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: SALEM 1

PLANT LOCATION: 20 MI S OF WILMINGTON, DEL
 DOCKET NUMBER: 050-00272
 ARCH/ENGINEER: PUBSERVE
 IE INSPECTOR: L. NORRHOLM

LICENSED POWER: 3338 MWT
 DESIGN POWER: 1090 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: W. ROSS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 08682 | SALEM 1 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 08170 | SALEM 1 - PWR HPSI-LPSI FLOW RESISTANCE | | 3 | 11/28/80 |
| 42331 | SALEM 1 - MASONRY WALL DESIGN REVIEW | | 2 | 11/28/80 |
| 43010 | SALEM 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 11/28/80 |
| 07245 | SALEM 1 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | 01/21/81 |
| 08993 | SALEM 1 - VENTING AND PURGING CONTAINMENTS WHILE AT FULL POWER AND EFFECT ON LOCA | | 1 | 01/31/81 |
| 10272 | SALEM 1 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 01/31/81 |
| 42100 | SALEM 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 03/06/81 |
| 12435 | SALEM 1 - LESSONS LEARNED IMPLEMENTATION | | 1 | 03/21/81 |
| 12935 | SALEM 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 10738 | SALEM 1 - GUARD TRAINING PLAN REVIEW | | 1 | 05/12/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 10746 | SALEM 1 - VITAL AREA ANALYSIS | | 1 | |
| 11142 | SALEM 1 - CONTAINMENT LEAK TESTING APP J | | 2 | |
| 11224 | SALEM 1 - INSERVICE TESTING (IST) | | 1 | |
| 11877 | SALEM 1 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | |
| 12276 | SALEM 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SALEM 1 | | 3 | |
| 43661 | SALEM 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 10336 | SALEM 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08824 | SALEM 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08154 | SALEM 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 42705 | SALEM 1 - IE BULLETIN 79-27 LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 12603 | SALEM 1 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42776 | SALEM 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43936 | SALEM UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 06659 | SALEM 1 - INSERVICE INSPECTION - (ISI) | | 1 | |
| 08398 | SALEM 1 - PWR MODERATOR DILUTION | | 3 | |
| 42866 | SALEM 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 08081 | SALEM 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 43150 | SALEM 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 10050 | SALEM 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 11711 | SALEM 1 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | |
| 12163 | SALEM 1 - EMERGENCY PLAN REVIEW | | 1 | |
| 13009 | SALEM 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 11143 | SALEM 1 - HYDRAULIC SNUBBERS | | 3 | |
| 11227 | SALEM 1 - HYDRAULIC SNUBBERS-UPGRADE TECH SPECS | | 3 | |
| 42467 | SALEM 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 44055 | SALEM 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44126 | SALEM 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SALEM 1

| TAC # | TAC DESCRIPTION | IMI ACTIONS | PRIORITY | CRITICAL DATE |
|--------------------------|--|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | (CONTINUATION) | | <u>TARGET</u> |
| 44197 | SALEM 1 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44263 | SALEM 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44330 | SALEM 1 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44405 | SALEM 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44476 | SALEM 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44547 | SALEM 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44618 | SALEM 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44650 | SALEM 1 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44690 | SALEM 1 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44729 | SALEM 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44815 | SALEM 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44886 | SALEM 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44957 | SALEM 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45028 | SALEM 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45099 | SALEM 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45169 | SALEM 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45247 | SALEM 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45293 | SALEM 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45379 | SALEM 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45410 | SALEM 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45459 | SALEM 1 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45489 | SALEM 1 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45519 | SALEM 1 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45665 | SALEM 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45862 | SALEM 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45981 | SALEM 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46052 | SALEM 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46124 | SALEM 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46196 | SALEM 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46268 | SALEM 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46340 | SALEM 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46411 | SALEM 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46480 | SALEM 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42067 | SALEM 1 - QUALITY ASSURANCE OF DIESEL GENERATOR FUEL OIL OIL | | | 10/16/80 |
| 05206 | SALEM 1 - REVIEW OF RESEARCH COPPER DATA | | 1 | 11/28/80 |
| 42069 | SALEM 1 - CLARIFICATION OF "OPERABLE" | | | 11/28/80 |
| 13048 | SALEM 1 - CHANGES OF THE 125-VOLT AND 28-VOLT BATTERY INDIVIDUAL CELL MINIMUM VOLTAGES | | 1 | 12/09/80 |
| 42914 | SALEM 1 - WATER LEVEL IRRADIATED FUEL | | | 12/09/80 |
| 43097 | SALEM 1 - RADIATION PROTECTION - ADMINISTRATIVE CHANGES | | | 12/09/80 |
| 43090 | SALEM 1 - AXIAL FLUX DIFFERENCE TARGET BAND | | | 12/22/80 |
| 43217 | SALEM 1 - MODIFICATION OF SURVEILLANCE FLOW TEST | | | 01/06/81 |
| 08821 | SALEM 1 - UTILITY ORGANIZATIONAL CHANGES | | | 02/02/81 |
| 43398 | SALEM 1 - SPENT FUEL POOL MOD HEARING EFFORT | | 1 | 02/02/81 |
| | | | 2 | 04/22/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SALEM 1

| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------|---|-----------------------|-----------------|----------------------|
| | (CONTINUATION) | | | |
| | ACTIVE ACTIONS | | | |
| 12335 | SALEM 1 - CHANGES TO REACTOR TRIP SETPOINTS | | 1 | TARGET |
| 43299 | SALEM 1 - ANALYSIS OF FLUX TILT DURING CYCLE #3 | | 1 | |
| 07549 | SALEM 1 - RADIOLOGICAL SAFETY IS RE: FIRE PROTECTION DETECTION INSTR. SURVEILLANCE REQUIREMENTS | | 1 | |
| 43339 | SALEM 1 - EXEMPTIONS FROM 10CFR 50.48 AND APPENDIX R TO 10CFR50 | | 1 | |
| 43812 | SALEM 1 - MODIFICATION OF COMPOSITION OF NUCLEAR REVIEW BOARD | | 1 | |
| 42068 | SALEM 1 - PSE&G BENCHMARK SEISMIC PROBLEMS | | 1 | |
| 42661 | SALEM 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: SALEM 2

PLANT LOCATION: 20 MI S OF WILMINGTON, DEL
 DOCKET NUMBER: 050-00311
 ARCH/ENGINEER: PUBSERVE
 IE INSPECTOR:

LICENSED POWER: 3411 MWT
 DESIGN POWER: 1115 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: G. MEYER
 BRANCH CHIEF: F. MIRAGLIA
 LIC. ASSISTANT: J. LEE

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 11712 | SALEM 2 THREE MILE ISLAND FOLLOW UP WORK | | 1 | <u>TARGET</u> |
| 12164 | SALEM 2, EMERGENCY PLAN REVIEW | | 1 | |
| 43674 | SALEM 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 43949 | SALEM 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42533 | SALEM 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44056 | SALEM 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44127 | SALEM 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44198 | SALEM 2 - NUREG-0737 I.A.2.1.4, UPGRAING OF RO AND SRO TRAINING | | | |
| 44269 | SALEM 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44339 | SALEM 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44406 | SALEM 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44477 | SALEM 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44548 | SALEM 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44619 | SALEM 2 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44651 | SALEM 2 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44691 | SALEM 2 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |
| 44730 | SALEM 2 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44816 | SALEM 2 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44887 | SALEM 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44958 | SALEM 2 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45029 | SALEM 2 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45100 | SALEM 2 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45170 | SALEM 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45248 | SALEM 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45294 | SALEM 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45380 | SALEM 2 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45411 | SALEM 2 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45460 | SALEM 2 - NUREG-0737 II.K.3.9, PID CONTROLLER | | | |
| 45490 | SALEM 2 - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | | |
| 45520 | SALEM 2 - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | | |
| 45666 | SALEM 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45863 | SALEM 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45982 | SALEM 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46053 | SALEM 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46125 | SALEM 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46197 | SALEM 2 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46269 | SALEM 2 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46341 | SALEM 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46412 | SALEM 2 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46481 | SALEM 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |

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SALEM 2

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ACTIVE ACTIONS

TARGET

1201 SALEM 2- SECURITY PLAN PSEG LTR OF 11-13-78

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CONDENSED MANAGEMENT REPORT

FACILITY: SAN ONOFRE 1

PLANT LOCATION: 5 MI S OF SAN CLEMENTE, CA
 DOCKET NUMBER: 050-00206
 ARCH/ENGINEER: BECH
 IE INSPECTOR: H. CANTER

LICENSED POWER: 1347 MWT
 DESIGN POWER: 0436 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: S. NOWICKI
 BRANCH CHIEF: D. CKUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10153 | SAN ONOFRE 1 - REVIEW MATERIAL SURVEILLANCE RESULTS | | 3 | <u>COMPLETE</u> 01/11/81 |
| 10587 | SAN ONOFRE 1 - GUARD TRAINING PLAN REVIEW | | 1 | 01/22/81 |
| 10588 | SAN ONOFRE 1 - CONTINGENCY PLAN REVIEW | | 1 | 01/22/81 |
| 11114 | SAN ONOFRE 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 02/04/81 |
| 12293 | SAN ONOFRE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SAN ONOFRE | | 3 | 02/18/81 |
| 12920 | SAN ONOFRE 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 03/03/81 |
| 07966 | SAN ONOFRE 1 - FLOOD OF EQUIP IMPORTANT TO SAFETY | | 2 | 03/18/81 |
| 10289 | SAN ONOFRE 1 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 03/25/81 |
| 10072 | SAN ONOFRE 1 - ENHANCED FISSION PRODUCT RELEASE FOR HIGH BURNUP LWR FUEL | | 1 | 04/28/81 |
| 10351 | SAN ONOFRE 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | 04/29/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 11696 | SAN ONOFRE 1 - TMI FOLLOW UP WORK | | 1 | <u>TARGET</u> |
| 12133 | SAN ONOFRE 1 - EMERGENCY PLAN REVIEW | | 1 | |
| 13178 | SAN ONOFRE 1 - REACTOR VESSEL CAVITY SEAL RING MISSILE POTENTIAL | | 2 | |
| 42131 | SAN ONOFRE 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43053 | SAN ONOFRE 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43664 | SAN ONOFRE 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 06836 | SAN ONOFRE 1 - REACTOR VESSEL OVERPRESSURE PROTECTION | | 2 | |
| 10148 | SAN ONOFRE 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08101 | SAN ONOFRE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 11231 | SAN ONOFRE 1 - INSERVICE TESTING (IST) | | 1 | |
| 08592 | SAN ONOFRE 1 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08082 | SAN ONOFRE 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08693 | SAN ONOFRE 1 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | |
| 10150 | SAN ONOFRE 1 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10189 | SAN ONOFRE 1 - CONTAINMENT PURGE | | 1 | |
| 10051 | SAN ONOFRE 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 10152 | SAN ONOFRE 1 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS | | 2 | |
| 10815 | SAN ONOFRE 1 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 12439 | SAN ONOFRE UNIT 1 - LESSONS LEARNED IMPLEMENTATION | | 1 | |
| 07808 | SAN ONOFRE 1 - STEAM GENERATOR WATER HAMMER | | 2 | |
| 10584 | SAN ONOFRE 1 - VITAL AREA ANALYSIS | | 1 | |
| 06491 | SAN ONOFRE 1 - SECONDARY WATER CHEMISTRY MONITORING REQUIREMENTS | | 2 | |
| 12582 | SAN ONOFRE 1 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 08174 | SAN ONOFRE 1 - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 42706 | SAN ONOFRE 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 43939 | SAN ONOFRE 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42777 | SAN ONOFRE 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42916 | SAN ONOFRE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43151 | SAN ONOFRE 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42516 | SAN ONOFRE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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SAN ONOFRE 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---------------------------|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 44057 | SAN ONOFRE 1 - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44128 | SAN ONOFRE 1 - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44199 | SAN ONOFRE 1 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44270 | SAN ONOFRE 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44340 | SAN ONOFRE 1 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44407 | SAN ONOFRE 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44478 | SAN ONOFRE 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44549 | SAN ONOFRE 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44620 | SAN ONOFRE 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44652 | SAN ONOFRE 1 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44692 | SAN ONOFRE 1 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44731 | SAN ONOFRE 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44817 | SAN ONOFRE 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44888 | SAN ONOFRE 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44959 | SAN ONOFRE 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45030 | SAN ONOFRE 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45101 | SAN ONOFRE 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45171 | SAN ONOFRE 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45249 | SAN ONOFRE 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45295 | SAN ONOFRE 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45381 | SAN ONOFRE 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45412 | SAN ONOFRE 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45461 | SAN ONOFRE 1 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45491 | SAN ONOFRE 1 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45521 | SAN ONOFRE 1 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45667 | SAN ONOFRE 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45864 | SAN ONOFRE 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45983 | SAN ONOFRE 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46054 | SAN ONOFRE 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46126 | SAN ONOFRE 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46198 | SAN ONOFRE 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46270 | SAN ONOFRE 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46342 | SAN ONOFRE 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46413 | SAN ONOFRE 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46482 | SAN ONOFRE 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 08047 | SAN ONOFRE 1 - CORE RELOAD FOR CYCLE 7 | | 1 | 01/20/81 |
| 13115 | SAN ONOFRE 1 - HIGH VOLTAGE TRANSMISSION LINES-TIEIN TO NEW UNITS 2&3 SWITCHYARD | | 1 | 02/06/81 |
| 43465 | SAN ONOFRE 1 - FIRE PROTECTION EXTENSION REQUEST | | 1 | 02/13/81 |
| 05205 | SAN ONOFRE 1 - REVIEW OF RESEARCH COPPER DATA | | 1 | 02/27/81 |
| 43464 | SAN ONOFRE 1 - PREPARATION OF EVENT V ORDER | | 1 | 03/03/81 |
| 12514 | SAN ONOFRE 1 - DELETE ETS SPECIFICATION 4.3 "ENTRAINMENT" 4.3 "ENTRAINMENT" | | 1 | 03/13/81 |
| 12869 | SAN ONOFRE 1 - DELETE PLANKTAN STUDIES | | 2 | 03/18/81 |
| 06309 | SAN ONOFRE 1 - COLLECTION OF CHLORINE DATA FOR ESB | | 3 | 03/25/81 |
| 43469 | SAN ONOFRE 1 - AMENDMENT NO. 96, ADMINISTRATIVE STAFF CHANGES | | 1 | 04/30/81 |
| 43757 | SAN ONOFRE 1 - AMENDMEN. 101 - ORGANIZATION CHANGES | | 2 | 04/30/81 |

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SAN ONOFRE 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 10853 | SAN ONOFRE 1 - CHANGE TESTING OF STATION BATTERY SYSTEM | | 2 | |
| 12547 | SAN ONOFRE 1 - EVACUATION ISSUE IN 10CFR 2.206 REQUEST REQUEST | | 1 | |
| 12552 | SAN ONOFRE 1 - SEISMIC ISSUE IN 10CFR 2.206 REQUEST REQUEST | | 1 | |
| 42089 | SAN ONOFRE 1 - CONTAINMENT PRESSURE REANALYSIS FOR MSLB T3 CONT. | | 1 | |
| 42090 | SAN ONOFRE 1 - STEAM GENERATOR IN SERVICE INSPECTION RESULTS - JUNE 1980 | | 1 | |
| 13132 | SAN ONOFRE 1 - SPECIFY LCD'S & S/R FOR CONTROL ROD MISALIGNMENT (PROPOSED CHANGE NO 89) | | 1 | |
| 42051 | SAN ONOFRE 1 - LOSS OF SALT WATER COOLING | | 1 | |
| 42377 | SAN ONOFRE 1 - STEAM GENERATOR ENVIRONMENTAL IMPACT ASSESSMENT | | 1 | |
| 43563 | SAN ONOFRE 1 - ISSUE SECURITY PLAN CHANGES | | 2 | |
| 43562 | SAN ONOFRE 1 - LOAD SEQUENCER MODIFICATIONS | | 1 | |
| 43712 | SAN ONOFRE 1 - LEAK RATE DESIGN CRITERIA FOR HYBRID SLEEVES | | 1 | |
| 46495 | SAN ONOFRE UNIT 1 - OPERATING MODES PROPOSED CHANGE 96 | | | |
| 46517 | SAN ONOFRE - UPS BATTERY SERVICE TEST | | | |
| 46516 | SAN ONOFRE - OPERATING MODES | | | |
| 11232 | SAN ONOFRE 1 - FULL TERM OPERATING LICENSE | | 1 | |
| 46518 | SAN ONOFRE - FIRE PROTECTION TECH SPECS | | | |
| 42615 | SAN ONOFRE 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | 1 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: SEQUOYAH 1

PLANT LOCATION: 9.5 MI NE OF CHATTANOOGA, TN
 DOCKET NUMBER: 050-00327
 ARCH/ENGINEER: TVA
 IE INSPECTOR:

LICENSED POWER: 3411 MWT
 DESIGN POWER: 1148 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: C. STAHL
 BRANCH CHIEF: A. SCHWENCER
 LIC. ASSISTANT: M. SERVICE

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 12173 | SEQUOYAH 1 - EMERGENCY PLAN REVIEW | | 1 | <u>TARGET</u> |
| 43675 | SEQUOYAH 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 43950 | SEQUOYAH - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42535 | SEQUOYAH 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44058 | SEQUOYAH 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44129 | SEQUOYAH 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44200 | SEQUOYAH 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44271 | SEQUOYAH 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44341 | SEQUOYAH 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44408 | SEQUOYAH 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44479 | SEQUOYAH 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44550 | SEQUOYAH 1 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44621 | SEQUOYAH 1 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44653 | SEQUOYAH 1 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION | | | |
| 44693 | SEQUOYAH 1 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |
| 44732 | SEQUOYAH 1 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44818 | SEQUOYAH 1 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44889 | SEQUOYAH 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44960 | SEQUOYAH 1 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45031 | SEQUOYAH 1 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45102 | SEQUOYAH 1 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45172 | SEQUOYAH 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45250 | SEQUOYAH 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45296 | SEQUOYAH 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45382 | SEQUOYAH 1 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45413 | SEQUOYAH 1 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45462 | SEQUOYAH 1 - NUREG-0737 II.K.3.9, PID CONTROLLER | | | |
| 45492 | SEQUOYAH 1 - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | | |
| 45522 | SEQUOYAH 1 - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | | |
| 45668 | SEQUOYAH 1 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45865 | SEQUOYAH 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45984 | SEQUOYAH 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46055 | SEQUOYAH 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46127 | SEQUOYAH 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46199 | SEQUOYAH 1 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46271 | SEQUOYAH 1 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46343 | SEQUOYAH 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46414 | SEQUOYAH 1 - NUREG-0737 II.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46483 | SEQUOYAH 1 - NUREG-0737 II.D.3.4, CONTROL ROOM HABITABILITY | | | |

TECHNICAL ASSIGNMENT CONTROL SYSTEM

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(CONTINUATION)

SEQUOYAH 1

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY CRITICAL DATE

ACTIVE ACTIONS

43229 SEQUOYAH 1 - AMENDMENT FOR IE BULLETIN 80-18

TARGET

1

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: ST LUCIE 1

PLANT LOCATION: 12 MI SE OF FT. PIERCE, FLA
 DOCKET NUMBER: 050-00335
 ARCH/ENGINEER: EBASCO
 IE INSPECTOR: S. ELROD

LICENSED POWER: 2560 MWT
 DESIGN POWER: 0802 MWE
 NSSS VENDOR: COMB

PROJECT MANAGER: C. NELSON
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUZER

| <u>IAC #</u> | <u>IAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 10809 | ST. LUCIE 1 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | <u>COMPLETE</u> 11/13/80 |
| 11731 | ST. LUCIE 1 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 01/19/81 |
| 07319 | ST. LUCIE 1 - STEAM GENERATOR TUBE DENTING AND SUPPORT PLATE MODIFICATIONS | | 3 | 02/02/81 |
| 10315 | ST. LUCIE 1 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 02/04/81 |
| 10421 | ST. LUCIE 1 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | 02/04/81 |
| 10422 | ST. LUCIE 1 - TECH SPEC SURVEILLANCE FOR MECHANICAL SNUBBERS | | 2 | 02/04/81 |
| 11307 | ST. LUCIE 1 - CONTAINMENT PURGE | | 1 | 02/09/81 |
| 10252 | ST. LUCIE 1 - INADVERTENT SAFETY INJECTION DURING COOLDOWN (PWR'S) | | 2 | 03/17/81 |
| 12151 | ST. LUCIE 1 - UPGRADED EMERGENCY PLAN | | 1 | 04/01/81 |
| 43630 | ST. LUCIE 1 - INADVERTENT SAFETY INSPECTION DURING SURVEILLANCE | | 2 | 04/06/81 |
| 12887 | ST. LUCIE 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 43034 | ST. LUCIE 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 05/28/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12319 | ST. LUCIE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ST. LUCIE 1 | | 3 | <u>TARGET</u> |
| 12739 | ST. LUCIE 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 43648 | ST. LUCIE 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 10621 | ST. LUCIE 1 - VITAL ARFA ANALYSIS | | 1 | |
| 12600 | ST. LUCIE 1 - ANAL OF TURBINE DISC CRACKS | | 2 | |
| 08920 | ST. LUCIE 1 - CE GENERIC RTD RESPONSE TIME | | 3 | |
| 10619 | ST. LUCIE 1 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 11108 | ST. LUCIE 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 42118 | ST. LUCIE 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43962 | ST. LUCIE 1 - IE BULLETIN 79-06 | | | |
| 06826 | ST. LUCIE 1 - REACTOR VESSEL OVERPRESSURE PROTECTION | | 2 | |
| 08250 | ST. LUCIE 1 - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | |
| 10052 | ST. LUCIE 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 10469 | ST. LUCIE 1 - BORON SOLUBILITY DURING LONG TERM COOLING FOLLOWING LOCA | | 3 | |
| 42778 | ST. LUCIE 1 - ESP RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 08083 | ST. LUCIE 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08155 | ST. LUCIE 1 - APP I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 42707 | ST. LUCIE 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 08292 | ST. LUCIE 1 - 10 CFR 50.55A(G) - INSERVICE INSPECTION AND TESTING - GENERIC | | 1 | |
| 42900 | ST. LUCIE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43157 | ST. LUCIE 1 - PWR FEEDWATER LINE CRACKS TERM CORRECTIVE ACTION | | 2 | |
| 08918 | ST. LUCIE 1 - CEA POSITION INDICATOR CHANNELS | | 3 | |
| 08923 | ST. LUCIE 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08924 | ST. LUCIE 1 - PWR PUMP & S/G SUPPORTS-LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10377 | ST. LUCIE 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 11308 | ST. LUCIE 1 - REACTOR PROTECTION SYSTEM LOGIC | | 3 | |
| 42492 | ST. LUCIE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ST LUCIE 1

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|-----------------------|--|---|----------|-------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44054 | ST. LUCIE 1 - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | |
| 44125 | ST. LUCIE 1 - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44196 | ST. LUCIE 1 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44267 | ST. LUCIE 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44337 | ST. LUCIE 1 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44404 | ST. LUCIE 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44475 | ST. LUCIE 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44546 | ST. LUCIE 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44617 | ST. LUCIE 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44667 | ST. LUCIE 1 - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44748 | ST. LUCIE 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44814 | ST. LUCIE 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44885 | ST. LUCIE 1 - NUREG-0737 | II.C.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44956 | ST. LUCIE 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45027 | ST. LUCIE 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45098 | ST. LUCIE 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45168 | ST. LUCIE 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45266 | ST. LUCIE 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45312 | ST. LUCIE 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45378 | ST. LUCIE 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45433 | ST. LUCIE 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45664 | ST. LUCIE 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45861 | ST. LUCIE 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45980 | ST. LUCIE 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46051 | ST. LUCIE 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46123 | ST. LUCIE 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46195 | ST. LUCIE 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46267 | ST. LUCIE 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46339 | ST. LUCIE 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46410 | ST. LUCIE 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46479 | ST. LUCIE 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| 43378 | ST LUCIE 1 - AFW AUTO INITIATION & FLOW INDICATION | | 1 | 07/01/81 |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE COMPLETE |
|--------------------------|-----------------|---|----------|---------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42253 | ST. LUCIE 1 - | LESSONS LEARNED ITEM 2.1.8.P 6-27-80 PROPOSAL | | 10/31/80 |
| 43235 | ST. LUCIE 1 - | SECURITY INSPECTION FINDINGS | | 12/19/80 |
| 43242 | ST. LUCIE 1 - | CONTAINMENT ISOLATION ON SIAS | | 01/19/81 |
| 06994 | ST. LUCIE 1 - | REVIEW OF 1976 METEOROLOGICAL DATA | 1 | 01/21/81 |
| 43326 | ST LUCIE 1 - | SECONDARY CONTAINMENT LEAKAGE & CONTROL ROOM INTAKE | | 02/25/81 |
| 11614 | ST. LUCIE 1 - | DELETION OF WATER QUALITY ETS | 2 | 03/10/81 |
| 12760 | ST. LUCIE 1 - | CHANGE FROM HYDRAULIC TO MECHANICAL SNUBBERS | 2 | 05/11/81 |
| 43526 | ST LUCIE 1 - | FUEL POOL CASK DROP ANALYSIS | 2 | 05/28/81 |
| 43466 | ST LUCIE 1 - | TMI FOLLOWUP | 1 | 05/29/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 11173 | ST. LUCIE 1 - | ULTIMATE HEAT SINK - TECHNICAL SPECIFICATIONS | 2 | |
| 42252 | ST. LUCIE 1 - | REACTOR VESSEL STEAM BUBBLE | 1 | |
| 43467 | ST LUCIE 1 - | DELETE BIOLOGICAL MONITORING FROM ETS | 2 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ST LUCIE 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | (CONTINUATION) | | <u>1:30ET</u> |
| 43629 | ST. LUCIE 1 - RADIOLOGICAL SAMPLE POINTS | | 1 | |
| 42435 | ST. LUCIE 1 - CE DESIGN METHODOLOGY REPORTS | | 1 | |
| 43852 | ST LUCIE 1 - EXEMPTION FROM APP. R SECT. 111G | | 1 | |
| 43853 | ST LUCIE 1 - EXEMPTION FROM APPENDIX R SECT. 111A | | 1 | |
| 43869 | ST. LUCIE 1 - FIRE PROTECTION TECH SPEC | | 2 | |
| 11483 | ST. LUCIE 1 - NA OH FLOW PATH VERIFICATION | | 2 | |
| 11496 | ST. LUCIE 1 - ENGINEERED SAFETY FEATURES RESPONSE TIMES | | 1 | |
| 43241 | ST. LUCIE 1 - STATION BLACKOUT | | 1 | |
| 43243 | ST. LUCIE 1 - POWER INCREASE | | 1 | |
| 42593 | ST. LUCIE 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |

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CONDENSED MANAGEMENT REPORT

FACILITY: SURRY 1

PLANT LOCATION: 17 MI NW OF NEWPORT NEWS, VA
 DOCKET NUMBER: 050-00280
 ARCH/ENGINEER: S&W
 IE INSPECTOR: D. BURKE

LICENSED POWER: 2441 MWT
 DESIGN POWER: 0788 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. NEIGHBORS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|--|-------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 08683 | SURRY 1 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 11878 | SURRY 1 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 11/26/80 |
| 07964 | SURRY 1 - FLOOD OF EQUIP IMPORTANT TO SAFETY | | 2 | 12/13/80 |
| 08251 | SURRY 1 - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | 12/18/80 |
| 10493 | SURRY 1 - GUARD TRAINING PLAN REVIEW | | 1 | 12/18/80 |
| 08513 | SURRY 1 - TECH SPEC SURVEILLANCE REQMTS. FOR MECHANICAL SNUBBERS | | 2 | 01/30/81 |
| 10388 | SURRY 1 - WESTINGHOUSE N-1 LOOP OPERATION | | 3 | 01/30/81 |
| 11713 | SURRY 1 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 01/30/81 |
| 08641 | SURRY 1 - INSERVICE INSP. OF STEAM GENERATOR TUBES | | | 02/10/81 |
| 12160 | SURRY 1 - UPGRADED EMERGENCY PLAN | | 1 | 04/01/81 |
| 12937 | SURRY 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 42101 | SURRY 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 05/02/81 |
| 13076 | SURRY 1 - REORGANIZATION | | 3 | 05/22/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 10053 | SURRY 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGES | | 2 | |
| 12277 | SURRY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SURRY 1 | | 3 | |
| 12976 | SURRY 1 - ADEQUACY OF STATION ELECTRIC. DISTR. SYS VOLTAGES | | 2 | |
| 43667 | SURRY 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 07906 | SURRY 1 - DIESEL GENERATOR LOCKOUT | | 3 | |
| 08357 | SURRY 1 - BLOCKING S.I. SIGNAL DURING COOLDOWN | | 2 | |
| 08401 | SURRY 1 - GENERIC - PWR MODERATOR DILUTION | | 3 | |
| 08620 | SURRY 1 - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT | | 2 | |
| 10840 | SURRY 1 - HYDRAULIC SNUBBERS | | 3 | |
| 07576 | SURRY 1 - OUTSIDE RECIRC AND LOW HEAD SAFETY INJECTION PUMPS REPAIR | | 3 | |
| 08638 | SURRY 1 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 11091 | SURRY 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 08084 | SURRY 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08617 | SURRY 1 - FILTER TECH SPECS | | 3 | |
| 08327 | SURRY 1 - 10 CFR 50.55A(G) - INSERVICE INSPECTION | | 1 | |
| 11216 | SURRY 1 - INSERVICE TESTING - PUMPS AND VALVES - IST | | 1 | |
| 08518 | SURRY 1 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10337 | SURRY 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 11974 | SURRY 1 - REACTOR CAVITY SEAL RING GENERIC ISSUE | | 2 | |
| 08521 | SURRY 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 10491 | SURRY 1 - VITAL AREA ANALYSIS | | 1 | |
| 08509 | SURRY 1 - TECH SPEC SURVEILLANCE REQMTS. FOR HYDRAULIC SNUBBERS | | 3 | |
| 12579 | SURRY 1 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42708 | SURRY 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43942 | SURRY UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42779 | SURRY 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42867 | SURRY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43153 | SURRY 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42468 | SURRY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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(CONTINUATION)

SURRY 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44059 | SURRY 1 - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS * | | <u>TARGET</u> |
| 44130 | SURRY 1 - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44201 | SURRY 1 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44272 | SURRY 1 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44342 | SURRY 1 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44405 | SURRY 1 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44480 | SURRY 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44551 | SURRY 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44622 | SURRY 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44694 | SURRY 1 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44733 | SURRY 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44819 | SURRY 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44890 | SURRY 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44961 | SURRY 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45032 | SURRY 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45103 | SURRY 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45173 | SURRY 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45251 | SURRY 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45297 | SURRY 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45383 | SURRY 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45414 | SURRY 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45463 | SURRY 1 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45493 | SURRY 1 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45523 | SURRY 1 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45669 | SURRY 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45866 | SURRY 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45985 | SURRY 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46056 | SURRY 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46128 | SURRY 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46200 | SURRY 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46272 | SURRY 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46344 | SURRY 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46415 | SURRY 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46484 | SURRY 1 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| 12639 | SURRY 1 - AUXILIARY FEEDWATER SYSTEM | | 1 | 03/31/80 |
| <u>PLANT SPECIFIC</u> | | | | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42856 | SURRY 1 - POST FES ACTIVITIES | | | <u>COMPLETE</u> |
| 42185 | SURRY 1 - DEFINE OPERABLE | | | 01/06/81 |
| 43205 | SURRY 1 - WATER LEVEL OVER SPENT FUEL | | | 02/06/81 |
| 42800 | SURRY 1 - SYSTEM NUCLEAR SAFETY AND OPERATING COMMITTEE | | 2 | 05/12/81 |
| | | | | 05/22/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 41038 | SURRY 1 - CHANGE FUEL ENRICHMENT | | 2 | <u>TARGET</u> |
| 07058 | SURRY 1 - NPSH REVIEW FOR CONTAINMENT RECIRC SPRAY PUMPS & LOW HEAD SAFETY INJECTION PUMPS | | 1 | |
| 06872 | SURRY 1 - DETERMINE 2 HR. SITE BOUNDARY DOSE | | 1 | |
| 13078 | SURRY 1 - AUXILIARY FW SYSTEM FOR FIRE PROTECTION | | 1 | |
| 43113 | SURRY 1 - HIGH ENERGY BREAK INSPECTION PROGRAM T.S. 4.15-A | | 1 | |

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TECHNICAL ASSIGNMENT CONTROL SYSTEM

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(CONTINUATION)

SURRY 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | (CONTINUATION) | | | |
| 43115 | SURRY 1 - FIRE PUMP TECH SPEC 4.18.B.1.F(2) | | 1 | <u>TARGET</u> |
| 43203 | SURRY 1 - LESSONS LEARNED TECH SPECS | | | |
| 43816 | SURRY 1 - ECCS - FQ CHANGE | | | |
| 42562 | SURRY 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |

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CONDENSED MANAGEMENT REPORT

FACILITY: SURRY 2

PLANT LOCATION: 17 MI NW OF NEWPORT NEWS, VA
 DOCKET NUMBER: 050-00281
 ARCH/ENGINEER: S&W
 IE INSPECTOR: D. BURKE

LICENSED POWER: 2441 MWT
 DESIGN POWER: 0788 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. NEIGHBORS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRIS

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|--|-------------|----------|-----------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 08684 | SURRY 2 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 11/13/80 |
| 11879 | SURRY 2 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 11/26/80 |
| 06697 | SURRY 2 - FLOOD OF EQUIP IMPORTANT TO SAFETY | | 2 | 12/18/80 |
| 08252 | SURRY 2 - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | 12/18/80 |
| 10494 | SURRY 2 - GUARD TRAINING PLAN REVIEW | | 1 | 12/18/80 |
| 08512 | SURRY 2 - TECH SPEC SURVEILLANCE REQMTS. FOR MECHANICAL SNUBBERS | | 2 | 01/30/81 |
| 10389 | SURRY 2 - WESTINGHOUSE N 1 LOOP OPERATION | | 3 | 01/30/81 |
| 11714 | SURRY 2 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 01/30/81 |
| 12161 | SURRY 2 - UPGRADED EMERGENCY PLAN | | 1 | 04/01/81 |
| 12938 | SURRY 2 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 42102 | SURRY 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 05/12/81 |
| 13077 | SURRY 2 - REORGANIZATION | | 3 | 05/22/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 10338 | SURRY 2 - LOSS OF 125V DC BUS VOLTAGE W/LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 12278 | SURRY 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SURRY 2 | | 3 | |
| 12977 | SURRY 2 - ADEQUACY OF STATION ELECTRIC. DISTR. SYS. VOLTAGES | | 2 | |
| 43668 | SURRY 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 07907 | SURRY 2 - DIESEL GENERATOR LOCKOUT | | 3 | |
| 08364 | SURRY 2 - BLOCKING S.I. SIGNAL DURING COOLDOWN | | 2 | |
| 08402 | SURRY 2 - GENERIC - PWR MODERATOR DILUTION | | 3 | |
| 08619 | SURRY 2 - FUEL HANDLING ACCIDENT INST'DE CONTAINMENT | | 2 | |
| 08103 | SURRY 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 10841 | SURRY 2 - HYDRAULIC SNUBBERS | | 3 | |
| 08368 | SURRY 2 - OUTSIDE RECIRC. AND LHSI PUMPS REPAIR | | 3 | |
| 08639 | SURRY 2 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08085 | SURRY 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08616 | SURRY 2 - FILTER TECH SPECS | | 3 | |
| 08517 | SURRY 2 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 3 | |
| 11092 | SURRY 2 - FIRE PROTECTION SER SUPPLEMENT | | 2 | |
| 11975 | SURRY 2 - REACTOR CAVITY SEAL RING GENERIC ISSUE | | 1 | |
| 08520 | SURRY 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 10054 | SURRY 2 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 10492 | SURRY 2 - VITAL AREA ANALYSIS | | 1 | |
| 08336 | SURRY 2 - 10 CFR 50.55A(G) - INSERVICE INSPECTION | | 1 | |
| 01510 | SURRY 2 - TECH SPEC SURVEILLANCE REQMTS. FOR HYDRAULIC SNUBBERS | | 3 | |
| 03217 | SURRY 2 - INSERVICE TESTING - PUMPS AND VALVES - IST | | 1 | |
| 12180 | SURRY 2 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42719 | SURRY 2 - IE BULLETIN 79-27, LOSS OF NON CLASS II INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43941 | SURRY UNIT 2 - ATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42780 | SURRY 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42868 | SURRY 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43154 | SURRY 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42469 | SURRY 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SURRY 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--------------------------------------|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44060 | SURRY 2 - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | <u>TARGET</u> |
| 44131 | SURRY 2 - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44202 | SURRY 2 - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44273 | SURRY 2 - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44343 | SURRY 2 - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44410 | SURRY 2 - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44481 | SURRY 2 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44552 | SURRY 2 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44623 | SURRY 2 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44695 | SURRY 2 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44734 | SURRY 2 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44820 | SURRY 2 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44891 | SURRY 2 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44962 | SURRY 2 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45033 | SURRY 2 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45104 | SURRY 2 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45174 | SURRY 2 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45252 | SURRY 2 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45298 | SURRY 2 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45384 | SURRY 2 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45415 | SURRY 2 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS ON TURBINE TRIP | | |
| 45464 | SURRY 2 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45494 | SURRY 2 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45524 | SURRY 2 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45670 | SURRY 2 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45867 | SURRY 2 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45986 | SURRY 2 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46057 | SURRY 2 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46129 | SURRY 2 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46201 | SURRY 2 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46273 | SURRY 2 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46345 | SURRY 2 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46416 | SURRY 2 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46485 | SURRY 2 - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| 12640 | SURRY 2 - AUXILIARY FEEDWATER SYSTEM | | 1 | 03/31/80 |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42186 | SURRY 2 - DEFINE OPERABLE | | | <u>COMPLETE</u> |
| 43491 | SURRY 2 - EXTEND SURVEILLANCE INTERVAL FOR SNUBBERS | | | 02/06/81 |
| 43206 | SURRY 2 - WATER LEVEL OVER SPENT FUEL | | | 03/16/81 |
| 4280 | SURRY 2 - SYSTEM NUCLEAR SAFETY AND OPERATING COMMITTEE | | 2 | 05/12/81 |
| | | | | 05/22/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 41039 | SURRY 2 - CHANGE FUEL ENRICHMENT | | | <u>TARGET</u> |
| 43114 | SURRY 2 - HIGH ENERGY BREAK INSPECTION PROGRAM TECH SPEC 4.15-A | | 2 | |
| 43116 | SURRY 2 - FIRE PUMP TECH SPEC 4.18.B.1.F(2) | | 1 | |
| 43204 | SURRY 2 - LESSONS LEARNED TECH SPECS | | 1 | |
| 43817 | SURRY 2 - ECCS - FQ CHANGE | | | |

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TECHNICAL ASSIGNMENT CONTROL SYSTEM

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(CONTINUATION)

SURRY 2

TAC # TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ACTIVE ACTIONS (CONTINUATION)

TARGET

42563 SURRY 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: THREE MILE ISLAND 1

PLANT LOCATION: 10 MI SE OF HARRISBURG, PA
 DOCKET NUMBER: 050-00289
 ARCH/ENGINEER: GIL
 IE INSPECTOR: D. HAVERKAMP

LICENSED POWER: 2535 MWT
 DESIGN POWER: 0819 MWE
 NSSS VENDOR: B&W

PROJECT MANAGER: D. DIANNI
 BRANCH CHIEF: R. REID
 LIC. ASSISTANT: R. INGRAM

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11376 | THREE MILE ISLAND 1 - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | <u>COMPLETE</u> 01/22/81 |
| 12147 | THREE MILE ISLAND 1 - UPGRADED EMERGENCY PLAN | | 1 | 01/26/81 |
| 08710 | THREE MILE ISLAND 1 - CONTAINMENT LEAK TESTING-APP J (GENERIC) | | 2 | 03/30/81 |
| 12879 | THREE MILE ISLAND 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 10562 | THREE MILE ISLAND 1 - CONTINGENCY PLAN REVIEW | | 1 | 04/22/81 |
| 10568 | THREE MILE ISLAND 1 - GUARD TRAINING PLAN REVIEW | | 1 | 04/22/81 |
| 08086 | THREE MILE ISLAND 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | 05/11/81 |
| 08600 | THREE MILE ISLAND 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | 05/15/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42127 | THREE MILE ISLAND 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | <u>TARGET</u> |
| 43043 | THREE MILE ISLAND 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 43654 | THREE MILE ISLAND 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 10384 | THREE MILE ISLAND 1 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 08156 | THREE MILE ISLAND 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 12601 | THREE MILE ISLAND 1 - ANAL OF TURBINE DISC CRACKS | | 2 | |
| 12737 | THREE MILE ISLAND 1 - ECCS CLAD SWELLING & RUPTURE | | 2 | |
| 43262 | THREE MILE ISLAND 1 - B&W ACCIDENT INDUCED NEUTRON FLUX ERRORS (MULTI-PLANT ITEM B-64) | | 3 | |
| 11238 | THREE MILE ISLAND 1 - FIRE PROTECTION SER SUPPLEMENT | | 1 | |
| 10969 | THREE MILE ISLAND 1 - DEGRADED GRID VOLTAGE | | 2 | |
| 11287 | THREE MILE ISLAND 1 - INSERVICE TESTING | | 1 | |
| 43921 | THREE MILE ISLAND UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42781 | THREE MILE ISLAND 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 10850 | THREE MILE ISLAND 1 - CONTROL ROD GUIDE TUBE WEAR | | 3 | |
| 42710 | THREE MILE ISLAND 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 08708 | THREE MILE ISLAND 1 - CONVERSION TO STANDARD TECH SPECS | | 3 | |
| 11286 | THREE MILE ISLAND 1 - TS FOR HYDRAULIC SNUBBERS | | 3 | |
| 10565 | THREE MILE ISLAND 1 - VITAL AREA ANALYSIS | | 1 | |
| 42919 | THREE MILE ISLAND 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 06495 | THREE MILE ISLAND 1 - SPENT FUEL CASK DROP | | 3 | |
| 42602 | THREE MILE ISLAND 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |
| 42513 | THREE MILE ISLAND 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
|-----------------------|---|-------------|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44411 | THREE MILE ISLAND 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | <u>TARGET</u> |
| 44624 | THREE MILE ISLAND 1 - NUREG-0737 II.D.1.2, RELIEF & SAFETY VALVE TESTING | | | |
| 44710 | THREE MILE ISLAND 1 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |
| 44756 | THREE MILE ISLAND 1 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 45210 | THREE MILE ISLAND 1 - NUREG-0737 II.K.2.14, LIFT FREQUENCY OF PORV'S AND SV'S | | | |
| 45274 | THREE MILE ISLAND 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45320 | THREE MILE ISLAND 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |

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(CONTINUATION)

THREE MILE ISLAND 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|----------------------------------|--|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| (CONTINUATION) | | | | |
| 45671 | THREE MILE ISLAND 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45987 | THREE MILE ISLAND 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46058 | THREE MILE ISLAND 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46130 | THREE MILE ISLAND 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46202 | THREE MILE ISLAND 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46274 | THREE MILE ISLAND 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 03663 | THREE MILE ISLAND 1 - REVIEW GAS VENT HEADER SYSTEM PROBLEMS AND FOLLOW | | 1 | 10/10/80 |
| 10793 | THREE MILE ISLAND 1 - RING GIRDER SURVEILLANCE SCHEDULE | | 1 | 10/31/80 |
| 11842 | THREE MILE ISLAND 1 - PURGE ISOLATION VALVE | | | 11/05/80 |
| 42550 | THREE MILE ISLAND 1 - LEAKAGE RATE CLARIFICATION TSC | | | 01/06/81 |
| 12493 | THREE MILE ISLAND 1 - SNUBBER DELETION ON NONSAFETY SYSTEM | | 1 | 01/19/81 |
| 43407 | THREE MILE ISLAND 1 - SLUG FLOW EFFECTS IN OTSG | | | 02/09/81 |
| 42548 | THREE MILE ISLAND 1 - PROTECTION CHANNEL TESTING | | | 02/11/81 |
| 42549 | THREE MILE ISLAND 1 - SAMPLING OF SECONDARY COOLANT | | | 02/11/81 |
| 42365 | THREE MILE ISLAND 1 - RESPIRATORY PROTECTION CANNISTERS | | | 02/25/81 |
| 43474 | THREE MILE ISLAND 1 - II K.2.19 BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW | | | 03/02/81 |
| 42547 | THREE MILE ISLAND 1 - SOURCE ALPHA EMITTING LEVEL | | 2 | 03/30/81 |
| 41015 | THREE MILE ISLAND 1 - SURVEILLANCE OF REACTOR INTERNAL VENT VALVES | | 1 | 04/19/81 |
| 42264 | THREE MILE ISLAND 1 - FA HOLDDOWN SPRINGS | | | 04/20/81 |
| 07626 | THREE MILE ISLAND 1 - ENVIRONMENTAL TECH SPECS | | 1 | 05/15/81 |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 43968 | TMI-1 CYCLE 5 RELOAD REVIEW | | | |
| 43377 | TMI 1 - TSCR-98 REPLACING MCI-ED WITH GPUNC | | 1 | |
| 43814 | THREE MILE ISLAND 1 - TSCR 102 RELIEF FROM CARBONS SAMPLING | | 1 | |
| 43815 | TMI-1 TSCR 99 ADMINISTRATIVE CHANGES | | | |
| 11962 | THREE MILE ISLAND 1 - STATION ELECTRIC DISTRIBUTION SYSTEM ADEQUACY | | | |
| 43768 | THREE MILE ISLAND 1 - NUREG 0694 IG1 | | | |
| 41016 | THREE MILE ISLAND 1 - TEMPERATURE DETECTORS INSIDE CONTAINMENT | | 1 | |
| 43432 | THREE MILE ISLAND 1 - ITEM II K.2.13 THERMAL-MECH EFFECTS OF HPI ON VESSEL | | | |
| 43767 | THREE MILE ISLAND 1 - TECH SPEC CHANGE REQUEST 100 | | 1 | |
| 43813 | THREE MILE ISLAND 1 - TSCR 101 EXTEND REFUELING TO RESTART | | 1 | |
| 43953 | TMI-1 TSCR 103 RESTART ITEMS IN NU REG 0680 SUPPL 1, 2 & 3 | | | |
| 42363 | THREE MILE ISLAND 1 - BORON DILUTION ANALYSIS | | | |
| 43829 | THREE MILE ISLAND 1 - TASK ACTION ITEM ID.1 | | | |
| 43056 | THREE MILE ISLAND 1 - STEAM GENERATOR INSP. TSCR 94 | | 2 | |
| 42963 | THREE MILE ISLAND 1 - NUREG 0578 CATEGORY A TECHNICAL SPECIFICATIONS | | 1 | |

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CONDENSED MANAGEMENT REPORT

FACILITY: TROJAN

PLANT LOCATION: 31 MI N OF PORTLAND, OR
 DOCKET NUMBER: 050-00344
 ARCH/ENGINEER: BECH
 IE INSPECT: M. MALMROS

LICENSED POWER: 3411 MW
 DESIGN POWER: 1130 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: C. TRAMMELL
 BRANCH CHIEF: R. CLARK
 LIC. ASSISTANT: P. KREUTZER

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 10558 | TROJAN - CONTINGENCY PLAN REVIEW | | 1 | 10/16/80 |
| 11073 | TROJAN - FIRE PROTECTION SER SUPPLEMENT | | 1 | 10/06/80 |
| 11880 | TROJAN - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 10/21/80 |
| 06678 | TROJAN - SECONDARY WATER CHEMISTRY CONTROL (PWR GENERIC) | | 2 | 10/28/80 |
| 08685 | TROJAN - PWR PRESSURE TEMPERATURE LIMIT TECH SPECS | | 1 | 11/13/80 |
| 07717 | TROJAN - GENERIC FHA INSIDE CONTAINMENT | | 2 | 12/11/80 |
| 42119 | TROJAN - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 12/11/80 |
| 43035 | TROJAN - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 12/11/80 |
| 07328 | TROJAN - DIESEL GENERATOR SURVEILLANCE, GENERIC ISSUE (DEGRADED GRID VOLTAGE) | | 2 | 04/16/81 |
| 12939 | TROJAN - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 08321 | TROJAN - 10 CFR 50.55A(G) - INSERVICE INSPECTION GENERIC | | 1 | 05/08/81 |
| 10339 | TROJAN - LOSS OF 125V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | 05/13/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 07358 | TROJAN - PUMP SUPPORT-LAMELLAR TEARING | | 2 | |
| 07716 | TROJAN - GENERIC - HEAVY LOADS NEAR SPENT FUEL (A36) | | 2 | |
| 07721 | TROJAN - APPENDIX I TECH SPECS IMPLEMENTATION REVIEW | | 3 | |
| 10141 | TROJAN - TECH SPEC SURVEILLANCE PER FOR MECHANICAL SNUBBERS | | 2 | |
| 10555 | TROJAN - GUARD TRAINING PLAN REVIEW | | 1 | |
| 11245 | TROJAN - 10 CFR 50.55A(G)-INSERVICE TESTING(IST) | | 1 | |
| 11298 | TROJAN - HYDRAULIC SNUBBERS UPGRADE TECH SPECS | | 3 | |
| 11715 | TROJAN - THREE MILE ISLAND FOLLOW UP WORK | | 1 | |
| 12144 | TROJAN - EMERGENCY PLAN REVIEW | | 1 | |
| 12279 | TROJAN - ENV. QUAL. OF CONTROL SYSTEMS TROJAN TROJAN | | 3 | |
| 43660 | TROJAN - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 07199 | TROJAN - OVERPRESSURE PROTECTION REVIEW (GENERIC) | | 2 | |
| 42711 | TROJAN - IF BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 06596 | TROJAN - ANALYSIS OF ASYMMETRIC LOCA LOADS AND EFFECT ON REACTOR VESSEL SUPPORTS | | 2 | |
| 43933 | TROJAN - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42783 | TROJAN - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43155 | TROJAN - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 42502 | TROJAN - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 44061 | TROJAN - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44132 | TROJAN - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44203 | TROJAN - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44274 | TROJAN - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44344 | TROJAN - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44412 | TROJAN - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44482 | TROJAN - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44553 | TROJAN - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

TROJAN

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|---|---|----------|-------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44625 | TROJAN - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44654 | TROJAN - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44696 | TROJAN - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44735 | TROJAN - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44821 | TROJAN - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44892 | TROJAN - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44963 | TROJAN - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45034 | TROJAN - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45105 | TROJAN - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45175 | TROJAN - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45253 | TROJAN - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45299 | TROJAN - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45385 | TROJAN - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45416 | TROJAN - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45465 | TROJAN - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45495 | TROJAN - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45525 | TROJAN - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45672 | TROJAN - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45868 | TROJAN - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 46059 | TROJAN - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46131 | TROJAN - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46203 | TROJAN - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46275 | TROJAN - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46346 | TROJAN - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46417 | TROJAN - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46486 | TROJAN - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |
| 43091 | TROJAN - UPGRADED EMERGENCY SUPPORT FACILITIES (EOP/TSC/SPDS/NDL)-TMI ACTION PLAN | | | 01/15/81 |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|--------------------------|--|----------------|----------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42955 | TROJAN - ADMIN CHANGES JF 9-30-80 | | | COMPLETE 10/28/80 |
| 43207 | TROJAN - WATER LEVEL DURING REFUELING | | 1 | 12/11/80 |
| 43218 | TROJAN - LESSONS LEARNED - CAT. A TECH SPECS | | | 01/21/81 |
| 13152 | TROJAN - CONTROL BUILDING HEARING FOLLOW-UP WORK | | 1 | 02/13/81 |
| 06527 | TROJAN - REVISED CONTAINMENT TENDON SURVEILLANCE PER RG 1.35 REV. 2 | | 1 | 02/20/81 |
| 43449 | TROJAN - REVIEW CONTROL BLDG MODIFICATION 50.59 CHANGES AND PROBLEMS (PGE LETTERS 2-6-81 AND 2-9-81) | | 1 | 03/25/81 |
| 41023 | TROJAN - MOUNT ST. HELENS IMPACT | | 1 | 03/26/81 |
| 12654 | TROJAN - MISC CHANGES OF 12-28-79 | | 2 | 04/16/81 |
| 43696 | TROJAN - APPENDIX J AIR LOCK TESTING | | | 04/16/81 |
| 43444 | TROJAN - CYCLE 4. TWO FUEL ASSES WITH 3 SS RODS. LCA 70 | | 1 | 05/08/81 |
| 12390 | TROJAN - REVIEW COOLING TOWER PLUME REPT OF 10/23/79 | | | 05/14/81 |
| 42719 | TROJAN - DELETE WATER QUALITY ENVIRONMENTAL IS ALAB-515 (YELLOW CRCK) | | 2 | 05/14/81 |
| 43455 | TROJAN - NEW SAMPLE POINTS FOR RADIOLOGICAL ENV. MONITORING PGE 1-30-81 | | | 05/14/81 |
| 12368 | TROJAN - REVIEW STEAM GENERATOR TUBE DATA | | 1 | 05/19/81 |

| TAC # | TAC DESCRIPTION | PRIORITY | CRITICAL DATE TARGET |
|-----------------------|---|----------|-------------------------|
| <u>ACTIVE ACTIONS</u> | | | |
| 43273 | TROJAN - EVALUATION OF FLAW IN FEEDWATER PIPING (WCAP-9613) PGE 10-17-80 LETTER | | |
| 42996 | TROJAN - SURVEILLANCE SPECIMEN-CAPSULE U ANALYSIS (WCAP - 9469) | 3 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

TROJAN

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|---|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | (CONTINUATION) | | | <u>TARGET</u> |
| 43246 | TROJAN - BORON DILUTION | | 1 | |
| 12369 | TROJAN - SEISMIC RESTRAINTS/WALL PROBLEM | | 1 | |
| 43461 | TROJAN - IMPLEMENT IAEA SAFEGUARDS | | 1 | |
| 42594 | TROJAN - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |

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CONDENSED MANAGEMENT REPORT

FACILITY: TURKEY POINT 3

PLANT LOCATION: 25 MI S OF MIAMI, FLA
 DOCKET NUMBER: 050-00250
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. VOGTLOWELL

LICENSED POWER: 2200 MWT
 DESIGN POWER: 0693 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: M. GROTENHAUS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|--|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11881 | TURKEY POINT 3 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | <u>COMPLETE</u> 10/21/80 |
| 10392 | TURKEY POINT 3 - CONTINGENCY PLAN REVIEW | | 1 | 11/28/80 |
| 10202 | TURKEY POINT 3 - CONTAINMENT PURGE | | 1 | 12/08/80 |
| 10276 | TURKEY POINT 3 - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 01/28/81 |
| 08686 | TURKEY POINT 3 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 01/29/81 |
| 11716 | TURKEY POINT 3 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 04/01/81 |
| 12152 | TURKEY POINT 3 - UPGRADED EMERGENCY PLAN | | 1 | 04/01/81 |
| 12940 | TURKEY POINT 3 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 11081 | TURKEY POINT 3 - FIRE PROTECTION SER SUPPLEMENT | | 1 | 04/26/81 |
| 13036 | TURKEY POINT 3 - CONTROL ROD MISALIGNMENT TECH SPECS | | 2 | 05/06/81 |
| 43013 | TURKEY POINT 3 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 05/12/81 |
| 08786 | TURKEY POINT 3 - TECH SPECS SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | 05/14/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 03963 | TURKEY POINT 3 - FILTER TECH SPECS | | 3 | <u>TARGET</u> |
| 07437 | TURKEY POINT 3 - BLOCKING SI SIGNAL DURING COOLDOWN | | 2 | |
| 08088 | TURKEY POINT 3 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08255 | TURKEY POINT 3 - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | |
| 08326 | TURKEY POINT 3 - INSERVICE INSPECTION EVALUATION TESTING GENERIC | | 2 | |
| 08779 | TURKEY POINT 3 - CONTAINMENT LEAK TESTING - APP J (GENERIC) | | 1 | |
| 08782 | TURKEY POINT 3 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08784 | TURKEY POINT 3 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS | | 2 | |
| 08793 | TURKEY POINT 3 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10057 | TURKEY POINT 3 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 10340 | TURKEY POINT 3 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 11293 | TURKEY POINT 3 - 10 CFR 50.55A 1ST PUMP AND VALVE | | 1 | |
| 12280 | TURKEY POINT 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE TURKEY POINT 3 | | 1 | |
| 12574 | TURKEY POINT 3 - ANALYSIS OF TURBINE DISC CRACKS | | 3 | |
| 12964 | TURKEY POINT 3 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 42103 | TURKEY POINT 3 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43649 | TURKEY POINT 3 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 2 | |
| 42322 | TURKEY POINT 3 - LESSONS LEARNED CAT. B | | 1 | |
| 42712 | TURKEY POINT 3 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 1 | |
| 43913 | TURKEY POINT UNIT 3 - STATION BLACKOUT PROCEDURES & TRAINING | | 2 | |
| 43849 | TURKEY POINT 3 - SAFE SHUTDOWN TO SDV-1V COUP | | | |
| 42784 | TURKEY POINT 3 - ESF RESET CONTROL DESIGN DEF. CONC. - I&E BULLETIN 80-06 | | 1 | |
| 42869 | TURKEY POINT 3 - MASONRY WALL DESIGN. RESPD' / IE BULLETIN 80-11 | | 3 | |
| 43156 | TURKEY POINT 3 - PWR FEEDWATER LINE CRACKS / TERM CORRECTIVE ACTION | | 2 | |
| 10394 | TURKEY POINT 3 - VITAL AREA ANALYSIS | | 2 | |
| 08127 | TURKEY POINT 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 1 | |
| 42470 | TURKEY POINT 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 3 | |
| | | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

TURKEY POINT 3

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|-----------------------|-----------------|--|----------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 42324 | TURKEY POINT 3 | - AUX FEED SYST REVIEW | 1 | |
| 44062 | TURKEY POINT 3 | - NUREG-0737 I.A. 1.1 STA LONG TERM REQUIREMENTS | | |
| 44133 | TURKEY POINT 3 | - NUREG-0737 I.A. 3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44204 | TURKEY POINT 3 | - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44275 | TURKEY POINT 3 | - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44345 | TURKEY POINT 3 | - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44413 | TURKEY POINT 3 | - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | |
| 44483 | TURKEY POINT 3 | - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44554 | TURKEY POINT 3 | - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44626 | TURKEY POINT 3 | - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44697 | TURKEY POINT 3 | - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44736 | TURKEY POINT 3 | - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44822 | TURKEY POINT 3 | - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44893 | TURKEY POINT 3 | - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44964 | TURKEY POINT 3 | - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | |
| 45035 | TURKEY POINT 3 | - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45106 | TURKEY POINT 3 | - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45176 | TURKEY POINT 3 | - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45254 | TURKEY POINT 3 | - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | |
| 45300 | TURKEY POINT 3 | - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | |
| 45386 | TURKEY POINT 3 | - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45417 | TURKEY POINT 3 | - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | |
| 45466 | TURKEY POINT 3 | - NUREG-0737 II.K.3.9, PID CONTROLLER | | |
| 45496 | TURKEY POINT 3 | - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45526 | TURKEY POINT 3 | - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45673 | TURKEY POINT 3 | - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45869 | TURKEY POINT 3 | - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | |
| 45988 | TURKEY POINT 3 | - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46060 | TURKEY POINT 3 | - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46132 | TURKEY POINT 3 | - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46204 | TURKEY POINT 3 | - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | |
| 46276 | TURKEY POINT 3 | - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46347 | TURKEY POINT 3 | - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46418 | TURKEY POINT 3 | - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46487 | TURKEY POINT 3 | - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE COMPLETE |
|--------------------------|-----------------|---|----------|------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42271 | TURKEY POINT 3 | - GREEN TICKET FM BEELAND | | 10/08/80 |
| 43064 | TURKEY POINT 3 | - STEAM GENERATOR INSPECTION (10-22880) | | 10/30/80 |
| 12995 | TURKEY POINT 3 | - DRAFT EIS FOR S.G. REPAIR PROGRAM | 1 | 12/18/80 |
| 13082 | TURKEY POINT 3 | - STEAM GENERATOR REPAIR PROGRAM SE UPDATES | 1 | 12/18/80 |
| 11453 | TURKEY POINT 3 | - FUEL RESIDENCE TIME | 1 | 12/19/80 |
| 10994 | TURKEY POINT 3 | - CONTAINMENT STRUCTURE POST-TENSIGNING SYSTEM 5TH YEAR SURVEILLANCE | 1 | 01/28/81 |
| 43405 | TURKEY POINT 3 | - REVIEW LETTER DATED 12-30-80 RE SECURITY PLAN CHANGE | | 02/03/81 |
| 43462 | TURKEY POINT 3 | - EXTENSION OF MODIFICATION COMPLETION DATE | | 02/13/81 |
| 43281 | TURKEY POINT 3 | - STEAM GENERATOR REPAIR FINAL ENVIRONMENTAL STATEMENT (FES) | 1 | 03/30/81 |
| 43544 | TURKEY POINT 3 | - RCP FLYWHEEL INSPECTION-CHANGE TO CONFORM TO RG 1.14 AND ASME CODE SECTION XI | | 04/16/81 |
| 43787 | TURKEY POINT 3 | - ORGANIZATION CHART UPDATE | | 05/12/81 |

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(CONTINUATION)

TURKEY POINT 3

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|----------------------------|---|-----------------------|-----------------|-------------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 06541 | TURKEY POINT 3 - POWER RANGE INSTRUMENT CHECK | | 2 | <u>TARGET</u> |
| 08810 | TURKEY POINT 3 - SINGLE FAILURE | | 1 | |
| 10902 | TURKEY POINT 3 - PERIODIC TESTING OF DIESEL GENERATOR UNITS | | 1 | |
| 43807 | TURKEY POINT 3 - REQ FOR EXEMPTION TO 50.48 AND APP R TO PART 50 | | 1 | |
| 43403 | TURKEY POINT 3 - STEAM GENERATOR INSPECTION LICENSE CONDITIONS | | 1 | |
| 43478 | TURKEY POINT 3 - STEAM GENERATOR INSPECTION | | 1 | |
| 43546 | TURKEY POINT 3 - SG TUBE PLUGGING LIMIT | | | |
| 43805 | TURKEY POINT 3 - FIRE PROTECTION TECH. SPEC. | | | |
| 43844 | TURKEY POINT 3 - AFG TECH SPEC REVIEW | | | |
| 43826 | TURKEY POINT 3 - OVER PRESSURE MITIGATING SYSTEM - REVISION | | | |
| 11989 | TURKEY POINT 3 - HEARING RE TURKEY POINT 3 STEAM GENERATOR REPAIR | | 1 | |
| 42564 | TURKEY POINT 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |
| <u>ANTICIPATED ACTIONS</u> | | | | |
| 43974 | TURKEY POINT 3 - BASELOAD AND RADIAL BURN DOWN | | | <u>INITIATION</u> 05/02/81 |

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CONDENSED MANAGEMENT REPORT

FACILITY: TURKEY POINT 4

PLANT LOCATION: 25 MI S OF MIAMI, FLA
 DOCKET NUMBER: 050-00251
 ARCH/ENGINEER: BECH
 IE INSPECTOR: R. VOGTLOWELL

LICENSED POWER: 2200 MWT
 DESIGN POWER: 0693 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: M. GROTENHUIS
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|--|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11882 | TURKEY POINT 4 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | <u>COMPLETE</u> 10/21/80 |
| 10393 | TURKEY POINT 4 - CONTINGENCY PLAN REVIEW | | 1 | 11/28/80 |
| 10201 | TURKEY POINT 4 - CONTAINMENT PURGE AT POWER & EFFECT ON LOCA | | 1 | 12/08/80 |
| 08687 | TURKEY POINT 4 - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | 01/26/81 |
| 10277 | TURKEY POINT 4 - CONTAINMENT LEAKAGE DUE TO SEAL ON ON | | 3 | 01/28/81 |
| 11717 | TURKEY POINT 4 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 04/01/81 |
| 12153 | TURKEY POINT 4 - UPGRADED EMERGENCY PLAN | | 1 | 04/01/81 |
| 12941 | TURKEY POINT 4 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/20/81 |
| 11082 | TURKEY POINT 4 - FIRE PROTECTION, LK SUPPLEMENT | | 1 | 04/26/81 |
| 13037 | TURKEY POINT 4 - CONTROL ROD MISALIGNMENT TECH SPECS | | 2 | 05/06/81 |
| 43014 | TURKEY POINT 4 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 05/12/81 |
| 08787 | TURKEY POINT 4 - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | 05/14/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 08089 | TURKEY POINT 4 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | <u>TARGET</u> |
| 08256 | TURKEY POINT 4 - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | |
| 08287 | TURKEY POINT 4 - ISI 10 CFR 50.55(A) GENERIC GENERIC | | 1 | |
| 08780 | TURKEY POINT 4 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08783 | TURKEY POINT 4 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08785 | TURKEY POINT 4 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS | | 2 | |
| 08792 | TURKEY POINT 4 - BLOCKING SI SIGNAL DURING COOLDOWN | | 2 | |
| 08794 | TURKEY POINT 4 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10058 | TURKEY POINT 4 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH G... VOLTAGE | | 2 | |
| 10341 | TURKEY POINT 4 - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10397 | TURKEY POINT 4 - GUARD TRAINING PLAN REVIEW | | 1 | |
| 11294 | TURKEY POINT 4 - 10 CFR 50.55A IST PUMP AND VALVE PROGRAM PROGRAM | | 1 | |
| 12281 | TURKEY POINT 4 - HELB AND CONSEQUENTIAL SYSTEM FAILURE TURKEY POINT 4 | | 3 | |
| 12575 | TURKEY POINT 4 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 12965 | TURKEY POINT 4 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | |
| 42104 | TURKEY POINT 4 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | |
| 43650 | TURKEY POINT 4 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 08781 | TURKEY POINT 4 - FILTER TECH SPECS | | 3 | |
| 42323 | TURKEY POINT 4 - LESSONS LEARNED CAT. B | | 1 | |
| 43424 | TURKEY POINT 4 - LESSONS LEARNED CATEGORY A TECH SPECS | | 1 | |
| 42713 | TURKEY POINT 4 - IE BULLETIN 79-27, LOSS ON NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43914 | TURKEY POINT UNIT 4 - STATION BLACKOUT PROCEDURES & TRAINING | | 3 | |
| 42785 | TURKEY POINT 4 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42870 | TURKEY POINT 4 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 11990 | TURKEY POINT 4 - HEARING RE TURKEY POINT 4 STEAM GENERATOR REPAIR | | 3 | |
| 43157 | TURKEY POINT 4 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 10395 | TURKEY POINT 4 - VITAL AREA ANALYSIS | | 1 | |
| 08128 | TURKEY POINT 4 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 42471 | TURKEY POINT 4 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

TURKEY POINT 4

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 42325 | TURKEY POINT 4 - AUX FEED SYST REVIEW | | 1 | <u>TARGET</u> |
| 44063 | TURKEY POINT 4 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | |
| 44134 | TURKEY POINT 4 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44205 | TURKEY POINT 4 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44276 | TURKEY POINT 4 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44346 | TURKEY POINT 4 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44414 | TURKEY POINT 4 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44484 | TURKEY POINT 4 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44555 | TURKEY POINT 4 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |
| 44627 | TURKEY POINT 4 - NUREG-0737 II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | | |
| 44698 | TURKEY POINT 4 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | | |
| 44737 | TURKEY POINT 4 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | | |
| 44823 | TURKEY POINT 4 - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | | |
| 44894 | TURKEY POINT 4 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | | |
| 44965 | TURKEY POINT 4 - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR | | | |
| 45036 | TURKEY POINT 4 - NUREG-0737 II.F.1.2, IODINE/ PARTICULATE SAMPLING | | | |
| 45107 | TURKEY POINT 4 - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | | |
| 45177 | TURKEY POINT 4 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | | |
| 45255 | TURKEY POINT 4 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION | | | |
| 45301 | TURKEY POINT 4 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES | | | |
| 45387 | TURKEY POINT 4 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES | | | |
| 45418 | TURKEY POINT 4 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS | | | |
| 45467 | TURKEY POINT 4 - NUREG-0737 II.K.3.9, PID CONTROLLER | | | |
| 45497 | TURKEY POINT 4 - NUREG-0737 II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | | |
| 45527 | TURKEY POINT 4 - NUREG-0737 II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | | |
| 45674 | TURKEY POINT 4 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES | | | |
| 45870 | TURKEY POINT 4 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE | | | |
| 45989 | TURKEY POINT 4 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER | | | |
| 46061 | TURKEY POINT 4 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER | | | |
| 46133 | TURKEY POINT 4 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY | | | |
| 46205 | TURKEY POINT 4 - NUREG-0737 III.A.1.2, NUCLEAR DATA DATA | | | |
| 46277 | TURKEY POINT 4 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | | |
| 46348 | TURKEY POINT 4 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE | | | |
| 46419 | TURKEY POINT 4 - NUREG-0737 III.D.3.3, INPLANT RADIATION MONITORING | | | |
| 46488 | TURKEY POINT 4 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY | | | |
| <u>COMPLETED ACTIONS</u> | | | | |
| 42272 | TURKEY POINT 4 - GREEN TICKET FM BEELAND | | | <u>COMPLETE</u> |
| 12996 | TURKEY POINT 4 - DRAFT EIS FOR S.G. REPAIR PROGRAM | | 1 | 10/08/80 |
| 13083 | TURKEY POINT 4 - STEAM GENERATOR REPAIR PROGRAM SE UPDATES | | 1 | 12/18/80 |
| 11454 | TURKEY POINT 4 - FUEL RESIDENCE TIME | | 1 | 12/19/80 |
| 43270 | TURKEY POINT 4 - STEAM GENERATOR INSPECTION | | | 01/15/81 |
| 18995 | TURKEY POINT 4 - CONTAINMENT STRUCTURE PAST-TENSIONING SYSTEM 5TH YR SURVEILLANCE | | 1 | 01/28/81 |
| 43406 | TURKEY POINT 4 - REVIEW LETTER DATED 12-30-80 RE SECURITY PLAN CHANGE | | | 02/03/81 |
| 43463 | TURKEY POINT 4 - EXTENSION OF MODIFICATION COMPLETION DATE | | | 02/13/81 |
| 43280 | TURKEY POINT 4 - STEAM GENERATOR REPAIR FINAL ENVIRONMENTAL STATEMENT (FES) | | 1 | 03/30/81 |
| 43788 | TURKEY POINT 4 - ORGANIZATION CHART UPDATE | | | 05/12/81 |
| 43545 | TURKEY POINT 4 - RCP FLYWHEEL INSPECTION-CHANGE TO CONFORM TO RG 1.14 AND ASME CODE SECTION XI | | | 05/16/81 |

DATA AS OF - 05/31/81

(CONTINUATION)

TURKEY POINT 4

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|----------------------------|---|-----------------------|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 08808 | TURKEY POINT 4 - POWER RANGE INSTRUMENT CHECK | | 1 | <u>TARGET</u> |
| 08812 | TURKEY POINT 4 - SINGLE FAILURE IN BACKUP LINE (EGCS) | | 1 | |
| 10903 | TURKEY POINT 4 - PERIODIC TESTING OF DIESEL GENERATOR UNITS | | 1 | |
| 43808 | TURKEY POINT 4 - REQUEST FOR EXEMPTION TO 50, 48 APP R TO PART 50 | | 1 | |
| 43404 | TURKEY POINT 4 - STEAM GENERATOR INSPECTION LICENSE CONDITIONS | | 1 | |
| 43547 | TURKEY POINT 4 - SG TUBE PLUGGING LIMIT | | | |
| 43806 | TURKEY POINT 4 - FIRE PROTECTION TECH. SPEC. | | | |
| 43845 | TURKEY POINT A KFS TECH SPEC REVIEW. | | | |
| 43827 | TURKEY POINT 4 - OVER PRESSURE MITIGATING SYSTEM - REVISION | | | |
| 43850 | TURKEY POINT 4 SAFETY SHUTDOWN | | | |
| <u>ANTICIPATED ACTIONS</u> | | | | |
| 43971 | TURKEY POINT 4 - EXTENTION OF OPERATING INTERVAL | | | <u>INITIATION</u> |
| 43975 | TURKEY POINT 4 - BASELOAD AND RADIAL BURN DOWN | | | 06/01/81 |
| 43959 | TURKEY POINT 4 FEEDWATER NOZZLE CRACK ROI-80-26 | | | 06/01/81 |
| | | | | 06/12/81 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: VERMONT YANKEE 1

PLANT LOCATION: 5 MI S OF BRATTLEBORO, VT
 DOCKET NUMBER: 050-00271
 ARCH/ENGINEER: EBASCO
 IE INSPECTOR: W. RAYMOND

LICENSED POWER: 1593 MWT
 DESIGN POWER: 0514 MWE
 NSSS VENDOR: GE

PROJECT MANAGER: V. ROONEY
 BRANCH CHIEF: T. IPPOLITO
 LIC. ASSISTANT: S. SHEPPARD

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|---|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 11115 | VERMONT YANKEE - 12 FIRE PROTECTION SER SUPPLEMENT SER SUPPLEMENT | | 1 | <u>COMPLETE</u> 10/24/80 |
| 10440 | VERMONT YANKEE - CONTINGENCY PLAN REVIEW | | 1 | 11/21/80 |
| 43023 | VERMONT YANKEE - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 12/19/80 |
| 11385 | VERMONT YANKEE - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | 01/14/81 |
| 12110 | VERMONT YANKEE - APPENDIX G FRACTURE TOUGHNESS | | | 01/14/81 |
| 06988 | VERMONT YANKEE - 10 CFR 73.55 IMPLEMENTATION | | | 02/23/81 |
| 12419 | VERMONT YANKEE - LESSONS LEARNED IMPLEMENTATION | | 1 | 03/02/81 |
| 11760 | VERMONT YANKEE THREE MILE ISLAND FOLLOW-UP | | 1 | 03/24/81 |
| 10425 | VERMONT YANKEE - BWR FEEDWATER AND CONTROL ROD DRIVE NOZZLE CRACKING | | 1 | 04/01/81 |
| 12138 | VERMONT YANKEE - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 12312 | VERMONT YANKEE - HELB AND CONSEQUENTIAL SYSTEM FAILURE VERMONT YANKEE | | 3 | <u>TARGET</u> |
| 13147 | VERMONT YANKEE - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 2 | |
| 08816 | VERMONT YANKEE - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08324 | VERMONT YANKEE - INSERVICE INSPECTION (ISI) ENERIC | | 1 | |
| 06369 | VERMONT YANKEE - SINGLE LOOP OPERATION | | 3 | |
| 07950 | VERMONT YANKEE - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION | | 1 | |
| 42229 | VERMONT YANKEE - BWR SCRAM DISCHARGE VOLUME CAPABILITY | | 1 | |
| 08129 | VERMONT YANKEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 08820 | VERMONT YANKEE - HYDRAULIC SNUBBERS | | 3 | |
| 08939 | VERMONT YANKEE - RPS POWER SUPPLY | | 1 | |
| 10059 | VERMONT YANKEE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED DEGRADED GRID VOLTAGE | | 2 | |
| 10308 | VERMONT YANKEE - CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATIONS | | 3 | |
| 10370 | VERMONT YANKEE - LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10428 | VERMONT YANKEE - MECHANICAL SNUBBERS | | 2 | |
| 10433 | VERMONT YANKEE - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10437 | VERMONT YANKEE - VITAL AREA ANALYSIS | | 1 | |
| 11250 | VERMONT YANKEE - INSERVICE TESTING (IST) | | 1 | |
| 42714 | VERMONT YANKEE - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. | | 2 | |
| <u>RESPONSES</u> | | | | |
| 42786 | VERMONT YANKEE - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43724 | VERMONT YANKEE - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM | | | |
| 42888 | VERMONT YANKEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 42480 | VERMONT YANKEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>IMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44064 | VERMONT YANKEE - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44135 | VERMONT YANKEE - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44206 | VERMONT YANKEE - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44277 | VERMONT YANKEE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44347 | VERMONT YANKEE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44415 | VERMONT YANKEE - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

VERMONT YANKEE 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|-----------------------------|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44485 | VERMONT YANKEE - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | <u>TARGET</u> |
| 44556 | VERMONT YANKEE - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44628 | VERMONT YANKEE - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44824 | VERMONT YANKEE - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44895 | VERMONT YANKEE - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44966 | VERMONT YANKEE - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45037 | VERMONT YANKEE - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45108 | VERMONT YANKEE - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45178 | VERMONT YANKEE - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45388 | VERMONT YANKEE - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45554 | VERMONT YANKEE - NUREG-0737 | II.K.3.13, HPCI AND RCIC INITIATION LEVELS | | |
| 45582 | VERMONT YANKEE - NUREG-0737 | II.K.3.15, ISOLATION OF HPCI AND RCIC MODIFICATION | | |
| 45606 | VERMONT YANKEE - NUREG-0737 | II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES | | |
| 45675 | VERMONT YANKEE - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45702 | VERMONT YANKEE - NUREG-0737 | II.K.3.18, ADS ACTUATION STUDY | | |
| 45726 | VERMONT YANKEE - NUREG-0737 | II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION | | |
| 45750 | VERMONT YANKEE - NUREG-0737 | II.K.3.21, RESTART OF CSS AND LPCI | | |
| 45774 | VERMONT YANKEE - NUREG-0737 | II.K.3.25, POWER ON PUMP SEALS | | |
| 45798 | VERMONT YANKEE - NUREG-0737 | II.K.3.27, COMMON REFERENCE LEVEL | | |
| 45871 | VERMONT YANKEE - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45898 | VERMONT YANKEE - NUREG-0737 | II.K.3.44, TRANSIENTS WITH SINGLE FAILURES | | |
| 45922 | VERMONT YANKEE - NUREG-0737 | II.K.3.45, MANUAL DEPRESSURIZATION | | |
| 45990 | VERMONT YANKEE - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46062 | VERMONT YANKEE - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46134 | VERMONT YANKEE - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46206 | VERMONT YANKEE - NUREG-0737 | III.A.1.2, NUCLEAK DATA DATA | | |
| 46278 | VERMONT YANKEE - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46349 | VERMONT YANKEE - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46420 | VERMONT YANKEE - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46489 | VERMONT YANKEE - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|-----------------------|-----------------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 13081 | VERMONT YANKEE - ANALOG TRIP SYSTEM | | | <u>COMPLETE</u> |
| 42813 | VERMONT YANKEE - HYDROGEN MONITORING | | 1 | 10/13/80 |
| 41008 | VERMONT YANKEE - ORGANIZATION CHARTS | | 1 | 11/03/80 |
| 42015 | VERMONT YANKEE - RESPONSE TO I&E BULLETIN 79-08 I & E BULLETIN 79-08 | | 2 | 11/04/80 |
| 43120 | VERMONT YANKEE - SPIRAL RELOAD | | | 11/04/80 |
| 41007 | VERMONT YANKEE - CONTAINMENT LEAK RATE TESTING | | 2 | 11/10/80 |
| 42017 | VERMONT YANKEE - GENERIC CLARIFICATION OF "OPERABILITY" OF "OPERABILITY" | | 2 | 12/09/80 |
| 42718 | VERMONT YANKEE - RELOAD 7 | | | 12/19/80 |
| 42003 | VERMONT YANKEE - INTERACTION BETWEEN NON-SAFETY & SAFETY SYSTEM | | 1 | 12/19/80 |
| 42852 | VERMONT YANKEE - NDT CURVE | | | 12/30/80 |
| 42004 | VERMONT YANKEE - STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES | | 1 | 01/13/81 |
| 43450 | VERMONT YANKEE - CASK LIFTING DEVICE | | | 01/26/81 |
| 43457 | VERMONT YANKEE - STABILITY TEST | | 2 | 03/02/81 |
| 12801 | VERMONT YANKEE - ORGANIZATION CHART | | 1 | 03/11/81 |
| 42817 | VERMONT YANKEE - AQ INTERPRETATION | | 2 | 04/06/81 |
| | | | 2 | 04/21/81 |

DATA #5 OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

VERMONT YANKEE 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|----------------------------|---|-----------------------|-----------------|-------------------------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 41099 | VERMONT YANKEE - REVIEW OF CORPORATE CAPABILITIES | | 1 | <u>TARGET</u> |
| 42006 | VERMONT YANKEE - DIESEL GENERATOR RELIABILITY | | | |
| 43078 | VERMONT YANKEE - CURRENT EVENTS | | 1 | |
| 43436 | VERMONT YANKEE - NEW APPENDIX "R" TO 10CFR50 REGARDING PROTECTION FEATURES. | | | |
| 43506 | VERMONT YANKEE - STATION BLACKOUT (GENERIC LETTER 81-04) | | | |
| 43776 | VERMONT YANKEE - FEEDWATER & NOZZLE CRACKING -NUREG 0619 IMPLEMENTATION | | | |
| 08531 | VERMONT YANKEE - FASTER SCRAM TIMES | | 2 | |
| 11504 | VERMONT YANKEE - APPENDIX B TECH SPECS | | 3 | |
| 43239 | VERMONT YANKEE - T-QUENCHER PIPING SAFETY CLASSIFICATION | | 1 | |
| 43437 | VERMONT YANKEE - PERIODIC UPDATING OF FINAL SAFETY ANALYSIS REPORTS (FSARS) | | 1 | |
| 42582 | VERMONT YANKEE - LONG TERM REVIEW CONTAINMENT PUPGE AND VENT (B-24) | | | |
| 43308 | VERMONT YANKEE - RELOAD ANALYSIS METHODS | | 1 | |
| 46498 | EACH BOTTOM 2 - REPLACMNT OF CARBON STEEL PIPE IN CRD FLOW STABILIZER LOOP | | | |
| <u>ANTICIPATED ACTIONS</u> | | | | |
| 43993 | VERMONT YANKEE - STA ON FIRE BRIGADE | | | <u>INITIATION</u> 06/09/81 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: YANKEE-ROWE 1

PLANT LOCATION: 25 MI NE OF PITTSFIELD, MASS
 DOCKET NUMBER: 050-00029
 ARCH/ENGINEER: S&W
 IE INSPECTOR: T. FOLEY

LICENSED POWER: 0600 MWT
 DESIGN POWER: 0175 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: R. CARUSO
 BRANCH CHIEF: D. CRUTCHFIELD
 LIC. ASSISTANT: H. SMITH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|----------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 07760 | YANKEE ROWE - SPENT FUEL POOL EXPANSION | | 1 | COMPLETE 10/01/80 |
| 12921 | YANKEE ROWE - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 10/01/80 |
| 10188 | YANKEE ROWE - CONTAINMENT PURGE | | 1 | 11/01/80 |
| 10290 | YANKEE ROWE - CONTAINMENT LEAKAGE DUE TO SEAT DETERIORATION | | 3 | 11/01/80 |
| 11697 | YANKEE ROWE - TMI FOLLOW UP WORK | | 1 | 11/01/80 |
| 05889 | YANKEE ROWE - PWR STEAM GENERATOR AND REACTOR COOLANT PUMP SUPPORTS - GENERIC | | 1 | 12/01/80 |
| 10589 | YANKEE ROWE - CONTINGENCY PLAN REVIEW | | 1 | 12/15/80 |
| 07965 | YANKEE ROWE - FLOOD OF EQUIP IMPORTANT TO SAFETY | | 2 | 12/18/80 |
| 10728 | YANKEE ROWE - WESTINGHOUSE N-1 LOOP OPERATION | | 3 | 02/26/81 |
| 12294 | YANKEE ROWE - HELB AND CONSEQUENTIAL SYSTEM FAILURE YANKEE ROWE | | 3 | 02/26/81 |
| 43054 | YANKEE ROWE - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | 02/26/81 |
| 12701 | YANKEE ROWE - ECCS CLAD SWELLING & RUPTURE | | 2 | 02/28/81 |
| 11103 | YANKEE ROWE - FIRE PROTECTION SER SUPPLEMENT | | 1 | 04/01/81 |
| 12139 | YANKEE ROWE - EMERGENCY PLAN REVIEW | | 1 | 04/01/81 |
| 12440 | YANKEE ROWE - LESSONS LEARNED IMPLEMENTATION | | 1 | 04/01/81 |
| 42132 | YANKEE ROWE - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | 04/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 10073 | YANKEE ROWE - ENHANCED FISSION PRODUCT RELEASE FOR HIGH BURNUP LWR FUEL | | | TARGET |
| 43672 | YANKEE ROWE - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 07045 | YANKEE ROWE - GENERIC REVIEW - BORON DILUTION INCIDENTS (ALL PWRs) | | 3 | |
| 07259 | YANKEE ROWE - INSERVICE INSPECTION PROGRAM TO MEET 10 CFR 50.55A | | 1 | |
| 10060 | YANKEE ROWE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 08104 | YANKEE ROWE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| 11230 | YANKEE ROWE - INSERVICE TESTING (IST) | | 1 | |
| 08591 | YANKEE ROWE - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 08091 | YANKEE ROWE - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 08694 | YANKEE ROWE - PWR PRESSURE-TEMPERATURE TECH SPECS | | 1 | |
| 10149 | YANKEE ROWE - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | |
| 10352 | YANKEE ROWE - LOSS OF 125-V DC BUS VOLTAGE W/LOSS OF ANNUNCIATOR SYSTEM | | 3 | |
| 10585 | YANKEE ROWE - GUARD TRAINING PLAN REVIEW | | 1 | |
| 10147 | YANKEE ROWE - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08302 | YANKEE ROWE - TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS | | 3 | |
| 10151 | YANKEE ROWE - TECH SPECS SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS | | 2 | |
| 10586 | YANKEE ROWE - VITAL AREA ANALYSIS | | 1 | |
| 08257 | YANKEE ROWE - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC | | 2 | |
| 12583 | YANKEE ROWE - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 42715 | YANKEE ROWE - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 43626 | YANKEE ROWE - APPENDIX R-ALTERNATE SAFE SHUTDOWN REVIEW | | 1 | |
| 43947 | YANKEE-ROWE - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 42787 | YANKEE ROWE - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 42917 | YANKEE ROWE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 43158 | YANKEE ROWE - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 11365 | YANKEE ROWE - REACTOR VESSEL BELTLINE MATERIAL SURVEILLANCE | | 3 | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

YANKEE-ROWE 1

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------------------|--|-------------|----------|---------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 42526 | YANKEE ROWE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | TARGET |
| 11539 | YANKEE ROWE - SPENT FUEL POOL EXPANSION | | 1 | |

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE |
|-----------------------|--------------------------|---|----------|---------------|
| <u>ACTIVE ACTIONS</u> | | | | |
| 44065 | YANKEE ROWE - NUREG-0737 | I.A.1.1 STA LONG TERM REQUIREMENTS | | TARGET |
| 44136 | YANKEE ROWE - NUREG-0737 | I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | |
| 44207 | YANKEE ROWE - NUREG-0737 | I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | |
| 44278 | YANKEE ROWE - NUREG-0737 | I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | |
| 44348 | YANKEE ROWE - NUREG-0737 | I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | |
| 44416 | YANKEE ROWE - NUREG-0737 | II.B.1, RCS HIGH POINT VENTS | | |
| 44486 | YANKEE ROWE - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44557 | YANKEE ROWE - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44629 | YANKEE ROWE - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44655 | YANKEE ROWE - NUREG-0737 | II.E.1.1, AFW SYSTEM EVALUATION | | |
| 44699 | YANKEE ROWE - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44738 | YANKEE ROWE - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44825 | YANKEE ROWE - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44896 | YANKEE ROWE - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44967 | YANKEE ROWE - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45038 | YANKEE ROWE - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45109 | YANKEE ROWE - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45179 | YANKEE ROWE - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45256 | YANKEE ROWE - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45302 | YANKEE ROWE - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45389 | YANKEE ROWE - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45419 | YANKEE ROWE - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45468 | YANKEE ROWE - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45498 | YANKEE ROWE - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45528 | YANKEE ROWE - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45676 | YANKEE ROWE - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45872 | YANKEE ROWE - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45991 | YANKEE ROWE - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46063 | YANKEE ROWE - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46135 | YANKEE ROWE - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46207 | YANKEE ROWE - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46279 | YANKEE ROWE - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46350 | YANKEE ROWE - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46421 | YANKEE ROWE - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 46490 | YANKEE ROWE - NUREG-0737 | III.D.3.4, CONTROL ROOM HABITABILITY | | |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE |
|--------------------------|--|----------------|----------|---------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42853 | YANKEE ROWE - OPERABILITY T.S. AMENDMENT | | | COMPLETE |
| 43284 | YANKEE ROWE - LOCA MODEL CHANGE | | | 12/01/80 |
| 43746 | YANKEE ROWE - EXTENSION OF CORE XIV LOCA LIMITS TO 16300 MWD/MTU | | | 01/23/81 |
| | | | | 04/28/81 |

ACTIVE ACTIONS

TARGET

TECHNICAL ASSIGNMENT CONTROL SYSTEM
CONDENSED MANAGEMENT REPORT

DATA AS OF - 05/31/81

(CONTINUATION)

YANKEE-ROWE 1

TAC # TAC DESCRIPTIONPLANT SPECIFIC

| <u>ACTIVE ACTIONS</u> | <u>(CONTINUATION)</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|-----------------------|--|-----------------|----------------------|
| 43151 | YANKEE ROWE - DEFERRAL OF ISI T.S. REQUIREMENTS | | |
| 43256 | YANKEE ROWE - LOCA MODEL CHANGES | | |
| 43387 | YANKEE ROWE - EVALUATION OF LOOP RELIEF VALVES | 1 | |
| 43687 | YANKEE ROWE - RELOAD FOR CYCLE XV | 1 | |
| 42616 | YANKEE ROWE - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | 1 | <u>TARGET</u> |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: ZION 1

PLANT LOCATION: 40 MI N OF CHICAGO, ILL
 DOCKET NUMBER: 050-00295
 ARCH/ENGINEER: S&L
 IE INSPECTOR: J. KOHLER

LICENSED POWER: 3250 MW
 DESIGN POWER: 1040 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. WIGGINTON
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| TAC # | TAC DESCRIPTION | MULTI-PLANT | PRIORITY | CRITICAL DATE |
|--------------------------|---|-------------|----------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 08869 | ZION 1 - CONTROL ROD GUIDE TUBE WEAR | | 3 | <u>COMPLETE</u> 10/06/80 |
| 11883 | ZION 1 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 10/15/80 |
| 10741 | ZION 1 - CONTINGENCY PLAN REVIEW | | 1 | 11/17/80 |
| 10739 | ZION 1 - GUARD TRAINING PLAN REVIEW | | 1 | 01/15/81 |
| 08874 | ZION 1 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | 01/27/81 |
| 11718 | ZION 1 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 03/27/81 |
| 13007 | ZION 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | 04/09/81 |
| 12942 | ZION 1 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES VALVES | | 1 | 04/23/81 |
| 12129 | ZION 1 - EMERGENCY PLAN REVIEW | | 1 | 05/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42105 | ZION 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | <u>TARGET</u> |
| 43015 | ZION 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 08875 | ZION 1 - PWR AUXILIARY FEEDWATER PUMPS (GL-4) | | 1 | |
| 08862 | ZION 1 - CONTAINMENT LEAK TESTING - APP. J (GENERIC) | | 2 | |
| 12587 | ZION 1 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 43899 | ZION UNIT 1 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 08872 | ZION 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08864 | ZION 1 - MECHANICAL SNUBBERS | | 2 | |
| 10061 | ZION 1 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 11228 | ZION 1 - HYDRAULIC SNUBBERS UPGRADE TECH SPECS | | 3 | |
| 12282 | ZION 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ZION 1 | | 3 | |
| 42716 | ZION 1 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV & EVAL. RESPONSES | | 2 | |
| 10745 | ZION 1 - VITAL AREA ANALYSIS | | 1 | |
| 08092 | ZION 1 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42472 | ZION 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| 42788 | ZION 1 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43637 | ZION 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 43159 | ZION 1 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 11071 | ZION 1 - FIRE PROTECTION SUPPLEMENT #2 | | 1 | |
| 11225 | ZION 1 - INSERVICE TESTING (IST) | | 1 | |
| 06431 | ZION 1 - N-1 LOOP OPERATION | | 3 | |
| 42871 | ZION 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11 | | 2 | |
| 08130 | ZION 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| <u>TMI ACTIONS</u> | | | | |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44066 | ZION 1 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44137 | ZION 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44208 | ZION 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44279 | ZION 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44349 | ZION 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44417 | ZION 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 1

| TAC # | TAC DESCRIPTION | TMI ACTIONS | PRIORITY | CRITICAL DATE TARGET |
|--------------------------------------|--|---|----------|-------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 44487 | ZION 1 - NUREG-0737 | II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | |
| 44558 | ZION 1 - NUREG-0737 | II.B.4.1, TRAINING FOR MITIGATING CORE | | |
| 44630 | ZION 1 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44700 | ZION 1 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44739 | ZION 1 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44826 | ZION 1 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44897 | ZION 1 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44968 | ZION 1 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45039 | ZION 1 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45110 | ZION 1 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45180 | ZION 1 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45257 | ZION 1 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45303 | ZION 1 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45390 | ZION 1 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45420 | ZION 1 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45469 | ZION 1 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45499 | ZION 1 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45529 | ZION 1 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45677 | ZION 1 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45873 | ZION 1 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45992 | ZION 1 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46064 | ZION 1 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46136 | ZION 1 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46208 | ZION 1 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46280 | ZION 1 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46351 | ZION 1 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46422 | ZION 1 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 42838 | ZION 1 - COMPLIANCE WITH NRC LETTERS ON AFWs RELIABILITY IMPROVEMENTS (ORDER 2/29/80 ITEM F.5) | | 1 | 07/10/81 |
| 13084 | ZION 1 - CONTROL ROOM HABITABILITY (ORDER 2-29-80, ITEM B-2) | | 1 | 10/01/81 |

| TAC # | TAC DESCRIPTION | PLANT SPECIFIC | PRIORITY | CRITICAL DATE COMPLETE |
|--------------------------|---|----------------|----------|---------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 42136 | ZION 1 - REACTOR OPERATOR QUALIFICATIONS (ORDER 2/29/80 ITEM B.7) | | | 10/31/80 |
| 42138 | ZION 1 - RETAIN AND RECERTIFY REACTOR OPERATORS (ORDER 2/29/80 ITEM B.8) | | | 10/31/80 |
| 42826 | ZION 1 - CLARIFICATION OF ORDER ITEM A.3 GROSS LEAK CHECK OF CONTAINMENT | | | 12/08/80 |
| 42183 | ZION 1 - REACTOR VESSEL UPPER SHELF ENERGY LEVEL | | | 12/09/80 |
| 42140 | ZION 1 - STEAM GENERATOR OPERATING LEVELS (ORDER 2/29/80 ITEM C.1) | | | 12/18/80 |
| 43254 | ZION 1 - HIGH BURNUP FUEL-LAST CYCLE | | | 12/31/80 |
| 07180 | ZION 1 - SURVEILLANCE TESTING REQUIREMENTS ON HIGH FLUX (LOW SETPOINT) REACTOR TRIP | | 1 | 01/14/81 |
| 42298 | ZION 1 - REMOVE SURVEILLANCE FROM TECH SPEC FOR NON-EXISTENT VALVES | | | 01/14/81 |
| 43111 | ZION 1 - DELETION OF PH REQUIREMENT FROM ETS | | | 01/14/81 |
| 42142 | ZION 1 - CHARCOAL FILTERS CO-IMPREGNATION WITH KI AND I2 AND AMINE (ORDER 2/29/80 ITEM C.2) | | 1 | 01/15/81 |
| 43438 | ZION 1 - ROD EXCHANGE FOR RODWORTH | | 1 | 03/12/81 |
| 42144 | ZION 1 - EFFECT OF POWER REDUCTION ON PLANT STABILITY | | 1 | 03/27/81 |
| 42175 | ZION 1 - MISCELLANEOUS (27 PAGE) TECH SPEC ADMIN CHANGES | | 1 | 04/07/81 |
| 43521 | ZION 1 - LL CATEGORY A TECH SPEC | | | 04/20/81 |
| 13054 | ZION 1 - LOCA PEAKING FACTOR INCREASE TO 2.20 | | 1 | 05/11/81 |

ACTIVE ACTIONSTARGET

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 1

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> <u>TARGET</u> |
|--------------------------------------|---|-----------------------|-----------------|---------------------------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | |
| 42040 | ZION 1 - LESSONS LEARNED CATEGORY B ITEMS | | | |
| 43699 | ZION 1 - NOBLE GAS MONITOR INDICATION | | 1 | |
| 43786 | ZION 1 - ROD BOW PENALTY | | 1 | |
| 08048 | ZION 1 - COMPARISON OF FACILITY DESIGN AGAINST CURRENT CRITERIA PER ACRS RECOMMENDATION | | 1 | |
| 42306 | ZION 1 - CONTROL ROOM - HUMAN FACTORS ENGINEERING (ORDER 2/29/80 ITEM E.2) | | 1 | |
| 13021 | ZION 1 - STANDARD TECH SPEC LCO'S FOR TASK ACTION PLAN ITEM F.1 (F)(3) | | 2 | |
| 11986 | ZION 1 - ENVIRONMENTAL MONITORING SUMMARY REPORT 1970-1978 | | 1 | |
| 13144 | ZION 1 - QA PROGRAM RE-REVIEW PER ACTION PLAN (ITEM F.1(F)(8) | | 1 | |
| 41044 | ZION 1 - RESIDUAL RISKS | | 1 | |
| 42836 | ZION 1 - FAILURE MODE EFFECTS ANALYSIS (2/29/80 ORDER ITEM F.4) | | 1 | |
| 42146 | ZION 1 - ATWS INSTRUMENTATION MODIFICATIONS (ORDER 2/29/80 ITEM C.4) | | 1 | |
| 42566 | ZION 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | | | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT REPORT

FACILITY: ZION 2

PLANT LOCATION: 40 MI N OF CHICAGO, ILL
 DOCKET NUMBER: 050-00304
 ARCH/ENGINEER: S&L
 IE INSPECTOR: J. KOHLER

LICENSED POWER: 3250 MWT
 DESIGN POWER: 1040 MWE
 NSSS VENDOR: WEST

PROJECT MANAGER: D. WIGGINTON
 BRANCH CHIEF: S. VARGA
 LIC. ASSISTANT: C. PARRISH

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>MULTI-PLANT</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------|--|--------------------|-----------------|-----------------------------|
| <u>COMPLETED ACTIONS</u> | | | | |
| 08870 | ZION 2 - CONTROL ROD GUIDE TUBE WEAR | | 3 | <u>COMPLETE</u> 10/06/80 |
| 11884 | ZION 2 - REVIEW OF SAFETY ASPECTS OF INADVERTENT SAFETY ACTUATIONS DURING SURVEILLANCE TESTING | | 2 | 10/15/80 |
| 10742 | ZION 2 - CONTINGENCY PLAN REVIEW | | 1 | 11/17/80 |
| 10740 | ZION 2 - GUARD TRAINING PLAN REVIEW | | 1 | 01/15/81 |
| 08875 | ZION 2 - PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS | | 2 | 01/27/81 |
| 11719 | ZION 2 - THREE MILE ISLAND FOLLOW UP WORK | | 1 | 03/27/81 |
| 13008 | ZION 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES | | 2 | 04/09/81 |
| 12943 | ZION 2 - PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES | | 1 | 04/23/81 |
| 12130 | ZION 2 - EMERGENCY PLAN REVIEW | | 1 | 05/01/81 |
| <u>ACTIVE ACTIONS</u> | | | | |
| 42106 | ZION 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS | | 2 | <u>TARGET</u> |
| 43016 | ZION 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS | | 2 | |
| 08476 | ZION 2 - PWR AUXILIARY FEEDWATER PUMPS (GL-4) | | 1 | |
| 08863 | ZION 2 - CO' AINMENT LEAK TESTING - APP. J (GENEPIC) | | 2 | |
| 12588 | ZION 2 - ANALYSIS OF TURBINE DISC CRACKS | | 2 | |
| 43900 | ZION UNIT 2 - STATION BLACKOUT PROCEDURES & TRAINING | | | |
| 08873 | ZION 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL | | 2 | |
| 08865 | ZION 2 - MECHANICAL SNUBBERS | | 2 | |
| 10062 | ZION 2 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE | | 2 | |
| 11229 | ZION 2 - HYDRAULIC SNUBBERS UPGRADE TECH SPECS | | 3 | |
| 12283 | ZION 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ZION 2 | | 3 | |
| 42717 | ZION 2 - IE BULLETIN 79-27, LOSS OF NON CLASS IE INST & CONTR. POWER. REV. & EVAL. RESPONSES | | 2 | |
| 10744 | ZION 2 - VITAL AREA ANALYSIS | | 1 | |
| 08093 | ZION 2 - CONTROL OF HEAVY LOADS NEAR SPENT FUEL | | 2 | |
| 42473 | ZION 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21) | | 1 | |
| 42789 | ZION 2 - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06 | | 3 | |
| 43638 | ZION 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM | | 1 | |
| 43160 | ZION 2 - PWR FEEDWATER LINE CRACKS LONG TERM CORRECTIVE ACTION | | 2 | |
| 11072 | ZION 2 - FIRE PROTECTION SER SUPPLEMENT #2 | | 1 | |
| 11226 | ZION 2 - INSERVICE TESTING (IST) | | 1 | |
| 08844 | ZION 2 - N-1 LOOP OPERATION | | 3 | |
| 08131 | ZION 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW | | 3 | |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>ACTIVE ACTIONS</u> | | | | |
| 44067 | ZION 2 - NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS | | | <u>TARGET</u> |
| 44138 | ZION 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS | | | |
| 44209 | ZION 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING | | | |
| 44280 | ZION 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES | | | |
| 44350 | ZION 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES | | | |
| 44418 | ZION 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS | | | |
| 44488 | ZION 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS | | | |
| 44559 | ZION 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE | | | |

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 2

| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>TMI ACTIONS</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------------------------------|---|---|-----------------|----------------------|
| <u>ACTIVE ACTIONS</u> (CONTINUATION) | | | | <u>TARGET</u> |
| 44631 | ZION 2 - NUREG-0737 | II.D.1.2, RELIEF AND SAFETY VALVE TESTING | | |
| 44701 | ZION 2 - NUREG-0737 | II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION | | |
| 44740 | ZION 2 - NUREG-0737 | II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION | | |
| 44827 | ZION 2 - NUREG-0737 | II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS | | |
| 44898 | ZION 2 - NUREG-0737 | II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY | | |
| 44969 | ZION 2 - NUREG-0737 | II.F.1.1, NOBLE GAS MONITOR | | |
| 45040 | ZION 2 - NUREG-0737 | II.F.1.2, IODINE/ PARTICULATE SAMPLING | | |
| 45111 | ZION 2 - NUREG-0737 | II.F.1.3, CONTAINMENT HIGH RANGE MONITOR | | |
| 45181 | ZION 2 - NUREG-0737 | II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION | | |
| 45258 | ZION 2 - NUREG-0737 | II.K.3.1, AUTO PORV ISOLATION | | |
| 45304 | ZION 2 - NUREG-0737 | II.K.3.2, REPORT ON PORV FAILURES | | |
| 45391 | ZION 2 - NUREG-0737 | II.K.3.3, REPORT ON RV/SV FAILURES | | |
| 45421 | ZION 2 - NUREG-0737 | II.K.3.5, AUTO TRIP OF RCPS | | |
| 45470 | ZION 2 - NUREG-0737 | II.K.3.9, PID CONTROLLER | | |
| 45500 | ZION 2 - NUREG-0737 | II.K.3.10, ANTICIPATORY TRIP MODIFICATIONS | | |
| 45530 | ZION 2 - NUREG-0737 | II.K.3.12, ANTICIPATORY TRIP ON TURBINE TRIP | | |
| 45678 | ZION 2 - NUREG-0737 | II.K.3.17, ECC SYSTEM OUTAGES | | |
| 45874 | ZION 2 - NUREG-0737 | II.K.3.30, SB LOCA OUTLINE | | |
| 45993 | ZION 2 - NUREG-0737 | III.A.1.2, TECHNICAL SUPPORT CENTER | | |
| 46065 | ZION 2 - NUREG-0737 | III.A.1.2, OPERATIONAL SUPPORT CENTER | | |
| 46137 | ZION 2 - NUREG-0737 | III.A.1.2, EMERGENCY OPERATIONS FACILITY | | |
| 46209 | ZION 2 - NUREG-0737 | III.A.1.2, NUCLEAR DATA DATA | | |
| 46281 | ZION 2 - NUREG-0737 | III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE | | |
| 46352 | ZION 2 - NUREG-0737 | III.A.2.2, METEOROLOGICAL DATA UPGRADE | | |
| 46423 | ZION 2 - NUREG-0737 | III.D.3.3, INPLANT RADIATION MONITORING | | |
| 42839 | ZION 2 - COMPLIANCE WITH NRC LETTERS ON AFW'S RELIABILITY IMPROVEMENTS (ORDER 2/29/80 ITEM F.5) | | 1 | 07/10/81 |
| 13085 | ZION 2 - CONTROL ROOM HABITABILITY (ORDER 2-29-80, ITEM B-2) | | 1 | 10/01/81 |
| <u>TAC #</u> | <u>TAC DESCRIPTION</u> | <u>PLANT SPECIFIC</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
| <u>COMPLETED ACTIONS</u> | | | | <u>COMPLETE</u> |
| 42137 | ZION 2 - REACTOR OPERATOR QUALIFICATIONS (ORDER 2/29/80 ITEM B.7) | | | 10/31/80 |
| 42139 | ZION 2 - RETAIN AND RECERTIFY REACTOR OPERATORS (ORDER 2/29/80 ITEM B.8) | | | 10/31/80 |
| 42827 | ZION 2 - CLARIFICATION OF ORDER ITEM A.3 GROSS LEAK CHECK OF CONTAINMENT | | | 12/08/80 |
| 42184 | ZION 2 - REACTOR VESSEL UPPER SHELF ENERGY LEVEL | | | 12/09/80 |
| 42141 | ZION 2 - STEAM GENERATOR OPERATING LEVELS (ORDER 2/29/80 ITEM C.1) | | | 12/18/80 |
| 43255 | ZION 2 - HIGH BURNUP FUEL - LAST CYCLE | | | 12/31/80 |
| 08854 | ZION 2 - SURVEILLANCE TESTING REQUIREMENTS (RPS SYSTEM) | | 1 | 01/14/81 |
| 43112 | ZION 2 - DELETION OF PH REQUIREMENTS FROM ETS | | | 01/14/81 |
| 42143 | ZION 2 - CHARCOAL FILTERS CO-IMPREGNATION WITH KI AND I2 AND AMINE (ORDER 2/29/80 ITEM C.2) | | | 01/15/81 |
| 43439 | ZION 2 - ROD EXCHANGE FOR ROD WORTH | | 1 | 03/12/81 |
| 42145 | ZION 2 - EFFECT OF POWER REDUCTION ON PLANT STABILITY | | 1 | 03/27/81 |
| 42833 | ZION 2 - REVIEW OF LER'S PER ORDER OF 2/29/80 ITEM F.1 | | 1 | 03/27/81 |
| 42176 | ZION 2 - MISCELLANEOUS (27 PAGE) TECH SPEC ADMIN CHANGES | | 1 | 04/01/81 |
| 43520 | ZION 2 - LL CATEGORY A TECH SPEC | | | 04/20/81 |
| 13055 | ZION 2 - LOCA PEAKING FACTOR INCREASE TO 2.20 | | 1 | 05/11/81 |
| <u>ACTIVE ACTIONS</u> | | | | <u>TARGET</u> |
| 42041 | ZION 2 - LESSONS LEARNED CATEGORY B ITEMS | | | |
| 43700 | ZION 2 - NOBLE GAS MONITOR INDICATION | | | |

TECHNICAL ASSIGNMENT CONTROL SYSTEM
CONDENSED MANAGEMENT REPORT

R-1208620-001

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(CONTINUATION)

ZION 2

PLANT SPECIFIC

| <u>IAC #</u> | <u>TAC DESCRIPTION</u> | <u>PRIORITY</u> | <u>CRITICAL DATE</u> |
|--------------|---|-----------------|----------------------|
| | (CONTINUATION) | | <u>TARGET</u> |
| 43785 | ZION 2 - ROD BOW PENALTY | 1 | |
| 08851 | ZION 2 - COMPARISON OF FACILITY DESIGN AGAINST CURRENT CRITERIA PER ACBS RECOMMENDATION | 1 | |
| 42307 | ZION 2 - CONTROL ROOM - HUMAN FACTORS ENGINEERING (ORDER 2/29/80 ITEM E.2) | 1 | |
| 13022 | ZION 2 - STANDARD TECH SPEC LCOS FOR TASK ACTION PLAN ITEM F.1 (F)(3) | 2 | |
| 11987 | ZION 2 - ENVIRONMENTAL MONITORING SUMMARY REPORT 1970-1978 | 1 | |
| 42565 | TURKEY PT 4 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | 1 | |
| 41045 | ZION 2 - RESIDUAL RISKS | 1 | |
| 42837 | ZION 2 - FAILURE MODE EFFECTS ANALYSIS (2/29/80 ORDER ITEM F.4) | 1 | |
| 42147 | ZION 2 - ATMS INSTRUMENTATION MODIFICATIONS (ORDER 2/29/80 ITEM C.4) | 1 | |
| 42872 | ZION 2 - MASONRY WALL DESIGN - RESPONSE TO IE BULLETIN 30-11 | 1 | |
| 42567 | ZION 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24) | 1 | |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT SUMMARY REPORT

| FACILITY | COMPLETE | ACTIVE | ANTICIPATED |
|-------------------|----------|--------|-------------|
| ARKANSAS 1 | 37 | 95 | 0 |
| ARKANSAS 2 | 19 | 80 | 0 |
| BEAVER VALLEY 1 | 28 | 66 | 0 |
| BIG ROCK POINT 1 | 24 | 65 | 0 |
| BROWNS FERRY 1 | 15 | 92 | 0 |
| BROWNS FERRY 2 | 15 | 85 | 0 |
| BROWNS FERRY 3 | 14 | 82 | 0 |
| BRUNSWICK 1 | 10 | 76 | 2 |
| BRUNSWICK 2 | 9 | 77 | 3 |
| CALVERT CLIFFS 1 | 23 | 85 | 1 |
| CALVERT CLIFFS 2 | 16 | 74 | 0 |
| COOK 1 | 15 | 67 | 0 |
| COOK 2 | 17 | 70 | 0 |
| COOPER STATION | 13 | 66 | 0 |
| CRYSTAL RIVER 3 | 21 | 84 | 0 |
| DAVIS-BESSE 1 | 17 | 94 | 0 |
| DRESDEN 2 | 25 | 101 | 0 |
| DRESDEN 3 | 19 | 71 | 0 |
| DUANE ARNOLD | 11 | 80 | 0 |
| FARLEY 1 | 16 | 66 | 0 |
| FARLEY 2 | 0 | 45 | 3 |
| FITZPATRICK | 18 | 68 | 0 |
| FORT CALHOUN 1 | 18 | 59 | 1 |
| FORT ST VRAIN | 0 | 51 | 0 |
| GINNA | 22 | 71 | 0 |
| HADDAM NECK | 15 | 73 | 0 |
| HATCH 1 | 12 | 67 | 0 |
| HATCH 2 | 16 | 66 | 0 |
| INDIAN POINT 2 | 20 | 37 | 0 |
| INDIAN POINT 3 | 10 | 64 | 0 |
| KEWAUNEE | 11 | 77 | 0 |
| LA CROSSE | 11 | 69 | 2 |
| MAINE YANKEE | 13 | 71 | 0 |
| MCGUIRE 1 | 0 | 40 | 0 |
| MILLSTONE 1 | 21 | 64 | 2 |
| MILLSTONE 2 | 20 | 69 | 0 |
| MONTICELLO | 11 | 78 | 0 |
| NINE MILE POINT 1 | 26 | 66 | 0 |
| NORTH ANNA 1 | 24 | 56 | 0 |
| NORTH ANNA 2 | 15 | 41 | 0 |
| OCONEE 1 | 41 | 69 | 1 |
| OCONEE 2 | 38 | 68 | 1 |
| OCONEE 3 | 37 | 68 | 1 |
| OYSTER CREEK 1 | 13 | 70 | 0 |
| PALISADES | 13 | 82 | 1 |
| PEACH BOTTOM 2 | 13 | 63 | 0 |
| PEACH BOTTOM 3 | 13 | 64 | 0 |
| PILGRIM 1 | 16 | 65 | 0 |
| POINT BEACH 1 | 16 | 67 | 0 |
| POINT BEACH 2 | 15 | 68 | 0 |

DATA AS OF - 05/31/81

CONDENSED MANAGEMENT SUMMARY REPORT

| FACILITY | COMPLETE | ACTIVE | ANTICIPATED |
|---------------------|----------|--------|-------------|
| PRAIRIE ISLAND 1 | 17 | 72 | 0 |
| PRAIRIE ISLAND 2 | 15 | 72 | 0 |
| QUAD CITIES 1 | 26 | 66 | 1 |
| QUAD CITIES 2 | 24 | 64 | 2 |
| RANCHO SECO 1 | 19 | 87 | 1 |
| ROBINSON 2 | 14 | 75 | 0 |
| SALEM 1 | 22 | 67 | 0 |
| SALEM 2 | 0 | 41 | 0 |
| SAN ONOFRE 1 | 20 | 82 | 0 |
| SEQUOYAH 1 | 0 | 40 | 0 |
| ST LUCIE 1 | 21 | 72 | 0 |
| SURRY 1 | 17 | 73 | 0 |
| SURRY 2 | 16 | 71 | 0 |
| THREE MILE ISLAND 1 | 22 | 50 | 0 |
| TROJAN | 26 | 59 | 0 |
| TURKEY POINT 3 | 23 | 74 | 1 |
| TURKEY POINT 4 | 23 | 74 | 3 |
| VERMONT YANKEE 1 | 25 | 71 | 1 |
| YANKEE-ROWE 1 | 19 | 68 | 0 |
| ZION 1 | 24 | 70 | 0 |
| ZION 2 | 24 | 70 | 0 |
| TOTAL | 1,261 | 4,910 | 27 |

SECTION 4

MULTI-PLANT ACTIONS DESCRIPTION AND STATUS

Multi Plant Action No. A-01 & A-74

No. of Plants: 72

Title: Inservice Inspection and Testing (ISI/IST)

Priority: High

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: S. Varga

Lead Engineer: D. Chaney

Lead Technical Review Branch: MTEB/MEB

Other Review Branches: _____

Problem:

Licenses for boiling or pressurized water-cooled nuclear power facilities are required by 10 CFR 50.55a(g) to have an inservice inspection and testing program that meets the requirements of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda (The Code). If the licensee determines that it is impractical to meet certain requirements of the Code, justification must be provided to the Commission, which can grant relief from specific Code requirements or impose alternate requirements.

Background:

Staff review is currently several years behind originally planned schedules. The staff has previously given interim implementation guidance to the licensees by letter. In general, this guidance authorized the licensees to implement their ISI/IST plans as proposed pending completion of review by the staff. In some cases the period for which the proposed plans and interim implementation authorization would apply has expired and the licensees have made subsequent submittals before the staff completed review of the original plan.

Current Status:

A systematic plan has been developed to conduct the staff review of ISI/IST programs. The plan prioritizes reviews, assigns resources, and establishes a schedule for the continuing review as ISI/IST programs are updated.

Action(s) Required to Complete the Licensing Action:

A contractor (EG&G, Pacific NW, Brookhaven) will provide a TER which evaluates the licensees requests for relief from the Code requirements. The evaluation will be based on the practicality of the requirement in light of the justification provided, the safety concerns of granting relief from a requirement, any need for alternate requirements to ensure safety, and the acceptability of alternate testing proposed. The staff will prepare an SER based on this TER and grant relief from code requirements or impose alternate requirements as appropriate.

Multi-Plant Action No. 11 A-02

No. of Plants: 72

Title: Appendix A - AI/PA

Priority: 1

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: J. F. Stolz

Lead Engineer: P. C. Wagner

Lead Technical Review Branch: Effluent Treatment Systems Branch

Other Review Branches: Radiological Assessment Branch

Problem:

Implementation of Technical Specifications called for in 50.36a and Appendix I to 10 CFR 50.

Background:

Appendix I was published in 1975 and numerous TS were sent to all licensees on 7/78, 11/78 and 7/79. Submittals have been received from all licensees except Kewaunee.

Current Status:

No progress toward resolution has been made to date.

Action(s) Required to Complete the Licensing Action:

1. Establish acceptance criteria - ETSB/RAB
2. Select a contractor and let a contract.
3. Contractor provide NRR with updated SERs and negotiated TS.
4. DL issue license amendment.

No. of Plants: 0

Title: Security Reviews Modified Amendment Plans

Priority: Complete

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: J. Miller

Lead Engineer: J. Miller

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. A-04

No. of Plants: 47

Title: Containment Leak Testing - Appendix J

Priority: High

Cognizant Assistant Director: G. Laines

Cognizant Branch Chief: D. M. Crutchfield

Lead Engineer: W. Paulson

Lead Technical Review Branch: CSB

Other Review Branches: Contractor: Franklin Research Institute

Problem:

Some operating plants may not be in full compliance with Appendix J to 10 CFR Part 50.

Background:

Appendix J to 10 CFR Part 50 was published after many nuclear power plants had either received their operating license or their containments had reached an advanced stage of design or construction. Accordingly, some plants may not be in full compliance with Appendix J. Licensee submittals, including exemption requests and proposed Technical Specifications need to be reviewed.

Current Status:

Eight SER's have been forwarded to the licensee; two more are being processed by the PMs. CSB has 24 draft TER's and 6 final TER's from Franklin that need review. Franklin is working on 7.

Action(s) Required to Complete the Licensing Action:

CSB must review the draft TER's from Franklin. Almost all will need recycling to Franklin based on experience to date. The final TER's and a SER prepared by CSB are then forwarded to DL for action. CSB's schedule dated April 10, 1981 shows about 8 SER's being completed per month.

Multi-Plant Action No. MPD-01(USI) - Generic Activity A-05

No. of Plants: — 25 —

Title: Mark I Containment Program

Priority: _____

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: B. Siegel

Lead Technical Review Branch: None Work to be performed under Technical Assistance Contract

Other Review Branches: ORAB & Chris Grimes

Problem:

Since the original design of the Mark I containment system additional loading conditions resulting from the dynamic effects of drywell air and steam being rapidly furlled into the suppression pool during a postulated LOCA and from suppression pool response to SRV operation generally associated plant transient operating conditions have been identified, the objective of this program is to restore the originally intended design safety margins for each Mark I containment system.

Background:

A Mark I Owners Group (OG) with GE the lead technical organization was formed to quantify the hydrodynamic loads and assess the effects of these loads on the Mark I containment structure. The OG divided this task into a short term (STP) and long term program (LTP).

The objectives of the STP were to verify that each Mark I containment system would maintain its integrity and functional capability when subjected to the most probable loads induced by a postulated design-basis LOCA, and to verify that licensed Mark I BWR facilities would continue to operate safely, without endangering the health and safety of the public, while a methodical, comprehensive LTP was being conducted. The staff in the "Mark I Containment Short-Term Program Safety Evaluation Report," NUREG-0408 concluded that sufficient margin of safety had been demonstrated to assure the functional performance of the containment system and, therefore, any undue risk to the health and safety of the public was precluded.

The objectives of the LTP were to establish design-basis loads that are appropriate for the anticipated life of each Mark I BWR facility (40 years), and to restore the originally intended design-safety margins for each Mark I containment system. The principle thrust of this program has been the development of generic methods for the definition of suppression pool hydrodynamic loading events and the associated structural assessment techniques for the Mark I configuration. The

generic analysis techniques are intended to be used to perform a plant-unique analysis (PUA) for each Mark I facility. The staff in the "Mark I Containment Long-Term Program Safety Evaluation Report," NUREG-0561 concluded that the load definition procedures utilized by the BWR Owner Group, as modified by the staff's requirements, provide conservative estimates of these loading conditions and that the structural acceptance criteria are consistent with the requirements of the applicable codes and standards.

Current Status:

The licensees are in the process of performing PUA and designing and installing modifications to meet the Commission's Order date for each operating plant. However, based on the May 22, 1981 meeting with the BWR Mark I OG it appears many licensees will not meet the Commission's Order dates because the results of the final TORUS analysis is needed for inputs to the TORUS attached piping analysis. The OG is preparing a submittal which identifies the status of the modifications and establishes revised completion dates for this program.

Action(s) Required to Complete the Licensing Action:

- 1) Staff review and evaluation of OG status report
- 2) Preparation of a Commission paper justifying extension of Order dates
- 3) Submittal of the PUA to the NRC
- 4) Completion of major and minor and TORUS attached piping modifications
- 5) Post implementation audit of PUA by Franklin Institute and BNL on Technical Assistance Contracts
- 6) Issuance of SER to each licensee on acceptability of Mark I LTP modifications and analyses for their plant.

Multi-Plant Action No. A-06

No. of Plants: 1

Title: Respiratory Protection System

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: J. Shea

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

License Action No. 4407

Number of Plants: All

Title: Fracture Toughness Appendix G.

Priority: None

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Crutchfield

Lead Engineer: R. Caruso

Lead Technical Review Branch: MTEB

Other Review Branches: _____

Problem:

Establish that heatup and cooldown limits are in compliance with requirements of Appendix G. Review T.S. and revise as necessary.

Background:

See Appendix G

Current Status:

Complete

Action(s) Required to Complete the Licensing Action:

None

Multi Plant Action No. A-08
No. of Plants: 4 remaining
Title. ECCS Evaluation against 10 CFR 50.46
Priority: Medium
Cognizant Assistant Director: T. Novak
Cognizant Branch Chief: John Stolz
Lead Engineer: Dan Garner
Lead Technical Review Branch: ORAB
Other Review Branches: None

Problem:

Evaluation of compliance with original implementation of 10 CFR 50.46.

Background:

50.46 was implemented in 1974. Most plants were evaluated in a few years. The effort involved complete review of designs of ECCS systems.

Current Status:

4 Plants remaining: Humboldt Bay, Indian Point 1, Prairie Island Units 1 and 2. The first two need not be completed. Prairie Island 1 and 2 must be finished, however, and are being reviewed under contract with Livermore through ORAB.

Action(s) Required to Complete the Licensing Action:

1. Completion of review under contract through ORAB
2. Issuance of Prairie Island 1 and 2 SERs

Multi-Plant Action No. A-59

No. of Plants: 68

Title: Pressure Vessel Beltline material Surveillance

Priority: Low

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: J. F. Stolz

Lead Engineer: P. B. Erickson

Lead Technical Review Branch: MEB

Other Review Branches: _____

Problem:

In May 1977 the NRC requested that all licensees provide data on reactor vessel materials used at their facilities. This provides a data base for our reviews of applications involving changes in heatup and cooldown Tech Specs or other reviews involving fluence.

Background:

Data has been provided by licensees either in response to the May 1977 request or in support of requests for changes in heatup and cooldown Tech Specs.

Current Status:

Complete

Action(s) Required to Complete the Licensing Action:

None

Application Action No. A-10
No. of Plants: 9
Title: Contingency Planning
Priority: High
Cognizant Assistant Director: T. Novak
Cognizant Branch Chief: J. Miller
Lead Engineer: J. Miller
Lead Technical Review Branch: _____
Other Review Branches: _____

Problem:

Development of a plan to specify for licensee personnel specific objectives they must carry out in threats, thefts or radiological sabotage.

Background:

Current Status:

Reviews are being conducted of proposed plans for all operating facilities. Approximately 45 contingency plans have been approved.

Action(s) Required to Complete the Licensing Action:

Issuance of license amendments

No. of Plants: 33

Title: Guard Training Plans

Priority: High

Cognizant Assistant Director: T. Kovak

Cognizant Branch Chief: J. Miller

Lead Engineer: J. Miller

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Security personnel must meet minimum criteria to assure that they will effectively perform their assigned security related job duties.

Background:

Current Status:

Reviews are being conducted of proposed plans for all operating facilities
Approximately 35 guard training plans

Action(s) Required to Complete the Licensing Action:

Issuance of license amendments.

No. of Plants: 55

Title: Vital Area Analysis

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: J. Miller

Lead Engineer: J. Miller

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Identifier areas in Nuclear Power Plants that a saboteur must enter to cause a 10 CFR Part 100 release.

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Total Manpower effort - 14 man weeks/facility includes 1 week site visit.

Report No. 100-113
No. of Pages: 2
Title: Non Power Reactor Safeguards 10 CFR 73.60
Priority: Medium
Cognizant Assistant Director: T. M. Novak
Cognizant Branch Chief: J. Miller
Lead Engineer: J. Miller
Lead Technical Review Branch: _____
Other Review Branches: _____

Problem:

Security plan must be developed to protect special nuclear material from theft or diversion.

Background:

Current Status:

Reviews must be conducted on 66 nonpower reactors. Of these 12 security plans have been approved.

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. A-14

No. of Plants:- 72

Title: 10 CFR 50.55A(6) Inservice Testing

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: S. Varga

Lead Engineer: D. Chaney

Lead Technical Review Branch: MTEB/MEB

Other Review Branches: _____

Problem:

See MPA A-01

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. A-15

No. of Plants: All operating Plants except Oconee 1, 2, 3.

Title: Quality Assurance Request Regarding Diesel Generator Fuel Oil.

Priority: Low

Cognizant Assistant Director: Thomas Novak

Cognizant Branch Chief: John Stolz

Lead Engineer: Guy Vissing

Lead Technical Review Branch: QAB

Other Review Branches: _____

Problem:

Consumable items where quality is necessary for functional performance of safety related components should also be classified as safety related and thus subject to applicable provisions of Appendix B to 10 CFR 50. DG fuel oil should therefore, be on the Q list or included in the QA program.

Background:

At ANO-1 IE found that DG fuel oil was not on their Q list or in their QA program. On investigation, we found this to be the case in other plants as well. Therefore, in January 1980, we requested all licensees to check their QA programs with respect to DG fuel oil and to include DG fuel oil in their QA programs or provide justification for not doing so.

Current Status:

All plants are acceptable except Kewaunee, Robinson 2, Davis-Besse 1.

Action(s) Required to Complete the Licensing Action:

DB-1 proposes to submit TS. Followup by PM required for Robinson and Kewaunee.

Review Number: B-01

No. of Plants: 63 Units

Title: Diesel Generator Lockout

Priority: _____

Cognizant Assistant Director: Gus Lainas

Cognizant Branch Chief: D. M. Crutchfield

Lead Engineer: T. V. Wambach

Lead Technical Review Branch: ORAB

Other Review Branches: None

Problem:

Some failures-to-start of diesel generators have been attributed to diesel generator lock-out involving critical timing of control circuit relay operation or reset following manual shutdown of the diesel. IE has transferred responsibility to Licensing to determine whether similar circuitry problems exist at other facilities and what corrective measures should be implemented.

Background:

By memo dated 01-26-77, DOT has requested that a survey be made of all operating plants to determine whether lockout or diesel generator inoperability is specifically annunciated in the control room for each diesel generator. A letter was sent to all operating reactors in March 1977. Review of responses is underway under TACS No. 1228.

Current Status:

Of the 63 units, 52 units have been completed. NRC technical position has been documented to DL by ORAB.

Action(s) Required to Complete the Licensing Action:

Forward position to licensees of 11 remaining units. Obtain licensee commitment to meet position and write final letter closing out.

Licensing Action No. B-02

No. of Plants: 74

Title: Fire Protection - App. A to BTP 9.5-1

Priority: Complete

Cognizant Assistant Director: G. Iainas

Cognizant Branch Chief: D. M. Crutchfield

Lead Engineer: T. V. Wambach

Lead Technical Review Branch: CEB

Other Review Branches: None

Problem:

Protection of safe, shutdown capability from the adverse effects of fire in any area of the plant. During the OL review of plants licensed prior to July 1976, no in-depth review was performed in this area.

Background:

Fire at Browns Ferry Nuclear Plant in March 1975 disabled redundant safety divisions and normal safe shutdown equipment. App. A to BTP 9.5-1 was issued in September 1976.

Current Status:

The review of all plants against the criteria of App. A to BTP APCS 9.5-1 has been completed and all SERs have been issued.

Action(s) Required to Complete the Licensing Action:

None

Multi-Plant Action No. B-03

No. of Plants: 4

Title: PWR Moderator Dilution

Priority: Low

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: R. A. Clark

Lead Engineer: C. Nelson

Lead Technical Review Branch: RSB

Other Review Branches: _____

Problem:

Review potential for moderator dilution and verify that FSAR moderator dilution event is most limiting.

Background:

NaOH was inadvertently injected into the RCS of a B&W PWR via the DHR system during surveillance cycling of the NaOH tanks isolation valves.

Current Status:

Receive responses from licensees

Issue letters after review of licensees' responses

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. B-04

No. of Plants: 16 PWRs remaining

Title: Reactor Vessel Overpressurization Protection

Priority: Medium

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: J. Stolz

Lead Engineer: D. Garner

Lead Technical Review Branch: ORAB/RSB

Other Review Branches: _____

Problem:

Assure overpressure protection of pressure vessels when at low temperatures.
Peculiar to PWRs only.

Background: Many events prior to 1976 indicated that administrative controls were not sufficient to prevent low temperature overpressurization events at PWRs. All plants were requested to provide an overpressure prevention system that would be used whenever the plant was in a cold shutdown condition. All PWRs implemented their systems with preliminary approval from the NRC, and a complete review took place on a postimplementation basis. The review consists of three parts:
a) Electrical aspects; b) Reactor systems aspects; c) Technical specifications.

Current Status:

Plants remaining to be evaluated are being reviewed by contract with EG&G (through RSB for reactor systems aspects and through ORAB for electrical aspects). This includes 9 of the 16 remaining plants. For the other seven plants, the SER's have been completed with only PM processing remaining.

Action(s) Required to Complete the Licensing Action:

1. Completion of the 9 SERs by ORAB and RSB
2. Review of TSs by PM and lead PM
3. Issuance of SERs and TSs for 16 plants

Multi-Plant Action No. B-05 (USI A-42)

No. of Plants: 49

Title: Stress Corrosion Cracking - BWR, DCSDR

Priority: High

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: D. Clark

Lead Technical Review Branch: ORAB, MEB

Other Review Branches: _____

Problem:

Leaks and cracks in the heat-affected zones (HAZs) of welds in BWR austenitic stainless steel piping and components have been observed since the mid-1960s. All the cracks have been attributed to intergranular stress corrosion cracking (IGSCC).

Background:

In 1975, NRC (AEC) formed a Pipe Cracking Study Group (PCSG) to investigate the cause of cracking. In July 1977, the staff issued an implementation document, NUREG-0313, based on recommendations of the PCSG. As a result of cracking in large diameter pipes and in nozzles which had not been observed prior to 1975, a new PCSG was activated in September 1978. In October 1979, Revision 1 to NUREG-0313 was issued for comment; this contained the latest recommendations of the Study Group. Based on industry comments, the document was modified and issued in July 1980.

Current Status:

In February 1981, NUREG-0313, Rev. 1 was issued to all holders of BWR operating licenses or construction permits and to all applicants for operating licenses. By July 1, 1981, the applicants/licensees are to provide their program for replacement of service sensitive lines and welds, their program for augmented inservice inspection, their program for improving the water chemistry environment and incorporation of adequate leak detection capability.

Action(s) Required to Complete the Licensing Action:

Review of individual applicant and licensee responses to determine if they meet the guidelines published in NUREG-0313, Rev. 1.

In accordance with generic letter 81-04 issued February 26, 1981, 26 utilities are to provide by July 1, 1981 their plans and programs for implementing the guidelines and requirements in NUREG-0313, Revision 1.

Multi-Plant Action No. B-06

No. of Plants: All BWRs

Title: BWR Relief Valves

Priority: NA

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: J. Hannon

Lead Technical Review Branch: ASB

Other Review Branches: None

Problem:

S/RV Closure Reliability

Background:

NUREG-0462 dated July 1978

Current Status:

Complete - July 12, 1979 (AD to Branch Chief)

Action(s) Required to Complete the Licensing Action:

None

No. of Plants: 28

Title: Steam Generator Feedwater Flow Instability

Priority: Completed

Cognizant Assistant Director: Gus Lainas

Cognizant Branch Chief: D. Crutchfield

Lead Engineer: S. Nowicki

Lead Technical Review Branch: ASB

Other Review Branches: _____

Problem:

Feedwater system piping supports and valves have failed or been damaged by fluid flow instability (water hammer). Since the steam generators are used for plant cooldown, the instability phenomenon with potential equipment damage is considered a safety related issue.

Background:

The damaging potential of waterhammer caused by water slug impact in the steam generator feedwater systems of PWRs was first demonstrated conclusively by the study following the major incident at Indian Point #2 on November 13, 1973. Damage included a 180° circumferential fracture of the 18 inch diameter main feedwater main feedwater pipe to the #22 steam generator at the point where the pipe penetrated the reactor containment structure, gross thermal deformation of the metal containment liner near this juncture due to water sprayed from the ruptured pipe, and a large bulge in the main feedwater pipe in the horizontal run of pipe to the steam generator nozzle. Water level could not be reestablished in #22 steam generator and it was isolated from the system during reactor cooldown. Over three hours passed between the initiating event and complete isolation of #22 steam generator.

Since the incident at Indian Point #2, and up to December 1, 1976, there have been at least 16 reported events believed to involve water slug impact in the steam generator feedwater systems of U.S. PWRs. At least five similar waterhammer incidents were also identified prior to that at Indian Point #2 (the earliest recorded event was at Yankee Rowe in 1966), according to the available evidence. Several waterhammer events have also occurred at various PWR plants during system tests intended to demonstrate the absence of waterhammer.

These waterhammer incidents are triggered by unusual operating transients such as unexpected reactor or feedwater pump trips, which occur infrequently and do not necessarily lead to waterhammer. Thus, there is a considerable element of statistical randomness about these occurrences which is itself a major cause of present uncertainties.

Current Status:

Complete. Evaluated all W and CE operating plants. All plants that have had waterhammers have been modified.

Action(s) Required to Complete the Licensing Action:

None

Multi-Plant Action No. B-08

No. of Plants: _____

Title: DWR-HPSI-LSPI Flow Resistance

Priority: Complete

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: J. Stolz

Lead Engineer: Phil Wagner

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. B-09

No. of Plants: 14

Title: Charging line excessive vibration problem

Priority: Complete

Cognizant Assistant Director: Tom Novak

Cognizant Branch Chief: John Stolz

Lead Engineer: Dom DiIanni

Lead Technical Review Branch: ORAB

Other Review Branches: None

Problem:

Vibratory loads caused by high pressure charging pumps have resulted in fatigue failures in CVC pipe system.

Background:

Current Status:

Complete as of 8/14/30

Action(s) Required to Complete the Licensing Action:

None

Multi-Plant Action No. R-10

No. of Plants: B&W only (6)

Title: Burnable Poison Rod Failure

Priority: Medium

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: J. F. Stolz

Lead Engineer: P. B. Erickson

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Burnable poison rod failures occurred at CR-3 because of wear in holddown mechanism.

Background:

Correction was to replace BPRAs.

Current Status:

Completed with reloads for each B&W reactor.

Action(s) Required to Complete the Licensing Action:

B-11

No. of events: 17Title: Flooding of EquipmentPriority: LowCognizant Assistant Director: T. Novak

Cognizant Branch Chief: _____

Lead Engineer: D. Verrelli

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Flooding of safety related equipment could occur with failure of non-safety grade components, such as condensor bellows.

Background:

Current Status:

All plants have been reviewed to resolve this issue.

Action(s) Required to Complete the Licensing Action:

Technical review of contractor report and issuance of SER to licensee.-

Multi-Plant Action No R-13

No. of Plants: 1 (ANO-2)

Title: Fuel Rod Bow Penalty

Priority: Low

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: S. A. Varga

Lead Engineer: W. J. Ross

Lead Technical Review Branch: Core Performance

Other Review Branches: ORB #3

Problem:

Fuel Rod Bowing in PWRs

Background:

On 8/9/76 W provided data that showed previously developed methods for accounting for the effect of fuel rod bowing on DNBR may not contain adequate thermal margin when unheated rods are present. PWR licensees asked to review W data and submit new technical specifications if necessary.

Current Status:

Only ANO-2 has active TACS.

Action(s) Required to Complete the Licensing Action:

Guidance from D. E. Eisenhut (3/28/81) was to retain generic issue until ANO-2 completed its review approx. 7/81.

Licensing Action No. B-15No. of Plants: 1Title: CE Poison Rod GrowthPriority: LowCognizant Assistant Director: T. Novak

Cognizant Branch Chief: _____

Lead Engineer: M. Fletcher

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

In the early CE designed fuel assemblies, the poison rods were observed to show axial growth.

Background:

The cause was determined to be poison pellet swelling, in which the pellet was brought in contact with the tube causing growth as the pellet grew further in an axial direction.

Current Status:

Effectively closed on all CE facilities

Action(s) Required to Complete the Licensing Action:

CPB - approve topical report

Multi-Plant Action No. B-16

No. of Plants: 72

Title: Emergency Planning and Reviews

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief:

Lead Engineer: F. Pagano

Lead Technical Review Branch:

Other Review Branches:

Problem:

Upgrade emergency preparedness onsite and offsite in accordance with the new Emergency Planning Rule - 10 CFR 50, Appendix E

Background:

The Emergency Preparedness Program is being conducted through a review of licensee plans and procedures, conducting onsite appraisal visits, observing and evaluating joint onsite/offsite exercises, and reviewing the findings and determinations provided by FEMA on the adequacy of offsite preparedness.

Current Status:

Action(s) Required to Complete the Licensing Action:

Reviews need to be made on all operating plants

MB-17

No. of Plants: 75Title: TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUDDERSPriority: LowCognizant Assistant Director: Tom NovakCognizant Branch Chief: Robert Clark/ORB-3Lead Engineer: Leon B. EngleLead Technical Review Branch: ORABOther Review Branches: LGB/DST & MEB/DE

Problem: Operability of snubbers is required to provide assurance that the structural integrity of the reactor coolant system and all other related systems is maintained during and following a seismic or other event initiating dynamic loads. OPERABILITY is verified by an Inservice Inspection and Testing Surveillance program specified in plant Technical Specifications (TS). Recent operating experience has indicated the need for changes, clarifications and improvements in the Inservice Surveillance Requirements for hydraulic snubbers.

Background: By letter dated November 20, 1981, the NRC requested that all power reactor licensee's (except SEP) incorporate the model NRC TS in plant specific Appendix A TS. The NRC TS require that the visual inspection frequency be based upon maintaining a constant level of snubber protection to systems. In order to provide assurance of hydraulic snubber functional reliability, a representative sampling of plant installed snubbers must be functionally tested during plant shutdowns at 18 month intervals. The required sampling provides a confidence level of 95% that 90% to 100% of plant specific snubbers will be OPERABLE within acceptance limits.

The NRC letter requested that licensees submit license amendment applications to incorporate the applicable portion of the NRC model TS within 120 days (March 20, 1981). A similar NRC request was sent to SEP operating licensees on March 23, 1981.

Current Status: As of June 17, 1981, actions required to complete this task (issuance of license amendments) are as follows:

1. Licensee's have submitted responses for 36 units
2. Submitted responses under review total 36 units
3. Licensee's not responding to date total 16 units
4. SEP responses due August 1, 1981 total 11 units
5. Amendments completed total one (1) unit
6. Amendments estimated to be completed by June 30, 1981 total 7 units

it should be noted that NTOH's presently under review include the NRC model IS discussed above.

Action Required to Complete the Licensing Action:

The issuance of all licensing amendments required to complete Task MD-17 is estimated to be 8/30/81.

Multi Plant Action No. R-1R

No. of Plants: —0—

Title: Northington PWR Pump Shaft Integrity

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: C. Nelson

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

M-114 Plant Action No. MB-19No. of Plants: 5Title: Neutron Shielding - CE ReactorsPriority: LowCognizant Assistant Director: Tom NovakCognizant Branch Chief: Robert Clark/ORB-3Lead Engineer: Leon B. EngleLead Technical Review Branch: RAB/DSI

Other Review Branches: _____

Problem: In mid-1970's, excessive neutron dose rates were observed inside and around containment during start-up testing at new vintage CE plants. The high dose rates were caused by neutron streaming out of the annular gap between the reactor vessel and shield wall.

Background: All of the CE plants with this problem made NRC approved shielding changes.

Current Status: After the changes, additional neutron radiation surveys showed dose rates to be within acceptable limits.

Actions Required to Complete the Licensing Action:

None - Not Active at this time.

Multi-Plant Action No. B-20No. of Plants: 65Title: Containment Leakage Due to Seal DeteriorationPriority: HighCognizant Assistant Director: T. NovakCognizant Branch Chief: S. VargaLead Engineer: E. ReevesLead Technical Review Branch: CSB/DSIOther Review Branches: EQB/DE, ORAB/DL & AEB/DSIProblem:
See MPA B-24

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Classification: B-21

No. of Plants: 1

Title: Loss of 125V DC Bus Voltage with Loss of Annunciator System

Priority: Low

Cognizant Assistant Director: Gus Lainas

Cognizant Branch Chief: D. Crutchfield

Lead Engineer: S. Nowicki

Lead Technical Review Branch: PSB

Other Review Branches: _____

Problem:

Loss of a 125V DC bus, which supplies one division of a two division redundant DC power system, would generally result in a reactor trip and possible loss of the main control room annunciation system.

Background:

As a result of a review of an event at Zion Unit 2, which involved the loss of a safety related dc bus leading to a loss of control power to certain 4 kv ac buses, to all main control room annunciators, and to various other loads, it was concluded that sufficient operational status information on safety related dc systems may not be provided in the control room of all operating reactor facilities. As a corollary it was suggested that more attention should be given to the design and review of the annunciation systems of nuclear power stations.

The generic issue B-21 was proposed to require licensees of operating reactors to review their design and propose revisions, as necessary for:

- a) The plant annunciators and monitoring systems pertaining to the status of all dc buses in the plant is available to the control room operator at all times; and
- b) The plant bypass status indication systems, to monitor the position of station battery output breaker or fused disconnect switch (if provided) and the charger input and output breakers.

Current Status:

B-21 was never initiated; it was held in abeyance for the completion of Task Action A-30 "Adequacy of Safety Related DC Power Supplies"

Action(s) Required to Complete the Licensing Action:

Review of NUREG-0666 (published April 1981) does not cover recommendations to incorporate the concerns of B-21. When the final recommendations to Power Systems Branch are made B-21 should be covered by Task A-30, and B-21 should be cancelled.

MULTIPLIERS: _____ MB-22 _____

NO. of Plants: 75Title: TECH SPEC SURVEILLANCE FOR MECHANICAL SNUBBERSPriority: MediumCognizant Assistant Director: Tom NovakCognizant Branch Chief: Robert Clark/ORB-3Lead Engineer: Leon B. EngleLead Technical Review Branch: ORABOther Review Branches: LGB/DST & MEB/DE

Problem: OPERABILITY of snubbers is required to provide assurance that the structural integrity of the reactor coolant system and all other related systems is maintained during and following a seismic or other event initiating dynamic loads. Operability is verified by an Inservice Inspection and Testing Surveillance program specified in plant Technical Specifications (TS). Prior to initiation of MB-22, mechanical snubbers had not been included in operating plant surveillance programs. Recent operating experience has indicated that mechanical snubbers require OPERABILITY requirements similar to those required for hydraulic snubbers.

Background: By letter dated November 20, 1981, the NRC requested that all power reactor licensee's (except SEP) incorporate the model NRC TS for mechanical snubbers in plant specific Appendix A TS. The NRC TS require that the visual inspection frequency be based upon maintaining a constant level of snubber protection to systems. In order to provide assurance of mechanical snubber functional reliability, a representative sampling of plant installed snubbers must be functionally tested during plant shutdowns at 18 month intervals. The required sampling provides a confidence level of 95% that 90% to 100% of plant specific snubbers will be OPERABLE within acceptance limits.

The NRC letter requested that licensees submit license amendment applications to incorporate the applicable portion of the NRC model TS within 120 days (March 20, 1981). A similar NRC request was sent to SEP operating licensees on March 23, 1981.

Current Status: As of June 17, 1981, actions required to complete this task (issuance of license amendments) are as follows:

1. Licensee's have submitted responses for 36 units
2. Submitted responses under review total 36 units
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4. SEP responses due August 1, 1981 total 11 units
5. Amendments completed total one (1) unit
6. Amendments estimated to be completed by June 30, 1981 total 7 units

Plant-Field No. B-23

No. of Plant: BB

Title: B-23 - Potential Equipment Failures Associated with
Degraded Grid Voltage

Priority: Medium

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Crutchfield

Lead Engineer: Stan Nowicki

Lead Technical Review branch: PSB

Other Review Branches: _____

Problem:

Potential exists for supplying safety and non-safety equipment with voltage outside the range for which it was designed for longer than transient periods. Such conditions could result in safety-related equipment being unavailable to mitigate the consequences of an accident.

Background:

On July 20, 1976, Northeast Nuclear Energy Company (NNECO) reported that, following a trip of Millstone Unit No. 2 on July 5, 1976, several motors powered from 480 volt (v) motor control centers failed to start as required. The failure of the 480 v motors to start was traced to blown control power fuses on the individual motor controllers. These controllers receive control power through 480 v/120 v transformers within the controller.

NNECO's investigation disclosed that, as a result of the plant trip, the grid voltage dropped from 352 kv to 333 kv. This voltage drop, in conjunction with additional voltage drops associated with the transformers involved, reduced the control power and voltage within individual 480 v controllers to a voltage which was insufficient to actuate the main line controller contractors. As a result, when the motors were signalled to start, the control power fuses were blown. Subsequent testing by NNECO showed that the contractors required at least 410 v to function properly.

NNECO concluded that under similar low voltage conditions, the operability of 480 v Engineered Safety Feature equipment could not be assured.

NNECO's immediate corrective action was to raise the setpoint of the Engineered Safeguards Actuation System (ESAS) "loss of power" under-voltage relays to assure that the plant would be separated from the grid and emergency power system (dual) operation would be initiated before the control voltage fell below that required for contractor operation. A trip of the undervoltage relays causes the emergency buses to be de-energized and a load shed signal to strip the emergency buses, the diesel generators to start and power the emergency buses, and required safety related loads to sequence start on the buses.

On July 21, 1976, NNECO reported that the earlier corrective action taken was no longer considered appropriate because during starting of a circulating water pump, the voltage dropped below the new ESAS undervoltage relay setting. This de-energized the emergency buses, caused load shedding to occur, started diesel generators and began sequencing loads onto the emergency buses in accordance with the design. However, during sequencing of the loads onto the buses, the voltage again dropped below the undervoltage relay setting which caused the load shed signal to strip the buses. The result was energized emergency buses with no loads supplied.

Current Status:

27 SER's completed by contractors
15 SERs issued to OR Branches

Action(s) Required to Complete the Licensing Action:

Complete and Issue the remaining 51 SERs

Project-Plant Action No. B-24*

*B-20 integrated into B-24 on 9/25/80

No. of Plants:- 65Title: Venting and Purging Containment While at Full Power and
Effect on LOCAPriority: HighCognizant Assistant Director: T. NovakCognizant Branch Chief: S. VargaLead Engineer: E. ReevesLead Technical Review Branch: CSB/DSIOther Review Branches: EQB/DE, ORAB/DL & AEB/DSI

Problem:

Purging and/or venting of reactor containment buildings with reactor coolant temperature above 200°F may result in LOCA site boundary doses exceeding 10 CFR 50 Part 100 limits.

Background:

Events at Millstone 1 and Salem 1 (where the automatic high radiation closure of purge valves was overridden by operators) resulted in a generic letter being sent to all licensees of operating reactors in November 1978. A report was also made to Congress (#78-05). NRC advised of the events, requested licensees take action to minimize purge and vent operations, and requested response to SRP 6.2.4 and CSB BTP 6-4. In October 1979 an Interim Position was developed by the staff and licensees were requested to conform to it pending completion of our long-term review. In August 1980 generic item B-20 Leakage of Resilient Seals for purge and vent valves was integrated into the B-24 issue. A staff position to require Technical Specifications (as a leakage surveillance at either 3-months or 6-months intervals depending on valve usage) has recently been developed.

Current Status:

1. All licensees have responded to the November 1978 request for information. Reviews of licensee submittals are still underway although TMI-2 and the April 1980 reorganization of NRR caused delays. All licensee's responses to the October 1979 Interim Position have been reviewed and found acceptable as an interim measure. IE is monitoring licensees for continued compliance with these interim commitments.
2. The lead Project Manager has issued guidance in memoranda dated August 25, 1980 and March 24, 1981 to all OL OR Branch Chiefs and Project Managers and to the Technical Review Team. The August 25, 1980 memorandum identified the new TACS, the reviewers and assignments to assure continuation of reviews during the NRR reorganization period after April 1980.

3. Contracts are managed by the Technical Review Team as follows:
 - a. Electrical Reviews are managed by ORAB/OL with contracts to EG&G, Inc. Idaho and LLNL San Ramon, and FRC, Philadelphia. SEP reviews are managed by SEP with contract to EG&G Idaho.
 - b. Valve operability reviews are managed by EQB/DE with contract to BNL.
 - c. CSB 6-4 Review requirements are managed by CSB/DSI with contract to LLNL.
 - d. Radiological Impact Reviews are managed by AEB/DSI with contract to Exxon Nuclear Idaho Company, Inc.
4. As of June 8, 1981, a sample letter prepared by ORB #1 was concurred in by CSB/DSI and is ready for issuance to each licensee with the next NRC action letter (either Q's or SER). The letter contains recent NRC position clarifications and sample Technical Specifications to close out the long-term reviews. The overall B-24 review will complete TMI Item II.E.4.2, except for II.E.4.2.5 Containment Pressure Setpoint.

Actions Required to Complete B-24 and B-20 Licensing Actions:

1. Project Managers issue update status of NRC reviews to each licensee using sample letter (4 above).
2. CSB, EQB, AEB and ORAB complete any remaining plant specific SER's and provide to Project Managers.
3. Project Managers review the licensee Technical Specification submittals and issue license amendments as required.
4. TMI Item II.E.4.2.5 reviews by CSB.

Multi-Plant Action No. B-25
No. of Plants: 23
Title: BWR Feedwater Nozzle Cracking
Priority: High
Cognizant Assistant Director: G. Lainas
Cognizant Branch Chief: D. Crutchfield
Lead Engineer: R. Snaider
Lead Technical Review Branch: DST
Other Review Branches: ASB

Problem:

Cracking in BWR Feedwater and Control Rod Drive Return Line Nozzles due to thermal fatigue.

Background:

The generic issue has been resolved by issuance of NUREG-0619. Responses from licensees to an implementation letter have been received and recommended treatment of these responses have been submitted to management.

Current Status:

Action(s) Required to Complete the Licensing Action:

Review licensee responses to staff's position in NUREG-0619
Issue Confirmatory Action Letter or Orders requiring modifications to facility.

Plant Action No. 78-05

No. of Plants: 40

Title: Inadvertant Safety Injection During Cooldown

Priority: Medium

Cognizant Assistant Director: Tom Novak

Cognizant Branch Chief: R. Clark

Lead Engineer: R. Martin

Lead Technical Review Branch: Mt. E. B.

Other Review Branches: -

Problem: Inadvertent actuation of ECCS during cooldown have occurred in a number of PWR facilities. If this occurs when RCS pressure is less than ECCS discharge pressure and RCS temperature is significantly above ECCS water temperature then the regions of the injection nozzles can be subjected to thermal stresses.

Background: An inadvertent injection at Salem-1 on January 23, 1978 prompted the issuance of IE Circular 78-05 on May 23, 1978. The circular advised licensees of specific actions that could be taken to minimize the frequency of inadvertent safety injections and concluded that no further licensing action or response to NRC was required at that time. NRR sent letters to 39 of the 40 licensees advising them that the staff had concluded that nozzles could most likely withstand 50 or more such transients but if their plants experienced more than 25 they should consider a plant specific analysis to identify the number of transients the plant could safely withstand.

Actions Required to Complete the Licensing Action:

Action is complete on 39 of the 40 plants. The issue will be assessed for the remaining plant with respect to conflict with the issues identified in the staff's letter to all licensees dated April 20, 1981 on THERMAL SHOCK TO RPV's. Then a letter to the Director DL will be prepared as the mechanism for closing the issue.

Multi-Plant Action No. B-2/ _____

No. of Plants: - 0

Title: Review responses to IE Bulletin 78-03 (offgas explosions)

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: J. Shea

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. B-29

No. of Plants: 0

Title: BWR Feedwater Pump Trip

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: P. O'Connor

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. R-30

No. of Plants: 9

Title: Steam Generator Replacement Program

Priority: Low

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: R. A. Clark

Lead Engineer: R. Martin

Lead Technical Review Branch: MtEB, CMEB

Other Review Branches: _____

Problem:

"In 1975, 'denting' of steam generator tubes, a result of corrosion product build up in the tube to tube support plate crevices, was detected during routine inservice inspections in Westinghouse steam generators. Plugging of degraded steam generator tubes due to denting can ultimately lead to a decrease in electric power generation capability. Some Westinghouse units which have plugged large numbers of tubes are planning to replace or repair steam generators. Review of the replacement and repair programs is required."

Background:

The above statement comprises the total documentation known to the lead engineer for this matter in its context as a multi-plant action. There are other problems, in addition to "denting" that affect SGs and like denting problems they are handled on a plant specific basis.

Current Status:

The need for continuing this issue as a multi-plant action is questionable. Historically the problems with SGs have been plant specific and have been addressed on a case-by-case basis. Numerous interviews with NRR personnel have not identified a useful purpose that a multi-plant action on this issue could serve.

Action(s) Required to Complete the Licensing Action:

Lead Engineer to write a memo to Director, DL which closes out this issue.

Multi-Plant Action No. B-31

No. of Plants: 3

Title: Long Shaft LHSI and Outside Recirc Pump Degradation

Priority: Low

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: S. Varga

Lead Engineer: D. Neighbors

Lead Technical Review Branch: MTEB

Other Review Branches: _____

Problem:

Upgrade pumps at plants which have long shaft pumps. There were five plants in this category.

Background:

Excessive degradation occurred to bearings and shaft after short periods of operation of these pumps. Problems originally occurred at North Anna.

Current Status:

North Anna 1&2 problems are resolved. Beaver Valley and Surry 1 and 2 have made necessary modifications and testing to assure integrity of these pumps.

Action(s) Required to Complete the Licensing Action:

All work is complete except for a final write-off. This was to be done by Zudans (EB & EQB) but he left the agency. Not picked up again.

Multi-Plant Action No. B-32

No. of Plants: 5

Title: Blocked SI Signal During Cooldown

Priority: Medium

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: S. A. Varga

Lead Engineer: M. Grotenhuis

Lead Technical Review Branch: ORAB - EG&G

Other Review Branches: None

Problem:

NRR is requested to determine the acceptability of blocking the steam line differential pressure safety injection signal during cooldown simultaneously with the blocking of the coincident pressurizer low level and low pressure signal.

Background:

This arose during an inspection at H. B. Robinson No. 2 (Surry and Turkey Point were included because the plants were similar)

Current Status:

H. B. Robinson - complete
Surry 1/2
Turkey Point 3/4

Action(s) Required to Complete the Licensing Action:

Contractor report being prepared. Depending on results maybe letter to utilities. Maybe Tech Spec change required.

Multi-Plant Action No. B-33

No. of Plants: — 0 —

Title: Iodine Spiking

Priority: Complete, Deleted

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: S Sands

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. B-34

No. of Plants: 19 BWR units

Title: Jet Pump Integrity Assurance

Priority: Medium

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: R. Bevan

Lead Technical Review Branch: MtEB

Other Review Branches: _____

Problem:

Since the failure of a jet pump holddown beam bar at Dresden 3 in February 1980, there has been renewed interest in jet pump integrity assurance. Jet pump failure and displacement can lead to difficulty in maintaining core covering in a LOCA.

Background:

Occasional failure of retainer bolt keepers and related jet pump components since 1972 has sustained some level of interest in jet pump integrity. The more recent jet pump failure at Dresden and the subsequent discovery of beam bar cracking or crack indications (UT examination) at other plants has increased staff interest in this area.

Current Status:

Operating plants are monitoring jet pump action per IEB 80-06 and GE SIL 330. This action will continue until new improved beam bars are made available in sufficient quantity to change out all the older ones now shown to be subject to failure. GE is currently developing replacement pieces and IE is reviewing this activity and verifying acceptability as per TIA 80-16.

Action(s) Required to Complete the Licensing Action:

- 1) Review by MEB any proposed jet pump design changes
- 2) Develop plan/schedule for changeout of all beam bars, consistent with GE making such (approved) pieces available (these are under development at GE)
- 3) Evaluate need for jet pump ISI
- 4) Solicit appropriate TS changes from licensees for long term surveillance of jet pump integrity
- 5) Review licensee responses and revise TS as appropriate.

Multi-Plant Action No. B-35

No. of Plants: 8

Title: Orifice Rod Assembly (ORA) Integrity - B&W

Priority: Issue Complete

Cognizant Assistant Director: Thomas Novak

Cognizant Branch Chief: John Stolz

Lead Engineer: Guy Vissing

Lead Technical Review Branch: CPB

Other Review Branches: _____

Problem:

The latch mechanism for ORA in B&W reactors have indications of wear. If these latch mechanisms would fail, the ORA could be ejected from the core and cause damage to the OTSG.

Background:

In April 1978, CR-3 had an event which involved burnable poison rod assemblies (BPRA) being ejected from the core while at full power. The latch mechanism failed. Because the latch mechanism of the ORA is the same design as the latch mechanism of the BPRA, we requested all plants to look at the ORA latch mechanisms which showed wear. Therefore, DB-1 elected to remove the ORA. This was found acceptable. Subsequently, each licensee of B&W plants removed the ORA during a subsequent refueling outage.

Current Status:

Issue is complete for all affected plants.

Action(s) Required to Complete the Licensing Action:

None

MB-36

No. of plants: 8

Title: RTD Response Time for CE Facilities

Priority: Low

Cognizant Assistant Director: Tom Novak

Cognizant Branch Chief: Bob Clark

Lead Engineer: Monte Conner

Lead Technical Review Branch: Instrumentation & Control System Branch

Other Review Branches:

Problem: To accurately determine the response time for RTDs used in the RPS.

Background: With the adaptation of the STS definition of RPS response time requirements for primary sensors, the RTD portion of the response time became a difficult value to obtain. If the RTD is removed from its well in the RCS piping, how do you account for the well effect in finding the response time. New methodologies are now available to measure RTD response times directly during reactor operations.

Current Status: Draft NUREG, "Review of Resistance Temperature Detector Time Response Characteristics" was issued November 1980. It has been held up in Safety Program Evaluations Branch since then.

Actions Required to Complete the Licensing:

Get the NUREG issued and send to all licensees with an instruction letter.

License Action No. 10-37
No. of Plants: 15
Title: Steam Generator Tube Denting and Support Plate Modifications - LE
Priority: Low
Cognizant Assistant Director: Tom Novak
Cognizant Branch Chief: Robert A. Clark
Lead Engineer: Glode Requa
Lead Technical Review Branch: Materials Engineering Branch
Other Review Branches: EEB, RSB, MEB

Problem: In Mid 1975, "denting" of steam generator tubes, a result of corrosion product build-up in the tube to tube support plate crevices, was detected during routine inservice inspections in Westinghouse and CE plants.

Background: CE units have implemented modifications to steam generator tube support to plates to mitigate the effects of tube "denting" and initiated a steam generator tube inspection program. NUREG-0523 identifies operational problems, NRC staffs evaluation of problems and summarizes staff programs for resolution of each problem.

Current Status: Ongoing

Actions Required to Complete the Licensing Action:

1. Lead Engineer review program to determine if ongoing inspection programs and other licensee resolutions should now be plant specific.
2. Upon completion of 1., recommend actions to either complete MB-37 or change to plant specific.

Multi-Plant Action No. B-38

No. of Plants: at least 4

Title: Tendon Surveillance - Bechtel Containments

Priority: Low

Cognizant Assistant Director: Thomas Novak

Cognizant Branch Chief: John Stolz

Lead Engineer: Guy Vissing

Lead Technical Review Branch: SEB

Other Review Branches: _____

Problem:

The problem involves the surveillance of post tensioned prestressed concrete containment tendons. The problem relates to the method used to interpret the results at the sampling and testing of tendons.

Background: This task was first identified as a multi-plant issue in October 1977 when DOR put all outstanding issues into the RAMS system. It initially applied to ANO-1 and it involved some staff concerns on interpretations of the third year surveillance report for ANO-1 containment tendons. We found similar concerns were identified at other plants with post tensioned prestressed concrete containments. Therefore, we made it a multi-plant issue.

SEE ATTACHMENT TO THIS PAGE

Current Status:

The recommendations for closing out this issue were accepted by the Division Director; however, we were requested by the Division Director to prepare a letter to the licensees expressing our intentions and to be prepared for our next step.

Action(s) Required to Complete the Licensing Action:

Prepare letter to all licensees (this way we will be able to cover all who have post tensioned prestressed concrete containments).

To determine the scope of the issue we reviewed the Technical Specifications (TSs) of a sample of plants with prestressed concrete containments. We found that there were inconsistencies in the TSs for ungrouted tendons for the plants reviewed. Therefore, it was concluded that Tendon Surveillance TSs should be upgraded for many facilities with prestressed concrete containments.

Our plan was to request licensees of plants with post tensioned prestressed concrete containments to implement Standard Technical Specifications (STCs) for ungrouted tendons. However, the current STS for tendons are not considered by the staff to be the most acceptable. Rather, the staff believes that the methods provided by the proposed Revision 3 of Reg Guide 1.35 and Reg Guide 1.35.1 would be more appropriate for implementing into TSs. Revision 3 of Reg Guide 1.35 and Reg Guide 1.35.1 are presently issued for public comment. Meanwhile, the ASME has drafted surveillance guidelines for tendons in Section XI of the ASME Code. These too are to be issued for comment.

We found, in discussing the schedule for issuance of Reg Guides 1.35 and 1.35.1 with the Office of Standards Development, that Reg Guides 1.35 and 1.35.1 will not be issued until after the revisions of Section XI of the ASME Code have been completed and endorsed by Reg Guides 1.35 and 1.35.1.

This process will take 12-18 months at least. Therefore, we have not initiated any action with licensees with respect to this task. For this reason we recommended closing out this task.

Multi-Plant Action No. B-41

No. of Plants: 74

Title: Fire Protection - Final Tech Specs - Final SER Supplements

Priority: High

Cognizant Assistant Director: G. Lafnas

Cognizant Branch Chief: D. M. Crutchfield

Lead Engineer: T. V. Wambach

Lead Technical Review Branch: CEG

Other Review Branches: ASB, PSB, ICSB, RSB, PTRB, LQB

Problem:

Protection of safe, shutdown capability from the adverse effects of fire in any area of the plant. During the OL review of plants licensed prior to July 1976, no in-depth review was performed in this area.

Background:

Fire at Browns Ferry Nuclear Plant in March 1975 disabled redundant safety divisions and normal safe shutdown equipment. NUREG-0050

Current Status: 10 CFR 50.48 and App. R to 10 CFR 50 became effective February 17, 1981. The licensees for 71 units licensed prior to January 1, 1979 and for 3 units licensed after January 1, 1979 have made submittals describing (1) plans and schedules to meet App. R, (2) design descriptions of alternative, safe shutdown modifications, and (3) exemption requests from App. R.

Action(s) Required to Complete the Licensing Action:

The submittals described above for all 74 units must be reviewed and acted upon by June 1982. This will entail issuing exemptions, exemption denials and SERs approving alternative or dedicated safe shutdown modifications. Review of new OL's is now incorporated in the SRP. Final Tech Specs covering the equipment added as a result of App. R must also be issued after the modifications are complete.

No. of _____

Title: TMI Followup Items - All Plants

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: Fairfile

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Title: PWR Feedwater Line Cracks - Long Term Corrective Action

Priority: Medium

Cognizant Assistant Director: Tom Novak

Cognizant Branch Chief: Steve Varga

Lead Engineer: Sydney Miner

Lead Technical Review Branch: Materials Engineering Branch

Other Technical Review Branches: Safety Program Evaluation Branch

Problem: Leaking Circumferential cracks were detected in Feedwater lines of both Westinghouse and Combustion Engineering facilities in the immediate vicinity of the steam generator nozzles.

Background: The first incident of cracking occurred in May of 1979 in the feedwater lines of D. C. Cook Unit 2. This event prompted issuance of IE Bulletin 79-13 requiring inspection of all PWR feedwater lines.

Results of these inspections (17 incidents of cracking at 35 plants examined) led to the formulation of the PWR Pipe Crack Study Group (PCSG). Immediate short-term corrective actions taken by IE were directed towards restoration of the piping to its original condition.

In a memorandum dated Sept. 19, 1980, Ed Jordan of IE described the short-term actions as complete. Also, in referring to the PCSG final recommendations, he suggested that augmented ISI requirements extending beyond the next refueling be imposed by Technical Specifications tailored for each facility. He further suggested that NRR should develop positions on permanent measures necessary to prevent recurrence.

Subsequently, the Materials Engineering Branch (MTEB), Division of Engineering, NRR agreed to provide a memorandum on the above. However, prior to transmitting new licensing requirements to DL, MTEB coordinated the implementation of PCSG recommendations with the Division of Safety Technology (DST) through a value/impact study. In addition, the Division of Engineering agreed with a DST recommendation that a risk agreement would complement the deterministic analysis performed by the PCSG and aid in long term implementation.

Current Status: In accordance with a Feb. 22, 1981 request by the Division of Engineering, the Safety Program Evaluation Branch (SPEB) of DST performed a cost/benefit and risk assessment study. A draft of this study is currently being routed through DST management for review and approval.

Action(s) Needed to Complete the Licensing Action:

1. Approval by DST and issuance of their recommendations for implementation.
2. Issuance of a memorandum by DE to DL recommending their proposed long term actions.
3. Preparation of a set of instructions to Project Managers on long term actions required (this could be a generic letter depending on whether or not there are any plant specific requirements).
4. Issuance of a letter to the licensee setting forth the new requirements.
5. Review of licensee's responses and revision of corresponding Technical Specifications as required.

No. of Plants: 0

Title: Lessons Learned Implementation

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: T. Telford

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

No. of Plants:- 0

Title: WASH 1400 Event V; "Primary Coolant Pres. Isolation Valves"

Priority: Complete

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: P. Polk

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. B-46

No. of Plants: 70

Title: Analysis of Turbine Disc Cracks

Priority: Medium

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: S. A. Varga

Lead Engineer: W. J. Ross

Lead Technical Review Branch: Material Engineering

Other Review Branches: _____

Problem:

Monitor U.T. inspections of W and GE low pressure turbine discs for keyway cracks.

Background:

Inspections by W since 10/79 have revealed novel stress corrosion cracks in keyway and bore regions of low pressure turbine discs.

Current Status:

Staff action nearly complete. Draft letter to Licensees closing out staff monitoring activities undergoing concurrent review.

Action(s) Required to Complete the Licensing Action:

Concur on letter to licensees.

Multi Plant Action No B-47

No. of Plants: _____

Title: ECCS Clad Swelling and Rupture

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: L. Olshan

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. B-48

No. of Plants:- 72

Title: Adequacy of Station Electric Distribution Voltage

Priority: Medium

Cognizant Assistant Director: Gus Laines

Cognizant Branch Chief: D. Crutchfield

Lead Engineer: Lombaruo

Lead Technical Review Branch: PSB

Other Review Branches: _____

Problem:

This review is intended to determine if the onsite distribution system in conjunction with the offsite power sources has sufficient capacity and capability to automatically start and operate all required safety loads within the equipment voltage rating.

Background:

At Arkansas Nuclear 1, loss of offsite power resulted in analysis that indicated that the electric distribution systems inadequate for two unit operation. Asked all utilities to review their distribution systems to see if voltages were adequate.

Current Status:

Contractor has reviewed and issued 32 TERs. Thirteen SER's have been completed and issued to the cognizant Project Managers.

Action(s) Required to Complete the Licensing Action:

Review licensee's responses to NRC questions. Prepare SER's and modify the Technical Specifications where necessary.

Multi-Plant Action No. MB-49

No. of Plants: 17

Title: PWR Control Rod Misalignment

Priority: 4

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: J. F. Stoiz

Lead Engineer: P. C. Wagner

Lead Technical Review Branch: ORPII - Lead Engineer

Other Review Branches: Core Performance Branch

Problem:

Older Westinghouse Plant's Technical Specifications did not provide requirements to ensure compliance with assumptions used in various analysis regarding control rod alignment.

Background:

Letters were sent to all Westinghouse PWRs on October 29 or November 5, 1979 requesting revisions to their TS (if needed) to ensure rods were aligned within 15 actual inches or 7.5 indicated inches.

Current Status:

1. License amendments issued on 10 plants
2. SERs accepting proposed TS written for 3 plants
3. Awaiting TS proposal on 3 plants (Haddam Neck and Surry 1 & 2)
4. Reviewing latest letter from 1 plant (Kewaunee)

Action(s) Required to Complete the Licensing Action:

1. Review proposed TS from 3 plants when received.
2. Negotiate an agreement with remaining plant.

No. of Plants: 72

Title: Auxiliary Feedwater Systems Evaluation

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: R. Clark

Lead Engineer: G. Requa

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

By memorandum to DL, Deletion of task B-50 was requested because of its redundancy to NUREG-0737 task II.E.1.1.

Current Status:

Approval was granted and Task B-50 was deleted from the listing.

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. B-52

No. of Plants: 42 (PWRs)

Title: Inadvertent Safety System Actuations During Surveillance
testing

Priority: Medium

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: R. A. Clark

Lead Engineer: C. Trammell

Lead Technical Review Branch: RSB (when needed)

Other Review Branches: None

Problem:

This generic item was initiated by an event which occurred at Zion during safeguards testing wherein an inadvertent reactor trip and ECCS actuation occurred and numerous other failures also resulted.

Background:

As a result of the above, licensees were requested to review the Zion event to see whether similar errors have or could occur, review management policies and procedures to assure that multiple equipment failures are vigorously pursued, and review surveillance procedures to assure that appropriate cautions are included.

Current Status:

37 plants - completed
 5 outstanding

Action(s) Required to Complete the Licensing Action:

In a memo dated 6/12/81, PMs of the 5 remaining plants were requested to complete this item. PMs will contact resident inspector to see if he has any problems with licensee response, and issue close-out letter. RSB assistance not expected to be needed.

Multi-Plant Action No. 8-88

75

No. of Plants: 70

Title: Lessons Learned Category B Requirements

Priority: High

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: T. Telford

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

ompletion of recommended actions in NUREG-0737

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

1. Issue Q's to 70 licensees
2. Review responses
3. Meet w/Owners Groups
4. Issue SER
5. Issue technical specifications where appropriate

_____ Action No. 6-54 _____

No. of Plants: _____

Title: Category "A" Technical Specifications _____

Priority: High _____

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: T. Telford _____

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Issue Technical Specifications for the "short term" (1/1/80 requirements) of Lessons Learned effort.

Background:

Principal issues involve containment isolation valves, valve position indicators and system integrity for radioactive effluents.

Current Status:

Action(s) Required to Complete the Licensing Action:

Review by ORPM (an estimated 8 hrs for each plant)

6-58

NO. 77-101-23

Title: BGO Reports on BWRs

Priority: Medium

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: _____

Lead Engineer: D. Verrelli

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Following TMI-2 accident, BWRs were requested to review systems, procedures and training of personnel to determine applicability of the accident to BWRs.

Background:

Current Status:

This effort is complete. However, numerous recommendations were made via Action Plan (principally II.K.3.x) to upgrade BWR reliability.

Action(s) Required to Complete the Licensing Action:

Owner's Group activity involving:

1. ADS
2. RCK/HPCI
3. Water Level Instrumentation
4. CSP/LPCF Logic for Initiation
5. Isolation condensers
6. Recirculation Pump Seal Integrity
7. Relief valve reliability and failure rates.

Multi-Plant Action No. B-57

No. of Plants:- All PWRs

Title: Decay Heat Removal Technical Specifications

Priority: Low

Cognizant Assistant Director: T. Nozak

Cognizant Branch Chief: J. Stolz

Lead Engineer: D. Garner

Lead Technical Review Branch: By PM principally

Other Review Branches: ORAB

Problem:

To assure redundancy in decay heat removal capability by incorporation of requirements into Technical Specifications.

Background:

Various events at PWRs during shutdown conditions have indicated that administrative controls to assure decay heat removal capability have been inadequate. All PWR licensees were requested (in a letter dated June 11, 1980) to incorporate requirements for redundancy into the TSs.

Current Status:

Approximately 13 plants have been completed with TS changes. All other PWRs are in review. These remaining plants are under review by EG&G through ORAB.

Action(s) Required to Complete the Licensing Action:

1. Completion of standard SER by EG&G
2. Issuance of TSs to plants which complied with our model TSs
3. Review of our position on those facilities which took issue with our concerns and did not comply
4. Appropriate action on facilities in item 3.

Multi-Plant Action No. D-59-B

No. of Plants: All Plants (EXCEPT LACBWR) - 23

Title: BWR SCRAM DISCHARGE VOLUME LONG TERM MODIFICATIONS

Priority: High

Cognizant Assistant Director: T. NOVAK

Cognizant Branch Chief: L. IPPOLITO

Lead Engineer: J. HANNON

Lead Technical Review Branch: ASB

Other Review Branches: NONE

Problem: CONCERNS RAISED AS A RESULT OF THE PARTIAL FAILURE TO SCRAM EVENT AT BROWNS FERRY 3 ON JUNE 28, 1980 WERE DOCUMENTED IN THE STAFF'S SAFETY EVALUATION REPORT ON THE BWR SCRAM DISCHARGE SYSTEM DATED DECEMBER 1, 1980. A LETTER WAS ISSUED ON OCTOBER 1, 1980 REQUESTING A COMMITMENT TO CONFORM WITH REVISED CRITERIA.

A LETTER DATED DECEMBER 9, 1980 FORWARDED THE STAFF'S GENERIC SFR WITH REVISED CRITERIA. A LETTER DATED MARCH 30, 1981 PROVIDED CLARIFICATION OF THE REQUIREMENT FOR DIVERSE INSTRUMENTATION.

Current Status:

DRAFT ORDERS AND TECHNICAL SPECIFICATIONS HAVE BEEN PREPARED AND ARE WAITING RESPONSES FROM LICENSEES WHO HAVE NOT MADE ACCEPTABLE COMMITMENTS.

Action(s) Required to Complete the Licensing Action:

OBTAIN ACCEPTABLE RESPONSES AND COMMITMENT DATES. ISSUE ORDERS WITH MODEL TECHNICAL SPECIFICATIONS. ISSUE NRC CONTRACT TO FRC TO PERFORM POST-IMPLEMENTATION VERIFICATION.

Multi Plant Action No. B-58a

No. of Plants: 411 BWRs

Title: BWR Scram Discharge Volume Technical Specifications

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: J. Hannon

Lead Technical Review Branch: ASB

Other Review Branches: None

Problem:

Common cause failures of instrument volume level switches. Precursors at Hatch and Brunswick resulted in IE Bulletin 80-14.

Background:

Model Technical Specifications were sent to the industry by letter dated July 7, 1980. With the exception of Vermont Yankee, responsive submittals were subsequently received from the affected licensees.

Current Status:

NRC contract NRC-03-81-130 has been issued to Franklin Research Center to review licensee submittals and write TIAs.

Action(s) Required to Complete the Licensing Action:

Issue license amendments upgrading TS after receipt of TERs.

Multi-Plant Action No. B-59

No. of Plants: 70

Title: Masonry Wall Design (IEB 80-11)

Priority: Medium

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: R. A. Clark

Lead Engineer: C. Trammell, NRR, H. Wong, IE

Lead Technical Review Branch: SEB

Other Review Branches: None at present - consultant (contractor) being sought to do reviews under SEB supervision

Problem:

Inadequate design criteria for masonry walls.

Background:

Problem was discovered at Trojan during reviews associated with IEB 79-02 (anchor bolts) and IEB 79-14 (As-built verification). A wall was found which supported numerous safety-related pipes which was not adequate to resist seismic forces.

Current Status:

Very few reviews conducted by SEB.

Action(s) Required to Complete the Licensing Action:

SEB will engage a contractor to do reviews. Reviews to be done at each site or A/E office. SER will be written based on meeting. To be completed over 28-30 months.

Multi-Plant Action No B-60

No. of Plants: 72

Title: Environmental Qualification of Safety Related Electrical Equipment

Priority: High

DE DL

Cognizant Assistant Director: Johnston/Novak

DE DL

Cognizant Branch Chief: Rosztoczy/Ippolito

Lead Engineer: M. Williams

Lead Technical Review Branch: EQ&

Other Review Branches: Contractor Assistance - Franklin Research
Ctr (Phila.)

Problem:

To implement the requirements of 10 CFR 50 Appendix A GDC 4 as modified by the Commission's Memorandum and Order of May 23, 1980 (80-CLI-21).

Background:

USC petition on EQ and Fire Protection requested Commission directed shutdown of operating reactors and stoppage of construction on new plants.

Current Status:

Safety Evaluations have been issued for Operating Reactors (93% complete 6/12/81)
Franklin Research will perform a review of the licensee's response to our
Safety Evaluation.

Action(s) Required to Complete the Licensing Action:

Issue Supplements to the May/June Safety Evaluations - 1/1/82
Issue final supplement to SER's for all plants - 6/30/82*

*The Qualification Requirements for Mechanical Equipment and for the Seismic and Dynamic Qualification of Safety-Related equipment will be issued as a new Multi-Plant Action Item.

Multi-Plant Action No. B-61

No. of Plants: 67 (Except Fort St. Vrain, Indian Point 1 & TMI-2)

Title: IE Bulletin 79-27, Loss of Non Class IE Instrument & Control Power

Priority: Medium

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: J. Stolz

Lead Engineer: M. Fairtile

Lead Technical Review Branch: ICSB

Other Review Branches: ORAB

Problem: Loss of "non-nuclear" instrumentation power has caused loss of instruments needed to guide operators to safely shutdown the plant during a concurrent transient caused by the same power loss. In addition erroneous inputs have been fed into the control systems of B&W plants.

Background: IE Bulletin 79-27 was issued on November 30, 1979 due to a total loss of power to a non-safety class IE 120 volt AC power supply to the Oconee 3 integrated control system and non-nuclear Instrumentation System. On February 26, 1980 there was a power failure in a DC non-nuclear instrumentation system at Crystal River 3 that resulted in both IE and ONRR writing Orders to utilities that hadn't yet responded by February 26, 1980 to the Bulletin.

Current Status:

The I&CS Branch is preparing an internal memorandum to delineate the scope and schedule of this review. The program has been dormant for the first half of 1981.

Action(s) Required to Complete the Licensing Action:

To be determined after I&CS Branch issues the above mentioned memorandum.

Licensing Action No. R-62

No. of Plants: All PWRs

Title: 120 VAC Vital Instrument Busses and Invertor Tech. Specs

Priority: 7, 8

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: J. F. Stolz

Lead Engineer: M. L. Padovan

Lead Technical Review Branch: ORAB

Other Review Branches: _____

Problem:

Class 1E 120 VAC Vital Instrument Busses and Associated Invertors are taken out of service for unlimited time periods.

Background:

A review of operating experience and plant designs shows that 120 VAC vital busses and associated invertors can be taken out of service for extended periods of time. Plant safety could possibly be jeopardized beyond FSAR assumptions.

Current Status:

Std. Technical Specifications have been drafted
Generic Letter being drafted

Action(s) Required to Complete the Licensing Action:

- 1) Send out requests to licensee's to submit revised Tech Specs
- 2) Evaluate responses
- 3) Issue Tech Specs

Revision No. B-53No. of Plants: 22Title: Interim Procedures for Station BlackoutPriority: HighCognizant Assistant Director: T. NovakCognizant Branch Chief: J. StolzLead Engineer: C. NelsonLead Technical Review Branch: PTRB

Other Review Branches: _____

Problem:

Review current plant operations to determine capability to mitigate a station blackout event and implement emergency procedures and training for blackout events.

Background:

ASAB (ALAB-603) concluded that station blackout should be considered a design basis event for St. Lucie 2. NRC staff is currently assessing station blackout events on a ~~generic~~ basis.

Current Status:

Letters sent to all operating reactors on February 25, 1981 requesting assessment of facility procedures and training programs.

Action(s) Required to Complete the Licensing Action:

Review licensees submittals for adequacy

Multi-Plant Action No. B-64
 No. of Plants: 2
 Title: B&W Accident Induced Neutron Flux Errors
 Priority: Low
 Cognizant Assistant Director: T. Novak
 Cognizant Branch Chief: J. Stolz
 Lead Engineer: M. Fairtile
 Lead Technical Review Branch: CPB
 Other Review Branches: None

Problem:

By letter dated October 29, 1980, B&W (J. H. Taylor) to IE (V. Stello), B&W stated that a study of accident-induced neutron flux errors, indicates a possible error in the FSAR analyses in the non-conservative direction affecting both 177FA and 205FA cores. IE transferred the technical review aspects to ONRR by TIA ROI 80-43 signed by Eisenhut on November 26, 1980.

Background:

By memorandum dated January 14, 1981, Eisenhut to Novak, the Multi-Plant Program B-64 was established, and transmitted questions, same date, to all B&W licensees. Responses came in dated from mid-March to mid-April 1981. The responses were routed to the Core Performance Branch for review. (H. Richings is reviewer)

Current Status:

Arkansas still owes NRC response to our January 14, 1981 letter. Core Performance Branch considering another round of questions. Review on schedule for July 81 completion.

Action(s) Required to Complete the Licensing Action:

NRC will prepare a letter and Safety Evaluation if continued operation without Tech Spec changes are indicated. Otherwise, we will request appropriate TS changes, and issuance of that license amendment will close out review.

Multi-Plant Action No. B - 65

No. of Plants: ALL BWR's - 23

Title: SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN DWR SCRAM SYSTEM

Priority: High

Cognizant Assistant Director: T. NOVAK

Cognizant Branch Chief: T. IPPOLITO

Lead Engineer: J. HANNON

Lead Technical Review Branch: ORAB

Other Review Branches: ASR, P.RB, EQB, RAB, AEB, RRAB, CSB, RSB, ORB 2, ICSB, OLP, SPEB

Problem: ASSESS THE OVERALL RISK TO LONG TERM CORE COOLING FROM POSTULATED PIPE BREAK IN SCRAM DISCHARGE VOLUME; INVESTIGATE THE MITIGATION CAPABILITY FOR THE POSTULATED EVENT; ESTABLISH GENERIC RECOMMENDATIONS FOR CORRECTING ANY DEFICIENCIES IDENTIFIED.

Background: DRAFT NUREG - 0785 ISSUED APRIL 3, 1981 BY AEOD DESCRIBED CERTAIN SAFETY CONCERNS ASSOCIATED WITH POSTULATED PIPE BREAKS IN THE DWR SCRAM SYSTEM. NRC LETTER DATED APRIL 10, 1981 TO ALL BWR LICENSEES REQUESTED A GENERIC EVALUATION OF THE CONCERNS WITHIN 45 DAYS AND A PLANT SPECIFIC REVIEW WITHIN 120 DAYS. THE GENERIC EVALUATION WAS SUBMITTED BY GE ON BEHALF OF THE BWR OWNERS BY LETTER DATED APRIL 30, 1981.

Current Status: THE STAFF HAD ESSENTIALLY COMPLETED ITS REVIEW OF THE GENERIC EVALUATION ON JUNE 12, 1981. A DRAFT SER WAS PREPARED AND SENT TO THE BWR OWNERS GROUP FOR PROMPT REVIEW AND COMMENT. AFTER COMMENTS ARE INCORPORATED, THE SER WILL BE PUBLISHED AS A NUREG.

Action(s) Required to Complete the Licensing Action:

IF A GENERIC RESOLUTION CAN BE AGREED UPON, THE NUREG WILL PROPOSE AN IMPLEMENTATION SCHEDULE. A LETTER WILL BE SENT TO ALL BWR LICENSEES REVISING THE SCHEDULE FOR THE 120 DAY RESPONSES. THE STAFF HAS EVERY REASON TO BELIEVE THAT AN ACCEPTABLE GENERIC RESOLUTION TO THIS ISSUE WILL BE OBTAINED.

Multi-Plant Action no C-01
 No. of Plants: 35 units
 Title: PWR Secondary Water Chemistry Monitoring Requirements
 Priority: Medium
 Cognizant Assistant Director: T. M. Novak
 Cognizant Branch Chief: S. A. Varga
 Lead Engineer: R. Licciardo
 Lead Technical Review Branch: RSB
 Other Review Branches: _____

Problem:

The NRC staff's testimony at the January 1976 Prairie Island public hearings on steam generator tube integrity set forth the regulatory position regarding secondary water chemistry monitoring requirements. The NRC position is that monitoring requirements are needed to assure control of harmful impurity buildup in the steam generators that could cause tube corrosion and impair tube integrity.

Background:

Letters were sent to affected PWRs requesting Tech Spec changes to incorporate secondary water chemistry monitoring requirements. A number of licensees objected to this approach based on potential limitations to Plant Operating Flexibility. NRC changed its position requiring a Tech Spec condition requiring the implementation of a secondary water chemistry monitoring control program.

Current Status:

Licensing Action completed on 23 applicable units. Action outstanding on 12 units.

Action(s) Required to Complete the Licensing Action:

- . Check for and review outstanding submittals for completion
- . A long term resolution will be developed on a generic basis considering licensee responses to this action and information deriving from completion of Category A Technical activities A-3, A-4 and A-5 "Steam Generator Tube Integrity."

Multi-Plant Action No. C-02

N. of Plants: 0

Title: BWR-Recirc. Pump Trip (ATWS)

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: Rooney

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. C-00

No. of Plants: 1 Plant Remaining (Kewaunee)

Title: Qualification of Radiation Protection

Priority: N/A

Cognizant Assistant Director: Tom Novak

Cognizant Branch Chief: John Stolz

Lead Engineer: Dom DiIanni

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Have the licensee's commit to meeting the requirements of Reg. Guide 1.8 for the qualification of the radiation protection manager.

Background:

Current Status:

All licensees have committed to this requirement and the Tech Specs have been revised accordingly except for WPS. A 60 day letter has been issued to WPS and the response is due by 8/3/81.

Action(s) Required to Complete the Licensing Action:

The PM (RLicciardo) for Kewaunee is to issue an amendment to the license after the licensee has committed to this requirement.

No. of Plants: 6

Title: ESF Filter Tech Specs

Priority: Low

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Crutchfield

Lead Engineer: R. Caruso

Lead Technical Review Branch: ETSB

Other Review Branches:

Problem:

Some licensees have not submitted TS regarding ESF filters which comply with position of R. G. 1.52.

Background:

Ltrs sent out in 1976-1977 to all licensees. Most have been completed.

Current Status:

1-2 plants have not yet responded. 3-4 plants have comments which must be resolved prior to issuance of T.S.

Action(s) Required to Complete the Licensing Action:

ETSB to complete reviews and send questions to PMs for resolution of open items.

Multi-Plant Action No. MC-05

No. of Plants: -0

Title: Conversion to Standard Technical Specifications (STS)

Priority: _____

Cognizant Assistant Director: J. M. Novak

Cognizant Branch Chief: J. F. Stolz

Lead Engineer: F. C. Wagner

Lead Technical Review Branch: Licensing Guidance Branch

Other Review Branches: _____

Problem:

Conversion of custom Tech Specs of older plants to STS

Background:

Conversion is voluntary, a number of plant have, at times, expressed an interest in this conversion.

Current Status:

3 plants have been converted; all others have been on hold or the request to convert has been withdrawn. Changes to 50.36 are being processed which should make conversion attractive when finalized.

Action(s) Required to Complete the Licensing Action:

If a plant requests conversion and LGB is able to perform the review, a conversion can be accomplished in a short time.

Multi-Plant Action No. C-06

No. of Plants:- 35

Title: Pump Support - Lamellar Tearing

Priority: Medium

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Crutchfield

Lead Engineer: R. Snaider

Lead Technical Review Branch: DST

Other Review Branches: GIB

Problem:

The potential for low fracture toughness and lamellar tearing on steam generators (including vessel, pressurizer and torus supports) exists due to variability in fracture toughness and resistance to corrosion of support materials.

Background:

Was unresolved safety issue A-12. Additional complication is that supports other than integrally welded are not covered in ASME Code and GDC and may require rule-making. Resolution of issue will occur with issuance of the "Final" NUREG-0577 in summer of 1981.

Current Status:

Action(s) Required to Complete the Licensing Action:

DST/GIB to resolve.

Multi-Plant Action No C-07

No. of Plants: 39

Title: Fuel Handling Accident Inside Containment

Priority: Medium

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Crutchfield

Lead Engineer: W. A. Paulson

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

By letter dated January 3, 1977 to Commissioner Gilinsky, Mr. Robert Pollard recommended that NRR review the potential consequences of a postulated fuel handling accident at each operating plant because this accident had not been considered during licensing of the plants.

Background:

Letters were sent to licensees of 39 operating plants in January 1977 requesting that they provide a detailed evaluation of the potential consequences of this accident. Letters were not sent to licensees of 24 operating plants because this accident had been evaluated in their SER.

Current Status:

There are eight plants which are not complete. The issues range from concerns regarding X/Q to acceptability of filters.

Action(s) Required to Complete the Licensing Action:

The Project Manager's are pursuing resolution of the remaining issues.

Multi-Plant Action No. C-08

No. of Plants:- All BWRs

Title: BWR Post LOCA H2 Control

Priority: Low

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: J. Hannon

Lead Technical Review Branch: CSB

Other Review Branches: Research

Problem:

Develop model to predict $[H_2]$ vs time Post-LOCA

Background:

The accident at TMI involved a large amount of metal-water reaction in the core with resulting hydrogen generation well in excess of the amounts specified in NRC Regulations.

Current Status:

Rulemaking proceeding on degraded core and hydrogen management in process.

Action(s) Required to Complete the Licensing Action:

Complete rulemaking
Demonstrate model

Multi-Plant Action no. C-9

No. of Plants: 7

Title: PWR Aux. FW Pumps

Priority: High

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: S. Varga

Lead Engineer: M. Grotenhuis

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

This review covers the seismic design criteria of the condensate storage tanks for several plants in Region III; Kewaunee, Prairie Island, Point Beach and Zion.

Background:

This concern arose over some inspectors concerns and was referred to NRR for review.

Current Status:

This project is incorporated into B-50 (Auxiliary Feedwater system evaluation) is now also covered under C-14 (Auxiliary Feedwater Seismic Qualification)

Action(s) Required to Complete the Licensing Action:

None

Project Number: MC-10

No. of Plants: 54

Title: Control of Heavy Loads at Nuclear Power Plants

Priority: Phase I, High; Phase II, Medium

Cognizant Assistant Director: Tom Novak

Cognizant Branch Chief: Robert Clark

Lead Engineer: Glode Requa

Lead Technical Review Branch: Auxiliary Systems Branch

Other Review Branches: None

Problem: Dropped Heavy Loads may impact spent fuel, fuel in the core, or equipment that may be required to achieve safe shutdown and continued decay heat removal. If sufficient spent fuel or fuel in the core were damaged and if the fuel is highly radioactive, the potential releases of radioactive material could result in off-site doses that exceed 10 CFR Part 100 limits.

Background: USI - Task A-36 was established to examine staff licensing criteria, the adequacy of measures in effect at operating plants, and to recommend necessary changes to assure the safe handling of Heavy Loads. NUREG-0612 provides the results of the review and includes the task groups recommendations.

Current Status: NUREG-0612 was issued to all licensees by letter dated December 22, 1980. The letter requested; interim actions to be completed in 90 days, a Phase I Action (report, confirmation and justification) in six months and Phase II (specific requirements) in nine months.

Actions Required to Complete the Licensing Action:

1. Review licensees Phase I submittals, resolve any staff disagreements with submittals and issue a SER to licensee.
2. Review licensee's Phase II submittals same as item 1.

NOTE: DL has made arrangements with Franklin Research Center to review licensee's submittals.

Multi-Plant Action No. C-ii

No. of Plants: 22

Title: Reactor Protection System (RPS) Power Supplies

Priority: _____

Cognizant Assistant Director: T. Noyak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: J. VanVliet

Lead Technical Review Branch: ORAB (with Livermore assistance)

Other Review Branches: None

Problem:

Design deficiencies have been identified in the BWR RPS MG set regulator systems that could adversely affect the operability of the RPS if (1) a sequence of undetected single component failures were to occur, or (2) a seismic event occurred that resulted in the postulated component malfunctions.

Background:

The design deficiency was first identified during the Hatch 2 licensing process. By letter dated August 7, 1978 the other BWRs were directed to review their designs and submit a report. That letter also directed that a surveillance program be implemented. By letter dated September 24, 1980, the licensees were requested to: (1) commit to installing a Class 1E system, (2) provide a schedule for completion of the modification, and (3) provide a schedule for submitting design information and proposed Technical Specifications. All licensees responded satisfactorily.

Current Status:

Hatch 1 & 2 are complete. 4 other BWRs have submitted design information which is presently being reviewed. The remaining BWRs are not yet scheduled to submit information.

Action(s) Required to Complete the Licensing Action:

- 1) Review licensee's designs (post-implementation)
- 2) Review licensee's proposed TS changes
- 3) Issue license amendment that revises TS.

Multi-Plant Action No. C-12

No. of Plants: 6

Title: Boron Solubility During Long Term Cooling Following LOCA

Priority: Medium

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: R. A. Clark

Lead Engineer: T. Colburn

Lead Technical Review Branch: RSB

Other Review Branches: _____

Problem:

Review of the Emergency Core Cooling System (ECCS) predicted performance has led to concerns, for most PWRs designed by Combustion Engineering, regarding the suitability of systems available to prevent boron precipitation during long term cooling following a LOCA. Boron concentrations above the solubility limit would result in precipitation of boron, thus blocking the Reactor Pressure Vessel flow channels and thereby limiting ECCS flow.

Background:

The results of the above review were transmitted to the affected licensees by letters dated 9/7/77 (Palisades), 9/19/77 (Ft. Calhoun and Millstone 2), and 9/27/77 (St. Lucie 1 and Calvert Cliffs 1 and 2). Initial responses were received and a clarification meeting of the NRC staff with the licensees was held on 11/29/77. Additional licensee responses were received as requested during the meeting.

Current Status:

Calvert Cliffs 1 and 2 have addressed this problem in their Technical Specifications to the satisfaction of the ORPM. The ORPM feels licensing action is complete for these units as of 6/16/81. The action for Fort Calhoun is complete as of 3/10/81 with the issuance of an SER to that licensee. All other licensees have completed their response to our generic letter and are awaiting evaluation except Palisades which recently met with the NRC to discuss a revised proposal. We are awaiting their formal submittal.

Action(s) Required to Complete the Licensing Action:

1. Submittal required from Palisades
2. Approval of remaining licensing submittals by DSI, specifically RSB
3. Issuance of SER to DL for transmittal to licensees by the Project Managers
4. For those plants not approved (if any) request for additional information or request for Technical Specification change, as appropriate.

Multi-Plant Action No. C-13

No. of Plants: 72

Title: Loss of Offsite Power

Priority: Medium

Cognizant Assistant Director: Gus Lainus

Cognizant Branch Chief: D. Crutchfield

Lead Engineer: Lombardo

Lead Technical Review Branch: PSB

Other Review Branches: _____

Problem:

Study being performed to reevaluate the reliability of the offsite power system as the preferred emergency source of power after events occurring at several power plants.

Background:

Turkey Point and Indian Point blackouts raised concerns as to validity of using the offsite power system for the preferred emergency source of power.

Current Status:

Survey of the industry loss of power completed.

Action(s) Required to Complete the Licensing Action:

C-13 will be used for input into A-44 Station Blackout.

No. of Plans: 46

Title: Seismic Qualification of Auxiliary Feedwater Systems

Priority: High

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: W. Russell

Lead Engineer: K. Herring

Lead Technical Review Branch: SEPB

Other Review Branches: ORAB

Problem:

Certain PWRs are known to have auxiliary feedwater (AFW) systems with portions which are not seismically qualified. This action plan was formulated to evaluate the capability of the AFW systems for all operating PWRs to withstand the occurrence of earthquakes and to the extent practicable increase that capability up to and including the SSE for those plants without seismically qualified AFW systems, either in whole or in part.

Background:

On August 8, 1980, the Division of Safety Technology (DST) transmitted the results of a study on the seismic capability of decay heat removal systems to the Division of Licensing (DL). That study included a simplified probabilistic risk analysis and recommendation for action by DL regarding the seismic capability of the AFW systems for all operating PWRs. This action plan was formulated in response to those recommendations.

Current Status: 10 CFR 50.54(f) letters dated February 10, 1981 have been sent to all operating PWRs requesting details regarding the seismic capability of the AFW system, or alternate seismically qualified decay heat removal plants for those plants without an entirely qualified AFW system. All responses are scheduled for receipt by August 1981 with responses beginning to be received in June 1981.

Action(s) Required to Complete the Licensing Action:

The licensee responses to the 10 CFR 50.54(f) letters will be reviewed. For those plants with inadequate seismic qualification for decay heat removal systems, particularly the AFW systems, backfit considerations, if necessary, will be made on a case by case basis.

Project Number: 25
 Title: Mark I Containment Program

Priority: _____

Coordinating Assistant Director: T. Nover

Coordinating Branch Chief: T. Ippolito

Lead Engineer: Byron Siegel

Lead Technical Review Branch: None-Worked to be performed under Technical Assistance Contract

Other Review Branches: ORAB & Chris Grimes

Problem:

Since the original design of the Mark I containment system, additional loading conditions resulting from the dynamic effects of drywell air and steam being rapidly forced into the suppression pool during a postulated LOCA and from suppression pool response to SRV operation generally associated plant transient operating conditions have been identified. The objective of this program is to restore the originally intended design safety margins for each Mark I containment system.

Background:

A Mark I Owners Group (OG) with GE the lead technical organization was formed to quantify the hydrodynamic loads and assess the effects of these loads on the Mark I containment structure. The OG divided this task into a short term (STP) and long term program (LTP).

The objectives of the STP were to verify that each Mark I containment system would maintain its integrity and functional capability when subject to the most probable loads induced by a postulated design-basis LOCA, and to verify that licensed Mark I BWR facilities could continue to operate safely, without endangering the health and safety of the public, while a methodical, comprehensive LTP was being conducted. The staff in the "Mark I Containment Short-Term Program Safety Evaluation Report," NUREG-0408 concluded that sufficient margin of safety had been demonstrated to assure the functional performance of the containment system and, therefore, any undue risk to the health and safety of the public was precluded.

The objectives of the LTP were to establish design-basis loads that are appropriate for the anticipated life of each Mark I BWR facility (40 years), and to restore the originally intended design-safety margins for each Mark I containment system. The principle thrust of this program has been the development of generic methods for the definition of suppression pool hydrodynamic loading events and the associated structural assessment techniques for the Mark I configuration. The

generic analysis techniques are intended to be used to perform a plant-wide analysis (PWA) for each Mark I facility. The staff in the "Mark I Containment Long-Term Program Safety Evaluation Report," NUREG-0661 concluded that the load definition procedures utilized by the BWR Owners Group, as modified by the staff's requirements, provide conservative estimates of these loading conditions and that the structural acceptance criteria are consistent with the requirements of the applicable codes and standards.

Current Status:

The licensees are in the process of performing PVA and designing and installing modifications to meet the Commission's Order date for each operating plant. However, based on the May 22, 1981 meeting with the BWR Mark I OG it appears many licensees will not meet the Commission's Order dates because the results of the final torus analysis is needed for inputs to the torus attached piping analysis. The OG is preparing a submittal which identifies the status of the modifications and establishes revised completion dates for this program.

Action(s) Required to Complete the Licensing Action:

1. Staff review and evaluation of OG Status Report
2. Preparation of a Commission paper justifying extension of order dates
3. Submittal of the PVA to the NRC
4. Completion of major and minor and torus attached piping modifications.
5. Post implementation audit of PVA by Franklin Institute and BNL on Technical Assistance Contracts
6. Issuance of SER to each licensee on acceptability of Mark I LTP modifications and analysis for their plant.

Multi-Plant Action No. D-03

No. of Plants: 9

Title: Pressurizer Heatup Rate Error

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: Neighbors

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. D-04

No. of Plants:- 42 (All PWRs)

Title: PWR Reactor Vessel Cavity Seal Ring Missile Potential

Priority: Medium

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: R. A. Clark

Lead Engineer: C. Trammell

Lead Technical Review Branch: MTEB

Other Review Branches: None

Problem:
The reactor vessel cavity seal ring can become a missile if left in place during operation should a LOCA occur in or near the reactor vessel cavity.

Background:

Trojan disclosed this as a possible problem via an LER. On 2/2/78, letters were sent to all PWR licensees requesting a solution by either (a) removing the seal ring during operation or (b) confirming by analysis that the seal ring can withstand blowdown forces

Current Status:

38 plants - completed
4 remain

Action(s) Required to Complete the Licensing Action:

The remaining 4 plants have submitted "leak-before-break" analyses in response to generic issue A-2 "Asymmetric LOCA Loads." It is expected that NRC will accept this analysis. If so, this will also solve D-4 for these plants, since the seal ring will not become a missile under "leak" conditions.

Multi-Plant Action No. U-05

No. of Plants: 6

Title: Plant Upper Plenum Injection (UPI) Model

Priority: Low

Cognizant Assistant Director: T. M. Novak

Cognizant Branch Chief: S. A. Varga

Lead Engineer: R. Licciardo

Lead Technical Review Branch: RSB

Other Review Branches: _____

Problem:

The ECCS evaluation model for the Westinghouse 2 loop plants. Considers ECCS water, which is actually injected into the upper plenum, as if it were directly injected into the lower plenum.

Background:

Related units are operating on an interim acceptance model; this multiplant action is to provide a long term resolution.

Current Status:

RSB has reasonable assurance that the interim acceptance model adequately covers any concerns until a generic resolution can be achieved. No activity at this time based on relatively low priority.

Action(s) Required to Complete the Licensing Action:

- 1) Develop an acceptable generic resolution on basis of improved models becoming available
- 2) Plant specific application:
Issue Questions to Licensees; Review Responses; Prepare SER; Modify Tech Specs

Licensing Action No. MD-00No. of Plants: 0Title: Peaking Model Change For CE CoresPriority: CompleteCognizant Assistant Director: Tom NovakCognizant Branch Chief: Bob ClarkLead Engineer: Monte Conner

Lead Technical Review Branch: _____

Other Review Branches: _____

Description: In late 1977, CE informed the NRC of nonconservative results in their analysis of record for calculating water hole peaking factors of the CE facilities. New analysis indicated that both the LSSS and LCO limits protecting DNBR and peaking factors should be reduced.

Action Required to Complete the Licensing Action:

Find a way to get completed actions out of this system.

Utility-Plant Action No. U-07No. of Plants: 2Title: PWR Power Level for RUMPriority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: R. Clark

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi Plant Action No D-09

No. of Plants: 0

Title: GE ECCS Input Errors

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: Rooney

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multiple Plant Action No. D-10

No. of Plants: 29

Title: unresolved Safety Issue - Asymmetric Blowdown Loads on Reactor
Primary Coolant Systems

Priority: medium

Cognizant Assistant Director: J. Knight

Cognizant Branch Chief: R. Bosnak

Lead Engineer: J. Rajan

Lead Technical Review Branch: MER

Other Review Branches: MTEB, CPB, CSB

Problem:

Potential asymmetric forces and magnitudes could damage RV supports, RV internals, and other primary system components. Core geometry could be impaired as a result, and core cooling could be in jeopardy.

Background:

January 1978, all licensees of operating PWRs were required by NRC to provide an assessment of adequacy of RV supports and other affected structures and systems to withstand asymmetric LOCA loads including an assessment of the effects of asymmetric loads produced by various pipe breaks both inside and outside the reactor vessel cavity. This potential safety problem was subsequently identified as an unresolved safety issue Task Action Plan A-2.

Current Status:

All PWR plant assessments for asymmetric loads have been received and are currently being evaluated by staff and contractor EG&G. As a basis for this evaluation, staff is implementing criteria developed as part of the Task Action Plan A-2. These criteria are included in NUREG 0609. Concurrently, a few PWR licensees have performed a study of "leak before break". Staff action is likely to affect resolution of this safety issue.

Action(s) Required to Complete the Licensing Action:

Complete Owners Group submittals and plant specific submittals and issue SER.

License Action No. D-11

No. of Plants: 0

Title: Fission Gas Release

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: M. Fletcher

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. D-12

No. of Plants: 1

Title: BWR Non Jet Pump Core Spray Performance

Priority: Low

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: P. J. Polk

Lead Technical Review Branch: ORAB

Other Review Branches: To be determined

Problem:

Core Spray System nozzle coverage differs in a steam environment vis-a-vis an atmospheric environment. The original design is based on atmospheric analyses and tests. The generic issue requires that nozzle coverage be reanalyzed to reflect the steam environment.

Background:

Current Status:

All non-Jet Pump BWRs have either made the necessary modifications or committed to same with the exception of Nine Mile Point.

Action(s) Required to Complete the Licensing Action:

Review and approval of both the Oyster Creek and Nine Mile Plants is necessary to complete this generic activity.

Multi-Plant Action No. D-13

No. of Plants: _____

Title: B&W Small Break Error

Priority: Complete

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: M. Fairtile

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Generic Issues Report dated 5/16/81 does not reflect completion of this Action; however, last TAC was completed June 11, 1980.

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. D-14 _____

No. of Plants: _____

Title: Reactor Vessel Weld W/e Deficiency _____

Priority: Complete _____

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: M. Fairtile _____

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. D-15
No. of Plants: All
Title: High Energy Line Break and Consequential System Failure
Priority: Low
Cognizant Assistant Director: T. M. Novak
Cognizant Branch Chief: S. Varga
Lead Engineer: M. Grotenhuis
Lead Technical Review Branch: SIB
Other Review Branches: _____

Problem:

NRC required all LWR licensees to review their plants to determine whether control system failures caused by high energy line breaks (HELB) could result in consequences more severe than previously analyzed for the HELB.

Background:

This problem arose due to an LER issued by Salem 1. All licensees were asked to review their plants and respond regarding the status.

Current Status:

All utilities responded. The responses were reviewed. Continued operation was found to be acceptable. Task force wrote final report with some further concern. This further concern turned out to be included in TAP IIC1.

Action(s) Required to Complete the Licensing Action:

Letter to Utilities re: results
Further review will be under Task Action Plan II.C.1

Multi-Plant Action No. D-16

No. of Plants: 70

Title: Review of Corporate Management Capability

Priority: Low

Cognizant Assistant Director: Tom Novak

Cognizant Branch Chief: Steve Varga

Lead Engineer: Syd Miner

Lead Technical Review Branch: Licensee Qualification Branch

Other Technical Review Branches: N.A.

Problem: to develop criteria and review for conformance to the criteria for onsite and offsite organizations, both management and technical, that will assure the safe operation of the plants during normal and abnormal conditions and the capability necessary to respond to accident situations.

Background: In a letter dated June 29, 1979, shortly after the TMI-2 accident, we requested all power reactor licensees to supply information on their specific management and technical resources.

Upon review of the licensees' responses, we sent a second letter in February of 1980 stating that additional information is necessary to complete the evaluation. We also informed the licensees at that time that an outside contractor (Technicon Research, Inc.) in assisting the NRC in developing criteria, would contact them and request the information concerning their utility. Technicon reviewed the responses and the results of their effort were published in NUREG.

Current Status: Subsequently, the Licensee Qualification Branch (LQB) developed guidelines and published them in NUREG-0731 and at the present time are revising this edition. No definite schedule has been established for this effort because of its low priority. However, issuance of a revised edition can be expected to take about 6 months to a year.

Action(s) Required to Complete the Licensing Action:

1. Issuance of guidelines by LQB.
2. Issuance of requirements by NRR to all operating plants.
3. Review of licensees' responses.
4. Issuance of a Safety Evaluation Report.

Multi-Plant Licensing Action No. D-17

No. of Plants: 31

Title: Definition of OPERABLE

Priority: Medium

Cognizant Assistant Director: Tom Novak

Cognizant Branch Chief: Steve Varga

Lead Engineer: Syd Miner

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem: Clarification of the term "OPERABLE" as it applies to support system outages or multiple outages of redundant components.

Background: In a letter dated April 10, 1980, an explanation of the problem, plus accompanying illustrate examples of recommended changes contained in the associated bases, were delineated to all licensees. The licensees were required to submit proposed changes to tech specs applicable to their plant and to implement procedures designed to assure compliance with the specifications containing new requirements.

Current Status: To date, out of sixty total responses, 31 remain active and the remainder are complete.

Action(s) Required to Complete the Licensing Action:

1. Approval of remaining changes to technical specifications of affected plants.
2. Issuance of SER's.

Multi-Plant Action No. E-01

No. of Plants: 8

Title: Spent Fuel Pool Expansions

Priority: high

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: R. Clark

Lead Technical Review Branch: ORB #2

Other Review Branches: MEB, RSB, CEB, SEB, CPB, AEB, RAB, ETSB, EEB

Problem:

Need to expand onsite storage capacity for spent fuel

Background:

In March 1981, DOE announced a change in their spent fuel management policy and program. The Federal permanent repository has been delayed to beyond the year 2000. Many plants that expanded their SFP storage capacity on the basis that a Federal repository would be available in 1985, are reapplying for approval to further expand the SFP storage capacity.

Current Status:

Most of the safety evaluation for each application is plant-specific. About half the Environmental Impact Appraisal can be handled on a generic basis.

Action(s) Required to Complete the Licensing Action:

Revise generic portion of EIAs as appropriate to reflect current DOE/NRC policies, status in industry, etc. Most recent revision was June 3, 1981.

Multi-Plant Action No. ___ F-02 ___

No. of Plants:- ___ 0 ___

Title: ___ Fuel Cask Drop ___

Priority: ___ Complete ___

Cognizant Assistant Director: _____

Cognizant Branch Chief: _____

Lead Engineer: ___ Neighbors ___

Lead Technical Review Branch: _____

Other Review Branches: _____

Problem:

Background:

Current Status:

Action(s) Required to Complete the Licensing Action:

Multi-Plant Action No. E-03

No. of Plants: 40

Title: Core Reloads Requiring Prior NRC Approval

Priority: Complete

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: S. Varga

Lead Engineer: W. Ross

Lead Technical Review Branch: ORB #1

Other Review Branches: _____

Problem:

Need for review of non 50.59c reviews of core reload

Background:

Generic action initiated to cover staff reviews of reloads

Current Status:

Terminated by memo Ross to Eisenhut 3/26/81

Action(s) Required to Complete the Licensing Action:

None

Multi-Plant Action No. I-04

No. of Plants: 20

Title: BWR Single Loop Operation

Priority: Low

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: T. Ippolito

Lead Engineer: R. Clark

Lead Technical Review Branch: ORAB

Other Review Branches: RSB, ICSB

Problem:

The standard BWR Technical Specifications do not permit a plant with two recirculation loops to operate either with one loop or in the natural circulation mode. Most operating BWRs have requested approval to operate with a single loop if problems develop with a jet pump, valves, seals, flow controller, etc. in the other loop. Most of the potential safety reviews can more appropriately be resolved on a generic basis.

Background:

Single loop operation at limited power levels has been authorized for 6 BWRs for short periods of time (one week to two months). Information developed during these operations is being factored into the generic review.

Current Status:

A meeting is being arranged with BWR licensees and GE to resolve the major question on flow stability. RSB is completing the plant-specific ECCS analysis reviews for 5 facilities. EG&G is completing any plant-specific instrumentation and control system reviews.

Action(s) Required to Complete the Licensing Action:

RSB and ICSB to provide generic safety evaluation.

Multi-Plant Action No. E-05

No. of Plants: 2

Title: Westinghouse N-1 Loop Operation

Priority: Low

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: S. Varga

Lead Engineer: L. Olshan

Lead Technical Review Branch: RSB

Other Review Branches: ICSB

Problem:

All Westinghouse plants have a provision in their license that prohibits operation with less than all loops(N) in service.

Background:

It may be desirable to operate plants with one loop out of service. Present restrictions in the license do not permit this type of operation.

Current Status:

Only two Westinghouse plants, Beaver Valley and Zion (1 and 2), have submitted proposed amendments for N-1 loop operation. Beaver Valley is lead plant and most of the review has been completed.

Action(s) Required to Complete the Licensing Action:

NRC to review contractor (Franklin Inst., Lawrence Livermore) inputs and write SER on Beaver Valley. Zion review to follow. Other Westinghouse plants will probably submit similar applications if Beaver Valley is approved.

No. of plants: 6

Title: CEA Position Indication

Priority: _____

Cognizant Assistant Director: Tom Novak

Cognizant Branch Chief: Bob Clark

Lead Engineer: Monte Conner

Lead Technical Review Branch: Instrumentation & Control System Br.

Other Review Branches: _____

Problem: Should we allow the use of the fully inserted or withdrawn reed switches for TS position indications.

Background: Several licensees with CE designed NSSS requested substitution of fully inserted or withdrawn reed switches for either one of the two other CEA position indication systems required by the STS for reactor startup or change of mode.

Current Status: No review of this issue has ever been started.

Actions Required to Complete the Licensing Action:

1. To approve or provide basis for denial.
2. Issue TS change or denial letter.

Licensee Application No. ME-07
No. of Plants: 5
Title: RPS Logic for CE Facilities
Priority: Medium
Cognizant Assistant Director: Tom Novak
Cognizant Branch Chief: Bob Clark
Lead Engineer: Monte Conner
Lead Technical Review Branch: ICSB
Other Review Branches: LASL

Problem: A TS discrepancy exists in the length of time the fourth RPS channel may be bypassed at CE designed facilities.

Background: In the Calvert Cliffs and St. Lucie STS, the RPS is controlled as a 2-out-of-4 logic system. However, the same RPS is operated as a 2-out-of-3 logic system with an installed spare in accordance with the Millstone-2 STS and the Ft. Calhoun, Maine Yankee and Palisade custom TS.

Current Status:

1. Waiting response from the four CE licensees on their facility's channel separation.
2. Planning a management meeting to review results of LASL data and make decision on remaining plants.

Actions Required to Complete the Licensing Action:

1. LASL will review the response to our request from the four CE licensees.
2. After management meeting, course of action will be developed.
3. Take final action (approve existing CE installation or order new TS).

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