



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated July 7, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

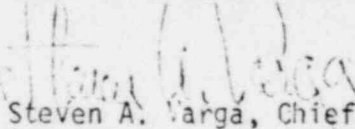
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 72, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 9, 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NO. 50-281

Revise Appendix A as follows:

Remove Page

Insert Page

4.17-1

4.17-1

4.17-2

4.17-2

4.17 SHOCK SUPPRESSORS (SNUBBERS)

Applicability

Applies to all hydraulic shock suppressors (snubbers) which are required to protect the reactor coolant system and safety related systems.

Objective

To specify the minimum frequency and type of surveillance to be applied to the hydraulic snubbers listed in Table 4.17-1 and 4.17-2.

Specification

- A. All hydraulic shock suppressors whose seal material has been demonstrated by operating experience, lab testing or analysis to be compatible with the operating environment shall be visually inspected.

This inspection shall include but not necessarily be limited to, inspection of the hydraulic-fluid reservoir, fluid connections, and linkage connections to the piping and anchor to verify snubber operability in accordance with the following schedule:

| Number of Snubbers Found Inoperable During Inspection or During Inspection Interval | Next Required Inspection Interval |
|---|-----------------------------------|
| 0 | 18 Months \pm 25% |
| 1 | 12 Months \pm 25% |
| 2 | 6 Months \pm 25% |

| | | |
|----------|----------|--------------|
| 3, 4 | 124 Days | $\pm 25\%$ |
| 5, 6, 7 | 62 Days | $\pm 25\% *$ |
| ≥ 8 | 31 Days | $\pm 25\%$ |

The required inspection interval shall not be lengthened more than one step at a time.

Snubbers may be categorized into two groups, "accessible" or "inaccessible" based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

- B. All hydraulic snubbers whose seal material are other than ethylene propylene or other material that has been demonstrated to be compatible with the operating environment shall be visually inspected for operability every 31 days.
- C. The initial inspection shall be performed within 6 months from the date of issuance of these specifications. For the purpose of entering the schedule into specification 4.17-A, it shall be assumed that the facility had been on a 6 month inspection schedule.
- D. Once each refueling cycle, a representative sample of 10 hydraulic snubbers or approximately 10% of the hydraulic snubbers, whichever is less, shall be functionally tested for operability including verification of proper piston movement, lock-up and bleed.

*The current inspection interval for Unit 2 may be extended to September 10, 1981.