7/15/81

IEC 81-09

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FROM: REGION ITT

American Electric Power Service Corporation D. C. Cook 1, 2 (50-315, 50-316)

Cincinnati Gas and Electric Company Zimmer (50-358)

Cleveland Electric Illuminating Company Perry 1, 2 (50-440, 50-441)

Commonwealth Edison Company Braidwood 1, 2 (50-456, 50-457) Byron 1, 2 (50-454, 50-455) Dresden 1, 2, 3 (50-10, 50-237, 50-249) LaSalle 1, 2 (50-373, 50-374) Quad-Cities 1, 2 (50-254, 50-265) Zion 1, 2 (50-295, 50-304)

Consumers Power Company Big Rock Point (50-155) Palisades (50-255) Midland 1, 2 (50-329, 50-330)

Dairyland Power Cooperative LACBWR (50-409)

Detroit Edison Company Fermi 2 (50-341)

Illinois Power Company Clinton 1, 2 (50-461, 50-462)

Iowa Electric Light & Power Company Duane Arnold (50-331)

Northern Indiana Public Service Company Bailly (50-367)

Northern States Power Company Monticello (50-263) Prairie Island 1, 2 (50-282, 50-306)

Public Service of Indiana Marble Hill 1, 2 (50-546, 50-547)

Toledo Edison Company Davis-Besse 1 (50-346)

Union Electric Company Callaway 1, 2 (50-483, 50-486)

Wisconsin Electric Power Company Point Beach 1, 2 (50-266, 50-301)

Wisconsin Public Service Cor, ration Kewaunee (50-305)

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UNITED STATES NUCLEAR REGULATORY COMMISSION OFFICE OF INSPECTION AND ENFORCEMENT WASHINGTON, D.C. 20555

July 10, 1981

IE Circular No. 81-09: CONTAINMENT EFFLUENT WAVER THAT BYPASSES RADIOACTIVITY MONITOR

Description of Circumstances:

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At Indian Point Units 2 and 3 and at H. B. Robinson Unit 2, licensee reviews of service water systems have identified unmonitored effluent paths from containment. Although containment cooler water effluent is monitored, containment cooler fan motor cooling water bypasses the monitors by joining the containment cooler water effluent downstream of the radiation monitoring equipment. This represents a potential unmonitored release path if the containment is at design pressure due to a design basis accident (DBA) and if leaks are present in the fan motor cooler system. Similar designs may exist at other plants. Appropriate monitoring of direct discharges (from containment to the environment following a DBA) having the potential to exceed the limits specified in 10 CFR Part 20 is required.

Recommended Actions:

- All water system effluents that are not automatically isolated by a high-1. containment-pressure containment isolation signal and that flow directly to the environment from containment should be reviewed to determine whether or not a pathway exists for "significant" unmonitored discharge. A "significant" discharge, for purposes of this circular, is a discharge where projected concentrations in unrestricted areas are likely to exceed the concentrations listed in 10 CFR Part 20, Appendix B, Table II, column 2 with the containment at design pressure due to a design basis accident and with maximum credible leakage, such as a single completely severed cocler tube, assumed to be present in the water system inside containment. You may take credit for design pressure in the water system being higher than containment design pressure only for cases where neither single failures, nor operation in degraded modes as permitted by Technical Specifications under a limiting condition of operation (LCO), are likely to result in operation of the water system at water pressures lower than the containment design pressure.
- 2. All water system effluents that are not automatically isolated by a highcontainment-pressure containment isolation signal and that flow directly to the environment from containment should be reviewed to determine whether or not any "significant" radioactive discharge can be isolated once it is detected. The review should include evaluation of the capability of the system to be isolated without interruption of any safety-related functions. Isolation of the system's inlet as well as its discharge may be required

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to prevent radioactive discharge through the inlet piping to the inlet piping of a parallel system and/or to the environment.

 Corrective actions to install detection and isolation methods that provide performance consistent with Technical Specification requirements shruld be initiated for any "significant" unmonitored and/or unisolable discharge pathways.

Although no written response to this circular is requested, a report and corrective actions may be required by applicable Technical Specifications in the event an unmonitored and/or unisolable effluent pathway is identified. If you desire additional information regarding this matter, please contact the appropriate IE Regional Office.

Attachment: Recently Issued IE Circulars

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Attachment IEC 81-09 July 10, 1981

RECENTLY ISSUED IE CIRCULARS

81-10Steam Voiding in the Reactor Coolant System During Decay Heat Removal Cooldown7/2/81All pow facilit OL or C81-08Foundation Materials5/29/81All pow facilit OL or C81-07Control of Radiactiviely Contaminated Material5/14/81All pow facilit OL or C81-06Potential Deficiency Affecting Certain Foxboro 20 to 50 Milliampere Transmitters4/14/81All pow facilit OL or C	to
81-08Foundation Materials5/29/81All pow facilit OL or C81-07Control of Radiactiviely Contaminated Material5/14/81All pow facilit OL or C81-06Potential Deficiency Affecting Certain Foxboro 20 to 50 Milliampere Transmitters4/14/81All pow facilit OL or C	ver reactor ties with an CP
81-07Control of Radiactiviely Contaminated Material5/14/81All pow facilit 0L or81-06Potential Deficiency Affecting Certain Foxboro 20 to 50 Milliampere Transmitters4/14/81All pow facilit 	ver reactor ties with an CP
81-06Potential Deficiency Affecting4/14/81All powCertain Foxboro 20 to 50facilitMilliampere Transmitters0L or 0	ver reactor ties with an CP
	ver reactor ties with an CP
81-05 Self-Aligning Rod End Bushings 3/31/81 All por for Pipe Supports facilit OL or (ver reactor ties with an CP
81-04 The Role of Shift Technical 4/30/81 All pow Advisors and Importance of facilit Reporting Operational Events OL or r	ver reactor ties with an near-term OL
81-03 Inoperable Seismic Monitoring 3/2/81 All pow Instrumentation facilit OL or (ver reactor ties with an CP
81-02 Performance of NRC-Licensed 2/9/81 All pow Individuals While on Duty facilit & test or CP	ver reactor ties (research) with an OL
81-01 Design Problems Involving 1/23/81 All pow Indicating Pushbutton facilit Switches Manufactured by an OL of Honeywell Incorporated	wer reactor ties with or CP
80-25 Case Histories of 12/5/80 All radi Radiography Events licensee	iography es

OL = Operating Licenses CP = Construction Permit

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