(7.77) LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: - 0 0 0 3 4 1 1 1 1 0 57 CAT 58 5 A D 2 2 0 0 0 - 0 0 0 0 LICENSEE CODE CONT L 6 0 5 0 0 0 2 6 5 7 0 5 1 8 8 1 8 0 6 1 0 8 1 0 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE (9) REPORT SOURCE 0 1 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) At 0010 on May 18, 1981, while performing surveillance test QOS 1000-3, per 0 2 Technical Specification 4.9.E., valve MO-2-1001-7B was closed but was unable to be 0 3 re-opened. The probable consequences of this occurrence are minimal. The other 0 4 three RHR pumps were fully operable therefore, the LPCI mode of RHR remained 0 5 unaffected by this occurrence. 0 5 80 COMP. VALVE SYSTEM CAUSE CAUSE SUBCODE COMPONENT CODE SUBCODE SUBCODE CODE CODE F | (15 ZI (16) Y (14 X F 18 REVISION REPORT OCCURRENCE SEQUEL TIAL NO. TYPE REPORT NO. CODE EVENT YEAR LER/RO 0 03 01 1 8 2 REPORT 32 NUMBER COMPONENT PRIME COMP. NPRD-4 ATTACHMENT SUBMITTED SHUTDOWN HOURS (22) ACTION FUTURE TAKEN ACTION MANUFACTURER FORM SUB. SUPPLIER METHOD ON PLANT G 0 8 Y 23 Y (24) (25) A (26)0 0 0 01 -Z (21) (18) Z (19) Z-(20) A CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The auxiliary contacts in the circuit breaker failed to disengage. Therefore, the opening coil could not energize disabling the valve. The corrective action was to immediately manually open the valve. The auxiliary contacts were then replaced and the valve demonstrated operable at 1505 on May 18, 1981. 80 9 METHOD OF DISCOVERY DESCRIPTION (32) (30)FACILITY OTHER STATUS DISCOVERY % POWER Routine Surveillance B (31) NA 914 01 1E 80 44 9 10 ACTIVITY CONTENT 13 LOCATION OF RELEASE AMOUNT OF ACTIVITY (35 RELEASED OF RELEASE NA NA Z 34 Z (33) 80 PERSONNEL EXPOSURES DESCRIPTION 39 TYPE NUMBER (37) Z 10 10 0 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER NA 0 10 40 80 11 12 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION TYPE NA Z (42) 80 8107160395 810610 NRC USE ONLY PUBLICITY 05000265 PDR DESCRIPTION (45 PDR ADOCK S N (44) NA 69 68 PHONE 309-654-2241, ext. 172 J. KOFACZ NAME OF PREPARER

- 1. LER NUMBER: LER/RO 81-12/03L-0
- II. LICENSEE NAME: Commonwealth Edison Company Quad-Cities Nuclear Power Station
- III. FACILITY NAME: Unit Two
- IV. DOCKET NUMBER: 050-265
- V. EVENT DESCRIPTION:

At 0010 on May 18, 1981, while performing surveillance testing, QOS 1000-3, for a Unit Two Diesel Generator outage, valve MO-2-1001-7B was closed but would not re-open from the Control Room. After repeated attempts were made to open the valve from the Control Room without success, the valve was manually opened.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The safety aspects of this event are minimal. The failure of the MO-2-1001-7B valve to open prohibited the "B" RHRS pump to take its normal suction from the pressure suppression pool, thereby rendering the pump inoperable for use in the LPCI mode of RHRS. However, the other three RHRS pumps were fully operable, hence the LPCI and containment cooling modes of the RHRS were still capable of performing their designed functions.

VII. CAUSE:

The cause of the deviation was equipment failure. The auxiliary contacts in the circuit breaker failed to disengage. With the auxiliary contacts engaged, the opening coil was unable to energize, therefore disabling the valve.

VIII. CORRECTIVE ACTION:

The valve was immediately manually opened. Work Request Q12549 was initiated to investigate and correct the cause of the deviation. The auxiliary contacts in the circuit breaker were replaced like for like under this Work Request. M0-2-1001-7B was tested satisfactorily at 3:05 p.m. on May 18, 1981. No further corrective action was necessary. On September 5, 1979, the M0-2-1001-7C valve failed to close during testing due to faulty auxiliary contacts.