BALTIMORE GAS AND ELECTRIC COMPANY

DOCKET NO. 50-317

CALVERT CLIFTS NUCLEAR POWER PLANT UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56 License No. DPR-53

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amondment by Baltimore Gas & Electric Company (the licensee) dated May 27, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and securicy or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-53 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 56, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective on May 27, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

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Robert A. Clark, Chief Operating Reactors Branch #3 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: June 23, 1981

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ATTACHER NT TO LICENSE AMENUMENT NO. 50

FACILITY OPERATING LICENSE NO. DPR-53

DOCKET NO. 50-317

Replace the following page of the Appendix A Technical Specifications with the enclosed page as indicated. The revised page is identified by Amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Page

3/4 3-41

TABLE 3.3-10

POST-ACCIDENT MUNITORING INSTRUMENTATION

22

56

ALVERT C	INSTRUMENT	MINIMUM CHANNELS OPERABLE
	1. Power Range Nuclear Flux	2
1 1	2. Containment Pressure	2
UNIT 1 2	3. Wide Range Logarithmic Neutron Flux Monitor	2
	4. Reactor Coolant Outlet Temperature	2
	5. Reactor Coolant Total Flow	2
3/4 3-41	6. Pressurizer Pressure	2
	7. Pressurizer Level	2
	8. Steam Generator Pressure	2/steam generator
	9. Steam Generator Level	2/steam generator
	10. Feedwater Flow	2
Ame	1. Auxiliary Feedwater Flow Rate	1/steam generator
ndment No. 5	12. RCS Subcooled Margin Monitor	1
	13. PORV/Safety Valve Acoustic Flow Monitoring	l/valve *
	14. PORV Solenoid Power Indication	1/valve

*Until June 1, 1981, the Unit 1 inoperable acoustic rlow monitor for pressurizer safety valve RV-201 may be replaced by observation of quench tank temperature, level and pressure and the safety valve tail pipe temperatures once per shift.

TABLE 4.3-10

CAL		POST-ACCIDENT MONITORING INSTRUM	MENTATION SURVEI	LLANCE REQUIREMENTS
VERT CL	INSTRUMENT		CHANNEL CHECK	CHANNEL CALIBRATION
IFFS - UNIT 1 3/4 3-42 Amendment No IFFS - UNIT 2	1.	Power Range Nuclear Flux	М	. Q
	2.	Containment Pressure	М	R
	3.	Wide Range Logarithmic Neutron Flux Monitor	М	ti.A.
	4.	Reactor Coolant Outlet Temperature	м	R
	5.	Reactor Coolant Total Flow	М	R
	6.	Pressurizer Pressure	М	R
	7.	Pressurizer Level	R	R
	8.	Steam Generator Cressure	М	R
	9.	Steam Generator Level	М	R
	10.	Feedwater Flow	М	R
	11.	Auxiliary Feedwater Flow Rate	М	R
	12.	RCS Subcooled Margin Monitor	М	R
	13.	PORV/Safety Valve Acoustic Monitor	N.A.	R
	14.	PORV Solenoid Power Indication	N.A.	Ν.Α.

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