UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84
License No. DPR-57

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Georgia Power Company, et al., (the licensee) dated April 28, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 84, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 7, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 84

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The overleaf page is provided to maintain document completeness.

Remove	Insert
3.6-11	3.6-11

Table 4.6-1

In-Service Inspection Program

CATEGORY	EXAMINATION AREA	EXAMINATION METHOD	INSPECTION DURING FIRST 5-YEAR INTERVAL	TENTATIVE INSPECTION DURING 10-YEAR INTERVAL
REACTOR VESSEL AND CLOSURE HEAD				
A	Longitudinal and circumferential shell welds in core region	Volumetric	None	5% of the length of each circumferential weld; 10% of the length of each longitudinal weld
B	Longitudinal and circumferential welds in the vessel shell and meridional and circumferential welds in vessel heads (other than those of Category A and C)	Volumetric	None	5% of the length of each circumferential weld; 10% of the length of each longitudinal weld
C	Vessel-to-flange and head-to-flange circumferential welds	Volumetric	1/3 of the vessel-to-flange and 1/3 of the head-to-circumferential weld	Cumulative 100% of the vessel-to-flange weld and of the head-to-flange weld
D	Primary nozzle-to-vessel welds and nozzle-to-vessel inside radiused section	Volumetric	Inspection of five nozzle-to-shell weld and inner nozzle radii	Inspection of all nozzle-to-shell welds and inner radius sections
E-2	Vessel penetrations, including control rod drive mechanism penetrations and control rod housing mechanism pressure boundary welds	Visual	10% of the control rod drive mechanism and instrumentation penetrations will be visually inspected for leakage	Cumulative 25% of the control rod drive mechanism and of the instrumentation penetrations will be visually inspected for leakage
F	Primary nozzles to safe end welds	Visual and surface and volumetric	The dissimilar weld on five nozzles	All of the dissimilar metal welds on the vessel nozzles will be examined
G-1	Closure studs and nuts	Volumetric and Visual	Cumulative 50%	Cumulative 100%

Table 4.6-1

In-Service Inspection Program

(Continued)

CATE-GORY	EXAMINATION AREA	EXAMINATION METHOD	INSPECTION DURING FIRST 5-YEAR INTERVAL	TENTATIVE INSPECTION DURING 10-YEAR INTERVAL
REACTOR VESSEL AND CLOSURE HEAD				
G-1	Ligaments between threaded stud holes	Volumetric	1/3 of the vessel-to-flange bolt ligaments will be examined	Cumulative 100% of the vessel flange bolt ligaments
G-1	Closure washers, bushings	Visual	Cumulative 50%	Cumulative 100%
H	Integrally welded vessel external supports	Volumetric	None	10% of the total weld length
I-1	Closure head cladding	Volumetric	None	The closure head is not clad. During the 10-year period, at least 6 points will be measured for thickness to determine the corrosion rate.
I-1	Vessel cladding	Visual	None	100% of selected areas at or near end of interval
N	Interior surfaces and internals and integrally welded internal supports	Visual	A critical examination will be made of the interior surface made available by normal refueling operations at the 1st refueling cycle. This will be repeated at the 4th refueling cycle with the amount of the inspection being dependent upon results of the 1st inspection and that made on other boiling-water systems	The inspections made at the 4th refueling cycle will be repeated at the 7th and 10th refueling cycle