

Georgia Power Company  
Post Office Box 442  
Baxley, Georgia 31513  
Telephone 912 367-6686  
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Edwin I. Hatch Nuclear Plant



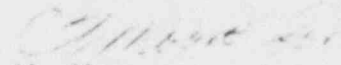
April 10, 1981  
PM-81-292

PLANT E. I. HATCH  
NRC Monthly Operating Report

Office of Plans and Schedules  
Directorate of Licensing  
United States Nuclear Regulatory Commission  
Washington, D. C. 20545

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC  
Monthly Operating Report for Hatch Unit 2, Docket #50-366.

  
M. Manry  
Plant Manager

CLC/pebc

8 10421 0543

R

OPERATING DATA REPORT

DOCKET NO. 50-366  
 DATE 04-10-81  
 COMPLETED BY GERRY ELAM  
 TELEPHONE (912) 367-7781 x 203

OPERATING STATUS

Unit Name: E I Hatch Nuclear Plant Unit 2  
 Reporting Period: 03-81  
 Licensed Thermal Power (MWe): 2436  
 Nameplate Rating (Gross MWe): 817.0  
 Design Electrical Rating (Net MWe): 784.0  
 Maximum Dependable Capacity (Gross MWe): 805.7  
 Maximum Dependable Capacity (Net MWe): 772.7  
 If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

\* Notes

Power Level To Which Restricted, If Any (Net MWe):  
 Reasons For Restrictions, If Any:

	This Month	Yr. -to-Date	Cumulative
Hours In Reporting Period	744	2160	13777
Number Of Hours Reactor Was Critical	503.0	723.7	8957.5
Reactor Reserve Shutdown Hours	0.0	0.0	0.0
Hours Generator On-Line	450.0	607.7	8296.0
Unit Reserve Shutdown Hours	0.0	0.0	0.0
Gross Thermal Energy Generated (MWH)	898374	1166526	10264424
Gross Electrical Energy Generated (MWH)	293250	380910	6048190
Net Electrical Energy Generated (MWH)	277132	353926	5756000
Unit Service Factor	60.5	21.1	60.2
Unit Availability Factor	10.0	18.1	60.2
Unit Capacity Factor (Using RDC Net)	48.2	21.2	54.1
Unit Capacity Factor (Using DER Net)	47.5	20.9	53.3
Unit Forced Outage Rate	39.5	36.7	14.3

Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

If Shut Down At End Of Report Period, Estimated Date of Startup:  
 Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	-----	-----
INITIAL ELECTRICITY	-----	-----
COMMERCIAL OPERATION	-----	-----

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-367  
DATE 04-10-81  
COMPLETED BY GERRY ELAN  
TELEPHONE (912) 367-7781 x 203

H 03-81

AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
163	17	279
599	18	322
695	19	682
367	20	777
147	21	548
205	22	575
-11	23	739
-10	24	785
-11	25	790
-11	26	786
-8	27	779
-6	28	546
-7	29	387
11	30	433
21	31	439
536		

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-366  
 UNIT NAME Hatch 2  
 DATE 4-10-81  
 COMPLETED BY G. H. Flam  
 TELEPHONE 912-367-7851

REPORT MONTH March

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
81-16	810301	F	14.3	B	3	NA	HA	VALVEX	Rx scram due to turbine stop valve fast closure
81-17	810301	F	19.6	B	5	NA	HA	VALVEX	Unit on startup ramp from above shutdown; #81-16
81-18	810304	F	22.3	A	3	81-015	IA	INSTRU	Rx scram due to IRI spike
81-19	810305	F	23.5	A	5	NA	IA	INSTRU	Recovery from above scram #81-18
81-20	810306	F	6.0	A	5	NA	CB	GENERA	Power reduction due to exciter problem of recirc pump 'A'
81-21	810306	F	196.3	A	1	81-017	CB	GENERA	Unit off line due to recirc pump 'A' exciter problems
81-22	810314	F	4.4	A	5	NA	DB	GENERA	Unit on startup from above shutdown #81-21

<sup>1</sup> F: Forced  
S: Scheduled

<sup>2</sup> Reason:  
A-Equipment Failure (Explain)  
B-Maintenance of Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & License Examination  
F-Administrative  
G-Operational Error (Explain),  
H-Other (Explain)

<sup>3</sup> Method:  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuations  
5-Load Reduction  
9-Other (Explain)

<sup>4</sup> Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup> Exhibit I - Same Source

**UNIT SHUT DOWNS AND POWER REDUCTIONS**

DOCKET NO. 50-366  
 UNIT NAME Hatch 2  
 DATE 4-10-81  
 COMPLETED BY G. H. Flam  
 TELEPHONE 912-367-7851

REPORT MONTH March

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
81-24	810315	F	18.9	A	3	NA	IA	INSTRU	Rx scram due to IRMs C&F spiked upscale
81-25	810315	F	24.1	A	5	NA	IA	INSTRU	Recovery from above scram 21-24
81-26	810317	F	16.6	A	3	NA	CD	VALVEX	Rx scram due to MSIV closure
81-27	810318	F	27.7	A	5	NA	CD	VALVEX	Recovery from above scram #81-26
81-28	810321	F	30.8	B	5	NA	HA	ZZZZZ	Load reduction to perform turbine testing
81-29	810327	S	5.5	F	5	NA	RB	CONROD	Load reduction to perform weekly turbine testing and rod pattern adjustment
81-30	810328	F	11.3	A	3	NA	CH	HTEXCH	Rx scram due to loss of feedwater heaters

<sup>1</sup> F: Forced  
S: Scheduled

<sup>2</sup> Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

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POOR ORIGINAL



### UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March

DOCKET NO. 50-366  
 UNIT NAME Hatch 2  
 DATE 4-10-81  
 COMPLETED BY G. H. Elam  
 TELEPHONE 912-367-7851

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
81-31	810329	F	10.0	A	5	NA	CH	TEXCH.	Recovery from above scram #81-30
81-32	810330	F	14.3	A	3	NA	HA	GENERA	Rx scram due to high water level caused by turbine trip
81-33	810331	F	8	A	5	NA	HA	GENERA	Recovery from above scram #31-32

**1**  
**F: Forced**  
**S: Scheduled**

**2**  
 Reason:  
 A-Equipment Failure (Explain)  
 B Maintenance of Test  
 C Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

**3**  
 Method:  
 1-Manual  
 2-Manual Scram.  
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NARRATIVE REPORT  
UNIT 2

March 1st Rx auto scram due to TSV fast closure at 0306

March 1st Rx critical at 1142, tied to line at 1724

March 4th Rx auto scram at 1142 due to IRM spike

March 4th Rx critical at 2046, tied to line at 1000 hours on March 5th

March 6th Reducing power at 0915 due to exciter problems of recirc pump A

March 6th Unit off line due to recirc pump A exciter problems

March 14th Rx critical at 0300, on line at 1938 hours

March 15th Rx auto scram at 0622 due to IRM 'C' and 'F' spiked upscale

March 15th Rx critical at 1136 and generator on line at 1855 hours

March 17th Rx auto scram at 0938 due to MSIV closure

March 17th Rx critical at 1745 hours and generator on line at 0216 hours on March 18th

March 21st Power reduced at 1110 for weekly turbine test

March 27th Power reduced at 2210 for weekly turbine test and rod pattern adjustment

March 28th Rx auto scram due to loss of feedwater heaters A&B

March 29th Rx critical at 0045, tied to line at 0602 hours

March 30th Rx auto scram due to high water level caused by turbine trip

March 30th Rx critical at 2114, on line at 0411 hours of March 31st

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS  
TO BE REPORTED FOR MARCH 1981

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
79-4584	2-17-81	Replaced (per DCR 79-32) downstream side of flanges, gaskets, and all connections between flanges and manifolds, field run piping to accommodate new MSIV air supply manifolds 2B21-F022 (A-D and F028A-D)
80-4532	1-20-81	Added unistrut, fittings, and pipe clamp at location W0639 and 18511 to 2C11-H-157 CBD piping, supports
81-1241	2-22-81	Changed core bottom limit to 698 on 2C51N001B B tip machine
80-3414	3-4-81	Replaced leaking outboard seal on main HPCI pump 2E41-C001
81-54	1-21-81	Removed and cleaned 2E41-F021 HPCI turbine exhaust manual valves
81-1146	3-4-81	Replaced and calibrated HPCI, EGM, EGR, ramp gen. and signal converter 2E41-R674
81-795	2-18-81	Fabricated and installed (per DCR 81-28) RCIC pipe support 2E51-RCIC-R701
81-1231	2-25-81	Repaired ribbon connector on RCIC - 2E51-C001
81-532	2-9-81	Removed and cleaned valves on 2551-F001 and F002 torus drain valve
80-4588	2-17-81	Removed and replaced INS Co. snubber with (per DCR 80-16) Pacific Scientific Co. snubbers 2L3J
80-4592	2-18-81	Removed and replaced INS Co. snubber with (per DCR 80-16) Pacific Scientific Co. snubbers 2L3J



80-4593	2-17-81	Removed and replaced INS Co. snubber with (per DCR 80-16) Pacific Scientific Co. snubbers 2L3J
80-4597	2-17-81	Removed and replaced INS Co. snubber with (per DCR 80-16) Pacific Scientific Co. snubbers 2L3J
80-4718	2-17-81	Removed and replaced INS Co. snubber with (per DCR 80-16) Pacific Scientific Co. snubbers 2L3J
80-4872	2-17-81	Removed and replaced INS Co. snubber with (per DCR 80-16) Pacific Scientific Co. snubbers 2L3J
80-5211	2-17-81	Removed and replaced INS Co. snubber with (per DCR 80-16) Pacific Scientific Co. snubbers 2L3J
80-3299	2-10-81	Installed 2P11-C001A&B condensate transfer pump enclosure per DCR 79-190, R1
81-728	2-11-81	Removed seal plate from exterior end of standby diesel service water pipe (2P41-600) and prepare for liquid penetrant examination
81-738	3-12-81	Removed and replaced seal gasket on 2P41-F410 hydrogen cooler PSW air relief valve
81-941	2-20-81	Removed seal plate from exterior end of standby diesel service water pipe 2P41-G001 and prepare for liquid penetrant exam
81-776	2-17-81	Delete eight existing pipe supports (per DCR 80-453) and replace with 4 cooling water system pipe supports 2P42
81-125	2-16-81	Pulled circuits TLX701C01 and TLX702C01 (per DCR 78-268) in reactor building Unit 2, 2T43

81-458	2-17-81	Removed existing conduits, installed pull bar and reinstalled conduits and pull cables (per DCR 78-135) in reactor HVAC unit 2 fire protection 2T43
81-902	2-17-81	Removed existing conduits, installed pull bar and reinstalled conduits and pull cables (per DCR 78-135) in reactor HVAC unit 2 fire protection 2T43
81-400	2-16-81	Installed instruments 2T47-N009 and 2T47-N003, circuits A's TFX922C09 and TFX023C01 on drywell cooling local RTD, 2T47
81-896	2-13-81	Terminate and redline TLX806M01 and TLA90F15 and calibrate 2Z43-P007 Alison Control Fire Detection Panel