



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 17, 1981

Docket No. 50-219
LS05-81-04-017

Mr. I. R. Finfrock, Jr.
Vice President
Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731



Dear Mr. Finfrock:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR SEP TOPIC "VI-4,
CONTAINMENT ISOLATION SYSTEMS (OYSTER CREEK)

During our review of SEP Topic VI-4, "Containment Isolation Systems" we found that additional information (see enclosed) is needed to complete our review. Provide your response within 30 days of the receipt of this letter so that our evaluation can be completed in a timely manner.

Sincerely,

Dennis M. Crutchfield
Dennis M. Crutchfield, Chief
Operating Reactors Branch No. 5
Division of Licensing

Enclosure:
Request for Additional
Information

cc w/enclosure:
See next page

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Mr. I. R. Finrock, Jr.

cc

G. F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

GPU Service Corporation
ATTN: Mr. E. G. Wallace
Licensing Manager
260 Cherry Hill Road
Parsippany, New Jersey 07054

Natural Resources Defense Council
917 15th Street, N. W.
Washington, D. C. 20006

Steven P. Russo, Esquire
248 Washington Street
P. O. Box 1060
Toms River, New Jersey 08753

Joseph W. Ferraro, Jr., Esquire
Deputy Attorney General
State of New Jersey
Department of Law and Public Safety
1100 Raymond Boulevard
Newark, New Jersey 07012

Ocean County Library
Brick Township Branch
401 Chambers Bridge Road
Brick Town, New Jersey 08723

Mayor
Lacey Township
P. O. Box 475
Forked River, New Jersey 08731

Commissioner
Department of Public Utilities
State of New Jersey
101 Commerce Street
Newark, New Jersey 07102

Gene Fisher
Bureau Chief
Bureau of Radiation Protection
380 Scotts Road
Trenton, New Jersey 08628

Commissioner
New Jersey Department of Energy
101 Commerce Street
Newark, New Jersey 07102

Plant Superintendent
Oyster Creek Nuclear Generating
Station
P. O. Box 388
Forked River, New Jersey 08731

Resident Inspector
c/o U. S. NRC
P. O. Box 445
Forked River, New Jersey 08731

Director, Criteria and Standards
Division
Office of Radiation Programs
(ANR-460)
U. S. Environmental Protection
Agency
Washington, D. C. 20460

U. S. Environmental Protection
Agency
Region II Office
ATTN: EIS COORDINATOR
26 Federal Plaza
New York, New York 10007

Request for Additional Information
SEP Topic VI-4
Oyster Creek, Unit 1
Docket No. 50-219

- 1 - The following references represent the information being used for containment isolation review:
 - a. Oyster Creek Nuclear Power Plant, Unit No. 1, Facility Description and Safety Analysis Report, Volume 1
 - b. JCP&L letter of 12/14/79 in response to Bulletin No. 79-08.
 - c. JCP&L letter of 4/10/80 in response to NUREG-0578 ImplementationIf the above documentation does not properly reflect the as-built condition, provide the necessary references.
- 2 - Provide current P&ID drawings on the following systems which penetrate primary containment and are classified as essential or nonessential.

Essential

- a. Emergency Condenser System
- b. Control Rod Drive System
- c. Core Spray System
- d. Containment Spray System
- e. Torus to Reactor Building Vacuum Breaker System

Nonessential

- a. Main Steam System
- b. Feedwater System
- c. Reactor Building Closed Cooling Water System
- d. Instrument Air System
- e. Demineralized Water System
- f. Reactor Cleanup System

ENCLOSURE

- g. Shutdown Cooling System
- h. Liquid Poison System
- i. Drywell Equipment Drain Tank
- j. Drywell Sump
- k. Drywell and Torus Atmosphere Control Systems
- l. Reactor Recirculation Loop Sample System
- m. Reactor Head Cooling System
- n. Nitrogen System
- o. Traversing In-Core Probe System

3 - SRP Section 6.2.4, II.3.f states that sealed closed barriers may be used in place of automatic isolation valves, and should be under administrative control to assure that they cannot be inadvertently opened. Sealed closed barriers include capped branch lines, test connections and vent lines. Discuss the administrative controls applied to such barriers and note methods of administrative control.

Examples as obtained from Reference 1b include the following:

<u>Figure</u>	<u>Valve Nos.</u>
B (Isolation Condenser Return)	V-14-28
	V-14-39
F (Shutdown Cooling Supply)	V-17-65
	V-17-21
H (Cleanup Supply)	V-16-4
I (Cleanup Return)	V-16-65
J (Core Spray System)	V-20-42
	V-20-44
P (Recirculation Loop Sample)	V-24-28
Q (Steam Line Drain)	V-7-137
S (Containment Spray)	V-6-134
	V-6-136

Y (Vacuum Breakers & Associated Piping, Vent Pipe between Drywell and Torus)	V-6-138
Z (Core Spray and Containment Spray Sup- ply From Torus)	V-20-29

- 4 - Reference 1b provides a table of isolation information for systems which penetrate primary containment. This table indicates that valves V-20-21, V-20-41, V-21-15 and V-21-18 are automatic isolation valves; however, no isolation signal code is provided. Provide the signals which are used to initiate these valves. This table also indicates that valves V-20-26 and V-20-27 are remote manual valves and provides an isolation signal code. Provide an explanation for this inconsistency.
- 5 - Figure M of Reference 1b indicates that valve V-12-60 of the demineralized water system is closed in normal, shutdown and post accident conditions. Discuss the administrative controls applied to this system to assure that it cannot be inadvertently opened.
- 6 - Provide current P&ID drawings of instrument lines penetrating the primary containment as noted in Reference 1b. Indicate how these systems are provided with isolation provisions in accordance with Regulatory Guide 1.11.
- 7 - SRP Sections 6.2.4, II.3.b and c state that containment isolation provisions for lines in engineered safety features or engineered safety feature-related systems as well as systems needed for safe shutdown of the plant may include remote-manual valves, but provisions should be made to detect possible leakage from these lines outside containment. Discuss what provisions are provided for leakage detection from such systems, examples of which include the liquid poison system, shutdown cooling system, containment spray supply and core spray supply from the torus.